STONE & WEBSTER ENGINEERING CORPORATION

245 SUMMER STREET, BOSTON, MASSACHUSETTS



ADDRESS ALL CORRESPONDENCE TO P.O. BOX 2325. BOSTON. MASS. 02107

NEW YORK BOSTON CHICAGO HOUSTON LOS ANGELES DENVER CHERRY HILL, N.J. PORTLAND, CREGON DESIGN CONSTRUCTION REPORTS EXAMINATIONS CONSULTING ENGINEERING

December 12, 1978

Nuclear Regulatory Commission Division of Project Management Washington, DC 20555

Attention: Mr. Walter P. Haass

Gentlemen:

In response to your letter of November 22, 1978, Subject "NRC Acceptance of Revised Stone & Webster Engineering Corporation Topical Report on Quality Assurance", we are pleased to submit 40 copies of Revision C to SWSQAP 1-74A for your information and use.

We will continue, as directed, to report organizational changes no later than 30 days after announcement.

Revision C has not been adopted by any of our Clients at this time; however, we have corresponded with all clients involved in the design and construction of nuclear stations to offer them the benefits of Pevision C. We will advise you of our Client's responses on or before June 1, 1979.

Please advise us if further information is desired.

Very truly yours .-ORIGINAL SIGNED E.T. Trainor Manager, Quality Assurance

Attachment

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UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

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Mr. E. F. Trainor, Manager Quality Assurance Department Stone & Webster Engineering Corporation P.O. Box 2325 Boston, Massachusetts 02107

Dear Mr. Trainor:

SUBJECT: NRC ACCEPTANCE OF REVISED STONE & WEBSTER ENGINEERING CORPORATION TOPICAL REPORT ON QUALITY ASSURANCE

By letters dated March 8, August 31, and November 17, 1978, you submitted Revision C to the Stone & Webster Engineering Corporation topical report, SWSQAP 1-74A, "Standard Nuclear Quality Assurance Program," for staff review and evaluation. Revision C revises the report to reflect organizational, procedural, and editorial changes.

Based on our review and evaluation of Revision C, we find that the applicable requirements of Appendix B to 10 CFR Part 50 are included in the Quality Assurance program description. Therefore, your revised report is acceptable, and you may implement it upon issuance of the revision. To use the report in applications, applicants need only reference it in Section 17 of the Safety Analysis Report. We do not intend to repeat our review of this report when it is referenced in an application.

Should regulatory criteria or regulations change such that our conclusions about this topical report are invalidated, we will notify you. You will be given the opportunity to revise and resubmit it should you so desire. Programmatic changes by Stone & Webster Engineering Corporation to this report are to be submitted to NRC for review prior to implementation. Organizational changes are to be submitted no later than 30 days after announcement.

Please replace our letter of August 26, 1976 with a copy of this letter, renumber the report as SWSQAP 1-74A, Revision C, and transmit 40 copies to the NRC. In your transmittal letter, indicate to which plants Revision C will be applicable.

Should you have any questions regarding our review or if we can provide assistance, please feel free to contact me or Jack Spraul at (301) 492-7741.

Sincerely, da. Walter P. Haass, Chief

Quality Assurance Branch Division of Project Management

Enclosure: Topical Report Evaluation

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TOPICAL REPORT EVALUATION

Report Number: SWSQAP 1-74, Nonproprietary Report Title: Standard Nuclear Quality Assurance Program Report Date: December, 1974. Revision Date: December 1, 1978 Originating Organization: Stone & Webster Engineering Corporation Reviewed By: Quality Assurance Branch

SUMMARY OF TOPICAL

Topical Report SWSQAP 1-74A describes the quality assurance (QA) program which the stone & Webster Engineering Corporation (S&W) applies to those design, procurement, and construction activities involving safety-related structures, systems, and components of nuclear power plants within the S&W scope of work. SWSQAP 1-74A commits S&W to comply with the requirements of Appendix B to 10 CFR Part 50 and with some exceptions which the staff has found acceptable, to comply with the QA guidance provided by the NRC in Regulatory Guides 1.25 (March 1978), 1.30 (August 1972), 1.37 (March 1973), 1.38 (May 1977), 1.39 (September 1977), 1.58 (August 1973), 1.64 (June 1976), 1.74 (February 1974), 1.88 (October 1976), 1.94 (April 1976), 1.116 (June 1976), and 1.123 (July 1977) as well as ANSI Standard N45.2.12 (Draft 3, Revision 4, February 1974).

S&W has provided for our evaluation a detailed organizational description of those individuals and groups involved in carrying out activities required by the QA program and a delineation of duties, responsibilities, and authority of those organizational elements involved in the QA program. SWSQAP 1-74A contains a description of the measures used to carry out the S&W QA program activities and describes how applicable requirements of Appendix B will be satisfied by the administration and implementation of these measures.

SUMMARY OF REGULATORY EVALUATION

We have evaluated the QA program and the organizations responsible for QA functions as described in SWSQAP 1-74. We find that QA policy and direction originate at an acceptably high management level and are effectively communicated to other parts of the organization. Those performing QA functions have responsibility and authority commensurate with their duties in implementing the QA program. We also find that measures have been established, to be implemented by written procedures and instructions, which address each of the criteria of Appendix B in an acceptable manner.

Based on our review and evaluation of SWSQAP 1-74A, we conclude that:

1. The organizations and persons performing QA functions within S&W have the required independence and authority to effectively carry out the QA program without undue influence from those directly responsible for costs and schedules, and

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2. The S&W QA program contains requirements, procedures, and controls which, when properly implemented, comply with the requirements of Appendix B to 10 CFR Part 50 and the applicable regulatory guides and ANSI standards contained in Section 17 of the NRC Standard Review Plan.

REGULATORY POSITION

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It is the staff's position that the S&W topical report SWSQAP 1-74A, "Standard Nuclear Quality Assurance Program" is acceptable for use in the design, procurement, and construction of nuclear power plants. The topical report can be referenced by report number in Section 17 of future Safety Analysis Reports.