AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 86-354

UNIT Hope Creek

DATE 1/15/88

COMPLETED BY H. Jensen

TELEPHONE (609) 339-5261

MONTH December 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1026	17	1015
2	1007	18	1000
3	1014	19	952
4	1031	20	978
5	991	21	1099
6	1023	22	994
7	1013	23	966
8	588	24	1003
9	0	25	1002
10	868	26	955
11	1017	27	1012
12	1003	28	1029
13	1009	29	1023
14	1017	30	1018
15	1007	31	1026
16	1006		

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OPERATING DATA REPORT

DOCKET NO. 86-354
UNIT Hope Creek
DATE 1/15/88
COMPLETED BY H. Jensen
TELEPHONE (609) 339-5261

OPERATING STATUS

1. REPORTING PERIOD Dec 1987 GROSS HOURS IN REPORTING PERIOD 744

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt) 3293
   MAX. DEPEND. CAPACITY (MWe-Net) 1067 *
   DESIGN ELECTRICAL RATING (MWe-Net) 1067

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net) None

4. REASONS FOR RESTRICTION (IF ANY)

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NO. OF HOURS REACTOR WAS CRITICAL	713.9	7570.1	7858.1
6. REACTOR RESERVE SHUTDOWN HOURS	0	0	0
7. HOURS GENERATOR ON LINE	709.4	7457.1	7745.1
8. UNIT RESERVE SHUTDOWN HOURS	0	0	0
9. GROSS THERMAL ENERGY GENERATED (MWH)	2,309,491	22,878,159	23,808,567
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	770,718	7,614,058	7,911,698
11. NET ELECTRICAL ENERGY GENERATED (MWH)	739,066	7,279,214	7,565,038
12. REACTOR SERVICE FACTOR	96.0	86.4	86.8
13. REACTOR AVAILABILITY FACTOR	96.0	86.4	86.8
14. UNIT SERVICE FACTOR	95.3	85.1	85.6
15. UNIT AVAILABILITY FACTOR	95.3	85.1	85.6
16. UNIT CAPACITY FACTOR (Using Design MDC)	93.1	77.9	78.4
17. UNIT CAPACITY FACTOR (Using Design MWe)	93.1	77.9	78.4
18. UNIT FORCED OUTAGE RATE	4.7	9.3	9.0

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, & DURATION):
    Refueling 2/12/88, 55 days

20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

* Data obtained in August is under management review.

OPERATING DATA REPORT

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 86-354

UNIT Hope Creek

DATE 1/15/88

COMPLETED BY R. Ritzman

REPORT MONTH Dec. 1987 TELEPHONE (609) 339-3737

NO.	DATE	TYPE F FORCED S SCHEDULED	DURATION (HOURS)	REASON (1)	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER (2)	CORRECTIVE ACTION/ COMMENTS
21	12/8	F	34.6	A	3	Reactor Scram caused by a spurious signal in the Main Steam Line Radiation Monitoring cabinets LER 87-051 1/6/88

SUMMARY

HOPE CREEK GENERATING STATION
MONTHLY OPERATING SUMMARY
DECEMBER 1987

Hope Creek entered the month of December operating at approximately 100% power. At 2:05 pm on December 8, the reactor automatically scrammed due to a spurious signal in the Main Steam Line Radiation Monitoring cabinets. The unit had been on-line for 41 consecutive days. The reactor went critical at 8:12 pm on December 9 and the generator was synchronized with the grid at 12:43 am on December 10. At months end the plant completed its 21st day of continuous power operation.

R-005
RAR:tlb

SUMMARY OF CHANGES, TESTS, AND EXPERIMENTS

FOR THE HOPE CREEK GENERATING STATION

DECEMBER 1987

The following Design Change Packages (DCPs) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the DCPs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These DCPs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

DCP	Description of Design Change Package
4-HC-0019/1	This DCP rewired Control Room Overhead Annunciator Windows to allow the annunciator to clear when the corresponding remote panel alarm has been acknowledged. This will eliminate nuisance alarms in the Control Room and reduce confusion by clearing alarms when they are no longer required to be displayed.
4-HC-0019/2	This DCP rewired Control Room Overhead Annunciator Windows to allow the annunciator to clear when the corresponding remote panel alarm has been acknowledged. This will eliminate nuisance alarms in the Control Room and reduce confusion by clearing alarms when they are no longer required to be displayed.
4-HM-0147	This DCP installed terminals in the Radiation Protection offices and the Operational Support Center to allow personnel to expeditiously obtain radiological and meteorological data during emergency conditions.
4-HM-0214	This DCP replaced a Reactor Protection System Scram Reset Relay with a functionally equivalent relay. The original style relay is no longer available.
4-HM-0216	This DCP modified the ICD cards and applicable drawings to allow the use of either of two logic cards for the Service Water Pump Lubricating Water Low Pressure Switch and the valve which allows Head Tank flow in the event of Low Lubricating Water Pressure. This change eliminates a Temporary Modification and allows for more flexibility when replacement parts are required.
4-HM-0252	This DCP changed the Main Steam Line Radiation Monitor Alarm and Trip Setpoints to be consistent with the measured full power background radiation monitor readings. The "B" and "C" Main Steam Line Radiation Monitors have been changed, "A" and "D" will be changed at a later date.
4-HM-0255	This DCP installed stiffeners on the flow distribution vanes in the transition duct between the Filtration, Recirculation and Ventilation System recirculation fan and its filter housing. Adding the stiffeners will eliminate the pressure pulsations which fatigued the duct, causing cracking at the area where the vane is welded to the duct.

The following Temporary Modification Requests (TMRs) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the TMRs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These TMRs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Safety Evaluation	Description of Temporary Modification Request (TMR)
87-0197	This TMR replaced resistors in the Filtration, Recirculation and Ventilation Radiation Monitoring System. This TMR was necessitated by the failure of a flow sensor in the Filtration, Recirculation and Ventilation System. Normally, the Radiation Monitors average 2 signals, however, with 1 of the signals inoperable this is not possible. This TMR allows the use of the valid signal rather than the average.
H-1-GUXX-MSE-0722	This TMR temporarily provides for the adequate filtration, recirculation, and ventilation of the Reactor Building. The TMR is required while ductwork in the Reactor Building Ventilation System is being replaced. The Filtration, Recirculation and Ventilation System will be able to provide Reactor Building cooling and be able to maintain negative pressure there during this time frame.

The following Deficiency Requests (DRs) have been evaluated to determine:

1) if the probability of occurrence or the consequences of an accident or malfunction of equipment important to safety previously evaluated in the safety analysis report may be increased; or

2) if a possibility for an accident or malfunction of a different type than any evaluated previously in the safety analysis report may be created; or

3) if the margin of safety as defined in the basis for any technical specification is reduced.

None of the DRs created a new safety hazard to the plant nor did they affect the safe shutdown of the reactor. These DRs did not change the plant effluent releases and did not alter the existing environmental impact. The Safety Evaluations determined that no unreviewed safety or environmental questions are involved.

Safety Evaluation	Description of Deficiency Report (DR)
87-0110	The insulation on Emergency Diesel Generator Combustion Air High Temperature Switch Alarm wire leads was discovered to be damaged. Repairing the insulation via the Raychem process will restore the insulation to its original integrity and allow the switch to operate as designed.
87-0194	Inter-step connection cables installed on the 125 VDC Class 1E Battery Bank were discovered to be out of compliance with the design documents. Additionally, they were not certified to the applicable standard. However, the cables are capable of performing their required function until they can be replaced.
87-0199	Emergency Diesel Generator Exhaust Silencer Inlet Expansion Joints show signs of distortion and possible leakage. The extent of leakage will be evaluated. The expansion joints may be used "as is" until the evaluations are complete and the required parts are available.
87-0201	A two inch wye pattern globe valve branching from the "C" Main Steam Line developed a packing leak. Drilling a small hole in the packing gland and injecting sealant to stop the leak will allow the valve to be used "as is" until it can be repaired.

PSE&G

Public Service Electric and Gas Company P.O. Box L Hancocks Bridge, New Jersey 08038

Hope Creek Operations

January 15, 1988

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
HOPE CREEK GENERATING STATION UNIT 1
DOCKET NO. 50-354

In compliance with Section 6.9, Reporting Requirements for the Hope Creek Technical Specifications, the operating statistics for December are being forwarded to yo" In addition, the summary of changes, tests, and experiments for December 1987 are included pursuant to the requirements of 10CFR50.59(b).

Sincerely yours,

S. LaBruna
General Manager -
Hope Creek Operations

RAR:tlb
Attachment

C Distribution

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