

CLEVELAND ELECTRIC ILLUMINATING COMPANY

PERRY NUCLEAR POWER PLANT

December 1, 1987 - December 31, 1987

MONTHLY OPERATING REPORT TO NRC

DOCKET NUMBER: 50-440

LICENSE NUMBER: NPF-58

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December 1987 Operations Summary

The Perry Plant entered the month of December, 1987 in a shutdown condition as a result of a Main Steam Isolation Valve failure on November 29, 1987. The plant remained shutdown until investigations were completed and corrective actions implemented. The reactor was restarted and the generator synchronized on December 9. Prior to continued operations at high power levels, the generator was temporarily removed from service for repair of a Moisture Separator Reheater Drain Valve Controller, and was again synchronized on December 10. The plant was operated at 95% power until December 20, when the generator was removed from service for weld repairs to a turbine test connection. Neither of these repair activities required the reactor to be shutdown. The generator was synchronized again on December 20, and the plant remained at 95% power through the end of the month.

Average Daily Unit Power Level

DOCKET NO. 50-440
UNIT Perry-One
DATE 1/15/88
Contact G.A.Dunn
Telephone (216)259-3737

MONTH December 1987

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	<u>0</u>	17	<u>1118</u>
2	<u>0</u>	18	<u>1117</u>
3	<u>0</u>	19	<u>1080</u>
4	<u>0</u>	20	<u>479</u>
5	<u>0</u>	21	<u>1119</u>
6	<u>0</u>	22	<u>1121</u>
7	<u>0</u>	23	<u>1120</u>
8	<u>0</u>	24	<u>1119</u>
9	<u>0</u>	25	<u>1127</u>
10	<u>49</u>	26	<u>1129</u>
11	<u>666</u>	27	<u>1056</u>
12	<u>1108</u>	28	<u>1118</u>
13	<u>1078</u>	29	<u>1120</u>
14	<u>1118</u>	30	<u>1121</u>
15	<u>1116</u>	31	<u>1118</u>
16	<u>1118</u>		

INSTRUCTIONS

On this form list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

Operating Data Report

DOCKET NO. 50-440
DATE 1/15/88
CONTACT G.A.Dunn
TELEPHONE (216)259-3737

OPERATING STATUS

1. Unit Name: Perry-One
2. Reporting Period: December 1987
3. Licensed Thermal Power (MWt): 3579
4. Nameplate Rating (Gross MWe): 1250
5. Design Electrical Rating (Net MWe): 1205
6. Maximum Dependable Capacity (Gross MWe): 1250 (Estimated)
7. Maximum Dependable Capacity (Net MWe): 1205 (Estimated)
8. If Changes Occur in Capacity Ratings (Item Numbers 3 through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): 100% *
10. Reasons for Restrictions, If Any: Technical Specification Limit

	This Month	Yr-to-Date	Cumulative
11. Hours In Reporting Period	<u>744</u>	<u>1,044</u>	<u>1,044</u>
12. Hours Reactor Was Critical	<u>548.3</u>	<u>811.3</u>	<u>811.3</u>
13. Reactor Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
14. Hours Generator On-Line	<u>510.6</u>	<u>773.4</u>	<u>773.4</u>
15. Unit Reserve Shutdown Hours	<u>0</u>	<u>0</u>	<u>0</u>
16. Gross Thermal Energy (MWH)	<u>1,647,252</u>	<u>2,561,519</u>	<u>2,561,519</u>
17. Gross Electrical Energy (MWH)	<u>565,226</u>	<u>878,462</u>	<u>878,462</u>
18. Net Electrical Energy (MWH)	<u>531,078</u>	<u>828,484</u>	<u>828,484</u>
19. Unit Service Factor	<u>68.6</u>	<u>74.1</u>	<u>74.1</u>
20. Unit Availability Factor	<u>68.6</u>	<u>74.1</u>	<u>74.1</u>
21. Unit Capacity Factor (Using MDC Net)*	<u>59.2</u>	<u>65.9</u>	<u>65.9</u>
22. Unit Capacity Factor (Using DER Net)*	<u>59.2</u>	<u>65.9</u>	<u>65.9</u>
23. Unit Forced Outage Rate	<u>31.4</u>	<u>25.9</u>	<u>25.9</u>
24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):			
<u>Surveillance/Maintenance Outage beginning January 3, 1988, Duration 19 days;</u>			
<u>Surveillance/Maintenance Outage estimated start March 11, 1988 estimated</u>			
<u>duration 10 days</u>			
25. If Shut Down at end of Report Period, Estimated Date of Startup:			

N/A

NOTE: Year-To-Date and cumulative data provided in this Operating Data Report reflects the period from 1200 hours on November 18, 1987 when commercial operation began, until 2400 hours on December 31, 1987. This is in accordance with the standard practice of providing cumulative data based on the date of commercial operation rather than the date of first electrical generation.

* Maximum Power Operation was limited due to the unavailability of the 'B' Final Stage Feedwater Heater and the 'A' Reactor Feedwater Pump. (NOTE: The Final Stage Feedwater Heater was erroneously identified as 'A' in the November, 1987 Operating Data Report.)

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH December 1987DOCKET NO. 50-440UNIT NAME Perry-OneDATE 1/15/88COMPLETED BY G.A. DunnTELEPHONE (216) 259-3737

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code ⁴	Component Code ⁵	Cause & Corrective Action to Prevent Recurrence
1	12/01/87	F	214.5	A D	4				Continuation of shutdown performance 11/29/87; refer to 11/87 Monthly Operating Report.
2	12/10/87	F	11.6	A	4				Automatic turbine trip due to high MSR drain tank level (Reactor was not shutdown). Drain tank level controller was repaired.
3	12/20/87	F	7.3	A	4				Turbine taken off-line due to a steam leak upstream of an instrument test cap (Reactor was not shutdown). The line was cut, capped and seal welded.

¹ F: Forced
S: Scheduled

² Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training &
License Examination
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

³ Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Other (Explain)

⁴ Exhibit G - Instructions
for Preparation of Data
Entry Sheets for Licensee
Event Report (LER) File (NUREG
1022)

⁵ Exhibit I-Same Source



THE CLEVELAND ELECTRIC ILLUMINATING COMPANY

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Serving The Best Location in the Nation
PERRY NUCLEAR POWER PLANT

Al Kaplan

VICE PRESIDENT
NUCLEAR GROUP

January 15, 1988
PY-CEI/NRR-0767 L

Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D. . 20555

Perry Nuclear Power Plant
Docket No. 50-440
Monthly Operating Report

Dear Sir:

Attached is the December 1987 Monthly Operating Report for Perry Unit 1. This report is submitted in accordance with Technical Specification 6.9.1.8.

Please feel free to contact me if you have any questions.

Very truly yours,

Al Kaplan
Vice President
Nuclear Group

AK:cab

Attachment

cc: T. Colburn
K. Connaughton
USNRC, Region III
Director, Office of Resource Management

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