

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

NEBRASKA PUBLIC POWER DISTRICT

DOCKET NO. 50-298

COOPER NUCLEAR STATION

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 52 License No. DPR-46

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment by Nebraska Public Power District dated July 22, 1977, as supplemented by letters dated November 30, December 16, 1977, June 12, 28, July 5 and 14, 1978, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I;
 - The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public: and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment and paragraph 2.C(2) of Facility License No. DPR-46 is hereby amended to read as follows:
 - (2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 52, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

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3. This license amendment is effective as of the date of its issuance.

* FOR THE NUCLEAR REGULATORY COMMISSION

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Brian K. Grimes, Assistant Director for Engineering and Projects Division of Operating Reactors

Attachment: Changes to the Technical Specifications

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Date of Issuance: SEPTEMBER : 9 1978

ATTACHMENT TO LICENSE AMENDMENT NO. 52

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FACILITY OPERATING LICENSE NO. DPR-46

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Remove page 218 of Appendix A and replace with revised page 218. Marginal lines indicate area of change.

5.4.C (cont'd.)

penetrations shall be designed in accordance with standards set forth in Section V-2.3.4 of the SAR.

5.5 Fuel Storage

- A. The new fuel storage facility shall be such that the K dry is less than 0.90 and flooded in less than 0.95.
- B. The spent fuel storage racks are designed and shall be maintained with a nominal 6 9/16 inch center-to-center distance between fuel assemblies placed in the storage racks to ensure a K_{eff} equivalent to \leq 0.95 with the storage pool filled with unborated water. The K_{eff} has been conservatively calculated to be 0.9271 which is well below the maximum allowable K_{eff} of 0.95 as described in Section X-3.5 of Volume IV of the SAR. In addition, fuel in the storage pool shall have a U-235 loading of \leq 14.5 grams of U-235 per axial centimeter of fuel assembly.
- C. The spent fuel storage pool is designed and shall be maintained with a storage capacity limited to no more than 2366 fuel assemblies.
- D. The fuel handling bridge fuel hoist has a load-limit cell set at no more than 1230 pounds.

5.6 Seismic Design

The seismic design for Class I structures and equipment is based on dynamic analyses using acceleration response spectrum curves which are based on a ground motion of 0.1.g. The vertical ground acceleration assumed is equal to b of the horizontal ground acceleration. For the design of Class I structures and equipment, the maximum horizontal and vertical accelerations were considered to occur simultaneously. Where applicable, stresses were added directly.

The combined stresses resulting from dead, live, pressure, thermal and earthquake having a ground acceleration of 0.2g are such that a safe shutdown can be achieved.

5.7 Barge Traffic

Barge traffic on the Missouri River past the site has been analyzed to determine that the present size and cargo materials do not create a hazard to the safe operation of the plant. Contact will be maintained with the Corps of Engineers to determine if and when additional analyses are required due to changes in barge size or cargo.