



UNITED STATES  
NUCLEAR REGULATORY COMMISSION  
WASHINGTON, D. C. 20555

PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-277

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 2

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 127  
License No. DPR-44

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 11 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-44 is hereby amended to read as follows:

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PDR ADOCK 05000277  
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

- 3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 18, 1988

PDI-2/D  
MCB  
1/20/88

PDI-2/DM  
REMartin:mr  
01/24/88

OGC ✓  
H Lewis  
2/1/88

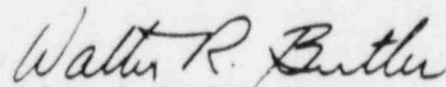
PDI-2/D  
WButler  
2/18/88

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 127, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 18, 1988

ATTACHMENT TO LICENSE AMENDMENT NO.127

FACILITY OPERATING LICENSE NO. DPR-44

DOCKET NO. 50-277

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
128	128
170	170
171	171

Replace the following page of the Appendix B Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

51	51
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LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT				
<p>3.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>	<p>4.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>				
<p>3. From and after the date that any 3 HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding fifteen days unless such pumps are sooner made operable provided all remaining components of the containment cooling system are operable.</p>	<p>3. When it is determined that any 3 HPSW pumps are inoperable, the remaining components of both containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.</p>				
<p>4. From and after the date that 3 containment cooling subsystem loops are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem loop is sooner made operable, provided that all active components of the other containment cooling subsystem loop, including its associated diesel generators, are operable.</p>	<p>4. When 3 containment cooling subsystem loops become inoperable, the operable subsystem loop and its associated diesel-generator shall be demonstrated to be operable immediately and the operable containment cooling subsystem loop daily thereafter.</p>				
<p>5. If the requirements of 3.5.B cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.</p>					
<p>C. <u>HPCI Subsystem</u></p>	<p>C. <u>HPCI Subsystem</u></p>				
<p>1. The HPCI Subsystem shall be operable whenever there is irradiated fuel in the reactor vessel, reactor pressure is greater than 105 psig, and prior to reactor startup from a Cold Condition, except as specified in 3.5.C.2 and 3.5.C.3 below.</p>	<p>1. HPCI Subsystem testing shall be performed as follows:</p> <table border="1" data-bbox="893 1634 1445 1804"> <thead> <tr> <th data-bbox="1004 1634 1075 1661"><u>Item</u></th> <th data-bbox="1235 1634 1384 1661"><u>Frequency</u></th> </tr> </thead> <tbody> <tr> <td data-bbox="893 1689 1153 1804">(a) Simulated Automatic Actuation Test</td> <td data-bbox="1235 1689 1417 1747">Once/operating cycle</td> </tr> </tbody> </table>	<u>Item</u>	<u>Frequency</u>	(a) Simulated Automatic Actuation Test	Once/operating cycle
<u>Item</u>	<u>Frequency</u>				
(a) Simulated Automatic Actuation Test	Once/operating cycle				

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

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| <p>3.7.A <u>Primary Containment (Cont'd.)</u></p> <p>3. <u>Pressure Suppression Chamber - Reactor Building Vacuum Breakers</u></p> <p>a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber-reactor building vacuum breakers shall be operable at all times when primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber-reactor building vacuum breakers shall be <math>0.5 \pm 0.25</math> psid.</p> <p>b. From and after the date that one of the pressure suppression chamber-reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable provided that the repair procedure does not violate primary containment integrity.</p> <p>4. <u>Drywell-Pressure Suppression Chamber Vacuum Breakers</u></p> <p>a. When primary containment is required, all drywell-suppression chamber vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except as specified in 3.7.A.4.b and c below.</p> <p>b. Drywell-suppression chamber vacuum breaker(s) may be "not fully seated" as shown by position indication if testing confirms that the bypass area is less than or equivalent to a one-inch diameter hole. Testing shall be initiated within 8 hours of initial detection of a "not fully seated" position</p> | <p>4.7.A <u>Primary Containment (Cont'd.)</u></p> <p>3. <u>Pressure Suppression Chamber - Reactor Building Vacuum Breakers</u></p> <p>a. The pressure suppression chamber-reactor building vacuum breakers and associated instrumentation including setpoint shall be checked for proper operation every refueling outage.</p> <p>4. <u>Drywell-Pressure Suppression Chamber Vacuum Breakers</u></p> <p>a. Each drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle once a month.</p> <p>b. When it is determined that a vacuum breaker is inoperable for opening at a time when operability is required, all other operable vacuum breakers shall be exercised immediately and every 15 days thereafter until the inoperable vacuum breaker has been returned to normal service.</p> <p>c. Once per operating cycle each vacuum breaker shall be visually inspected</p> |
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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd)4.7.A Primary Containment (Cont'd.)

indication and shall be performed periodically thereafter as follows:

- (1) Once every 15 days (only if "not fully seated" valve(s) are indicated).
- (2) Within 24 hours following vacuum breaker exercising required by 4.7.A.4.a and 4.7.A.4.B. (only if "not fully seated" valve(s) are indicated).

- c. Two drywell-suppression chamber vacuum breakers may be inoperable for opening.
- d. If specifications 3.7.A.4.a, b, or c cannot be met, the situation shall be corrected within 24 hours or the unit shall be placed in a cold shutdown condition in an orderly manner.

5. Oxygen Concentration

- a. The primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.
- b. Within the 24-hour period, subsequent to placing the reactor in the RUN mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

to insure proper maintenance and operation.

- d. A leak test of the drywell to suppression chamber structure shall be conducted at each refueling outage to assure no bypass larger than or equivalent to a one-inch diameter hole exists between the drywell and suppression chamber.

5. Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded at least twice weekly.



7.4 Plant Reporting Requirements7.4.1 Routine Reports

In addition to the environmental monitoring information required by Section 6.9.3.h of Appendix A to the Operating License, the following information shall be submitted in an annual report:

- A. Records of special study programs data and analysis thereof.
- B. Records of changes to the plant which affect the environmental impact of the facility.
- C. Records of changes to environmental permits and certificates.

7.4.2 Non-Routine ReportsA. Environmental Deviation Reports

In the event of an environmental deviation as defined in the environmental technical specifications, notification shall be made within 24 hours by telephone or telegraph to the Director of the NRC-Regional Inspection and Enforcement Office. A written report shall follow within 10 working days to the Director of Nuclear Reactor Regulation (copy to the Director of Regional Inspection and Enforcement Office).

The written report on an environmental deviation, and to the extent possible, the preliminary telephone and telegraph notification, should: (a) describe, analyze, and evaluate implications, (b) determine the cause of the occurrence and (c) indicate the corrective action (including any significant changes made in procedures) taken to preclude repetition of the occurrence and to prevent similar occurrences involving similar components or systems.

B. Reporting of Changes to the Plant or Permits

A written report, including an evaluation of the environmental impact resulting from a change, shall be forwarded to the Director, Office of Nuclear Reactor Regulation (copy to the Director of the Regional Inspection and Enforcement Office) in the event of:

- 1. Changes to the plant that affect the environmental impact evaluation contained in the Environmental Report or the





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NUCLEAR REGULATORY COMMISSION  
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PHILADELPHIA ELECTRIC COMPANY  
PUBLIC SERVICE ELECTRIC AND GAS COMPANY  
DELMARVA POWER AND LIGHT COMPANY  
ATLANTIC CITY ELECTRIC COMPANY

DOCKET NO. 50-278

PEACH BOTTOM ATOMIC POWER STATION, UNIT NO. 3

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 130  
License No. DPR-56

1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Philadelphia Electric Company, et al. (the licensee) dated August 6, 1981 as supplemented on April 2, 1984, December 2, 1985, October 29, 1986 and July 7, 1987, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations set forth in 10 CFR Chapter I.
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health or safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(2) of Facility Operating License No. DPR-56 is hereby amended to read as follows:

(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION

/s/

Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 18, 1988

PDI-2/D  
MCD  
1/20/88

PDI-2/D  
RE Martin:mr  
01/22/88

OGC  
S H Lewis  
2/1/88

PDI-2/D  
WButler  
2/18/88

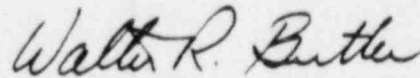
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(2) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 130, are hereby incorporated in the license. PECO shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective within 60 days of its date of issuance to accommodate the writing and approval of implementing procedures.

FOR THE NUCLEAR REGULATORY COMMISSION



Walter R. Butler, Director  
Project Directorate I-2  
Division of Reactor Projects I/II

Attachment:  
Changes to the Technical  
Specifications

Date of Issuance: February 18, 1988

ATTACHMENT TO LICENSE AMENDMENT NO. 130

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
128	128
170	170
171	171

Replace the following page of the Appendix B Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

51	51
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ATTACHMENT TO LICENSE AMENDMENT NO. 130

FACILITY OPERATING LICENSE NO. DPR-56

DOCKET NO. 50-278

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised areas are indicated by marginal lines.

<u>Remove</u>	<u>Insert</u>
128	128
170	170
171	171

Replace the following page of the Appendix B Technical Specifications with the enclosed page. The revised area is indicated by marginal lines.

51	51
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LIMITING CONDITIONS FOR OPERATION	SURVEILLANCE REQUIREMENT				
<p>3.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>	<p>4.5.B <u>Containment Cooling Subsystem (cont'd.)</u></p>				
<p>3. From and after the date that any 3 HPSW pumps are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding fifteen days unless such pumps are sooner made operable provided all remaining components of the containment cooling system are operable.</p> <p>4. From and after the date that 3 containment cooling subsystem loops are made or found to be inoperable for any reason, continued reactor operation is permissible only during the succeeding seven days unless such subsystem loop is sooner made operable, provided that all active components of the other containment cooling subsystem loop, including its associated diesel generators, are operable.</p> <p>5. If the requirements of 3.5.B cannot be met, an orderly shutdown shall be initiated and the reactor shall be in a Cold Shutdown Condition within 24 hours.</p>	<p>3. When it is determined that any 3 HPSW pumps are inoperable, the remaining components of both containment cooling subsystems shall be demonstrated to be operable immediately and weekly thereafter.</p> <p>4. When 3 containment cooling subsystem loops become inoperable, the operable subsystem loop and its associated diesel-generator shall be demonstrated to be operable immediately and the operable containment cooling subsystem loop daily thereafter.</p>				
<p>C. <u>HPCI Subsystem</u></p>	<p>C. <u>HPCI Subsystem</u></p>				
<p>1. The HPCI Subsystem shall be operable whenever there is irradiated fuel in the reactor vessel, reactor pressure is greater than 105 psig, and prior to reactor startup from a Cold Condition, except as specified in 3.5.C.2 and 3.5.C.3 below.</p>	<p>1. HPCI Subsystem testing shall be performed as follows:</p> <table border="1" data-bbox="961 1670 1472 1847"> <thead> <tr> <th data-bbox="1049 1670 1128 1703"><u>Item</u></th> <th data-bbox="1278 1670 1437 1703"><u>Frequency</u></th> </tr> </thead> <tbody> <tr> <td data-bbox="961 1725 1208 1847">(a) Simulated Automatic Actuation Test</td> <td data-bbox="1278 1725 1472 1791">Once/operating cycle</td> </tr> </tbody> </table>	<u>Item</u>	<u>Frequency</u>	(a) Simulated Automatic Actuation Test	Once/operating cycle
<u>Item</u>	<u>Frequency</u>				
(a) Simulated Automatic Actuation Test	Once/operating cycle				

~~April 1973~~

LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS

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| <p>3.7.A <u>Primary Containment (Cont'd.)</u></p> <p>3. <u>Pressure Suppression Chamber - Reactor Building Vacuum Breakers</u></p> <p>a. Except as specified in 3.7.A.3.b below, two pressure suppression chamber-reactor building vacuum breakers shall be operable at all times when primary containment integrity is required. The setpoint of the differential pressure instrumentation which actuates the pressure suppression chamber-reactor building vacuum breakers shall be <math>0.5 \pm 0.25</math> psid.</p> <p>b. From and after the date that one of the pressure suppression chamber-reactor building vacuum breakers is made or found to be inoperable for any reason, reactor operation is permissible only during the succeeding seven days unless such vacuum breaker is sooner made operable provided that the repair procedure does not violate primary containment integrity.</p> <p>4. <u>Drywell-Pressure Suppression Chamber Vacuum Breakers</u></p> <p>a. When primary containment is required, all drywell-suppression chamber vacuum breakers shall be operable and positioned in the fully closed position (except during testing) except as specified in 3.7.A.4.b and c below.</p> <p>b. Drywell-suppression chamber vacuum breaker(s) may be "not fully seated" as shown by position indication if testing confirms that the bypass area is less than or equivalent to a one-inch diameter hole. Testing shall be initiated within 8 hours of initial detection of a "not fully seated" position</p> | <p>4.7.A <u>Primary Containment (Cont'd.)</u></p> <p>3. <u>Pressure Suppression Chamber - Reactor Building Vacuum Breakers</u></p> <p>a. The pressure suppression chamber-reactor building vacuum breakers and associated instrumentation including setpoint shall be checked for proper operation every refueling outage.</p> <p>4. <u>Drywell-Pressure Suppression Chamber Vacuum Breakers</u></p> <p>a. Each drywell-suppression chamber vacuum breaker shall be exercised through an opening-closing cycle once a month.</p> <p>b. When it is determined that a vacuum breaker is inoperable for opening at a time when operability is required, all other operable vacuum breakers shall be exercised immediately and every 15 days thereafter until the inoperable vacuum breaker has been returned to normal service.</p> <p>c. Once per operating cycle each vacuum breaker shall be visually inspected</p> |
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LIMITING CONDITIONS FOR OPERATIONSURVEILLANCE REQUIREMENTS3.7.A Primary Containment (Cont'd)

indication and shall be performed periodically thereafter as follows:

- (1) Once every 15 days (only if "not fully seated" valve(s) are indicated).
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c. Two drywell-suppression chamber vacuum breakers may be inoperable for opening.

d. If specifications 3.7.A.4.a, b, or c cannot be met, the situation shall be corrected within 24 hours or the unit shall be placed in a cold shutdown condition in an orderly manner.

5. Oxygen Concentration

- a. The primary containment atmosphere shall be reduced to less than 4% oxygen with nitrogen gas during reactor power operation with reactor coolant pressure above 100 psig, except as specified in 3.7.A.5.b.
- b. Within the 24-hour period, subsequent to placing the reactor in the RUN mode following a shutdown, the containment atmosphere oxygen concentration shall be reduced to less than 4% by volume and maintained in this condition. De-inerting may commence 24 hours prior to a shutdown.

4.7.A Primary Containment (Cont'd.)

to insure proper maintenance and operation.

- d. A leak test of the drywell to suppression chamber structure shall be conducted at each refueling outage to assure no bypass larger than or equivalent to a one-inch diameter hole exists between the drywell and suppression chamber.

5. Oxygen Concentration

The primary containment oxygen concentration shall be measured and recorded at least twice weekly.

7.4 Plant Reporting Requirements

7.4.1 Routine Reports

In addition to the environmental monitoring information required by Section 6.9.3.h of Appendix A to the Operating License, the following information shall be submitted in an annual report:

- A. Records of special study programs data and analysis thereof.
- B. Records of changes to the plant which affect the environmental impact of the facility.
- C. Records of changes to environmental permits and certificates.

7.4.2 Non-Routine Reports

A. Environmental Deviation Reports

In the event of an environmental deviation as defined in the environmental technical specifications, notification shall be made within 24 hours by telephone or telegraph to the Director of the NRC Regional Inspection and Enforcement Office. A written report shall follow within 10 working days to the Director of Nuclear Reactor Regulation (copy to the Director of Regional Inspection and Enforcement Office).

The written report on an environmental deviation, and to the extent possible, the preliminary telephone and telegraph notification, should: (a) describe, analyze, and evaluate implications, (b) determine the cause of the occurrence and (c) indicate the corrective action (including any significant changes made in procedures) taken to preclude repetition of the occurrence and to prevent similar occurrences involving similar components or systems.

B. Reporting of Changes to the Plant or Permits

A written report, including an evaluation of the environmental impact resulting from a change, shall be forwarded to the Director, Office of Nuclear Reactor Regulation (copy to the Director of the Regional Inspection and Enforcement Office) in the event of:

- 1. Changes to the plant that affect the environmental impact evaluation contained in the Environmental Report or the