

UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

COMMONWEALTH EDISON COMPANY

DOCKET NO. 50-373

LA SALLE COUNTY STATION, UNIT 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No. 36 License No. NPF-11

- The Nuclear Regulatory Commission (the Commission or the NRC) having found that:
 - A. The application for amendment filed by the Commonwealth Edison Company (the licensee), dated January 9, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's regulations set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the regulations of the Commission;
 - C. There is reasonable assurance: (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations set forth in 10 CFR Chapter I;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the enclosure to this license amendment and paragraph 2.C.(2) of the Facility Operating License No. NPF-11 is hereby amended to read as follows:
 - (2) Technical Specifications and Environmental Protection Plan

The Technical Specifications contained in Appendix A, as revised through Amendment No. 36, and the Environmental Protection Plan contained in Appendix B, are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications and the Environmental Protection Plan.



3. This amendment is effective as of date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Elinor G. Adensam, Director

BWR Project Directorate No. 3 Division of BWR Licensing

Enclosure: Changes to the Technical Specifications

Date of Issuance: Mar 13 144

ENCLOSURE TO LICENSE AMENDMENT NO. 36

FACILITY OPERATING LICENSE NO. NPF-11

DOCKET NO. 50-373

Replace the following page of the Appendix "A" Technical Specifications with the enclosed page. The revised page is identified by Amendment number and contains a vertical line indicating the area of change.

REMOVE

INSERT

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3/4 3-18

TABLE 3.3.2-3

ISOLATION SYSTEM INSTRUMENTATION RESPONSE TIME

	RESPONSE TIME (Seconds)#
AUTOMATIC INITIATION	
PRIMARY CONTAINMENT ISOLATION	
a. Reactor Vessel Water Level	
1) Low, Level 3	NA
2) Low Low, Level 2	< 2.0*/c 13(a)**
 Drywell Pressure - High 	$\stackrel{\text{NA}}{\leq} 2.0 \text{(a)} \text{(a)}^{*}$
C. Main Steam Line	
1) Radiation - High ^(b)	< 1.0*/< 13(a)**
2) Pressure - Low	<pre> < 1.0*/< 13(a)** < 1.0*/< 13(a)** < 0.5*/< 13(a)** </pre>
3) Flow - High	< 0.5*/₹ 13 ^(a)
d. Main Steam Line Tunnel Temperature - High	1101
e. Condenser Vacuum - Low f. Main Steam Line Tunnel A Temperature - Hi	NA NA
Temperature Hi	gh NA
SECONDARY CONTAINMENT ISOLATION	
a. Reactor Building (Vent Exhaust Plenum Radiation - High	
Radiation - High	< 13 ^(a)
 Drywell Pressure - High 	< 13(a)
c. Reactor Vessel Water Level - Low, Level 2	< 13(a)
 Reactor Vessel Water Level - Low, Level 6 Fuel Pool Vent Exhaust Radiation - High 	<pre></pre>
REACTOR WATER CLEANUP SYSTEM ISOLATION	
a. Δ Flow - High	< 13 ^{(a)##}
b. Heat Exchanger Area Temperature - High	NA
c. Heat Exchanger Area Ventilation AT-High	NA
d. SLCS Initiation	NΛ
e. Reactor Vessel Water Level - Low Low, Leve	el 2 < 13 ^(a)
REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION	
a. RCIC Steam Line Flow - High	<pre></pre>
b. RCIC Steam Supply Pressure - Low	< 13(a)
c. RCIC Turbine Exhaust Diaphragm Pressure -	High NA
d. RCIC Equipment Room Temperature - High	NA NA
e. RCIC Steam Line Tunnel Temperature - High	NA NA
f. RCIC Steam Line Tunnel Δ Temperature - Hig	ih NA
g. Drywell Pressure - High	NA NA
h. RCIC Equipment Room Δ Temperature - High	NA
RHR SYSTEM STEAM CONDENSING MODE ISOLATION	
a. RHR Equipment Area Δ Temperature - High	NA
b. RHR Area Cooler Temperature - High	NA NA
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