NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Shoreham Nuclear Power Station Unit #1	0 5 0 0 0 3 2 2	1 OF 0 2

TITLE (4)
RBCLCW Split Due to a Low "A" Head Tank Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	1 2	8 6	8 6	— 0 0 7	— 0 0	0 3	1 4	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	POWER LEVEL (10)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)			
5	0 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Robert W. Grunseich, Operational Compliance Engineer	AREA CODE 5 1 6 9 2 9 - 8 3 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On February 12, 1986 at 0908, a split of the Reactor Building Closed Loop Cooling Water (RBCLCW) into its accident configuration occurred due to a low low RBCLCW "A" Head Tank Level. The plant was in Operational Condition 5 with the mode switch in Shutdown for an outage and all rods inserted in the core. An Equipment Operator (EO) was placing the "A" Fuel Pool Cooling Water Heat Exchanger in wet layup, when the "A" RBCLCW head tank level decreased below the setpoints for a system split. The low level alarm did not annunciate and has since been repaired. Station Procedure 23.118.01, RBCLCW System Operation, did not adequately address the requirements for filling the heat exchangers and will be revised to include the necessary steps to prevent recurrence of this event. Plant Management was notified of the event and the NRC was notified at 1230 per 10CFR50.72.

NRC Form 366A
(9-83)

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Shoreham Nuclear Power Station Unit #1	0 5 0 0 0 3 2 2	8 6	— 0 0 7	— 0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On February 12, 1986 at 0908, a split of the Reactor Building Closed Loop Cooling Water (RBCLCW) into its accident configuration occurred due to low low RBCLCW "A" Head Tank Level. The plant was in Operational Condition 5 with the mode switch in Shutdown for an outage and all rods inserted in the core.

An Equipment Operator (EO) was instructed by the Watch Engineer to place the "A" Fuel Pool Cooling Water (FPCW) Heat Exchanger in wet layup (the heat exchanger is normally out of service through the first refueling outage). The FPCW heat exchanger takes suction from the Reactor Building Closed Loop Cooling Water (RBCLCW) Head Tank. Taking special precautions to slowly fill up the heat exchanger, the EO first closed the vent and drain valves on the heat exchanger and checked the outlet valve shut. He then cracked open the inlet valve (approx. 1/4 to 1/2 of a turn) until he felt flow through the line. He then opened the valve to approximately half open over a ten to fifteen minute period. During this time, the level in the RBCLCW "A" Head Tank was decreasing below the level of the setpoints for a low level alarm, but the alarm did not annunciate. The level dropped to the Low Low Level setpoint and at 0908 the RBCLCW system split into its accident configuration ("A" side). The system isolated into two separate, independent loops by automatically closing the header cross-connect valves on the suction and discharge headers of the RBCLCW Heat Exchangers and pumps on the "A" side.

The inlet line to the heat exchanger is a four inch line and the head tank is being fed through a two inch water line. Therefore, water was being taken out of the head tank faster than it was being replaced. Approximately two minutes after the RBCLCW split, the head tank level returned to normal and the RBCLCW system was returned to its normal configuration prior to the event.

There was no safety significance to the event. The RBCLCW system operated as designed. Operators carried out all required actions. No ECC systems were challenged or required for the event. Plant Management was notified of the event and the NRC was notified at 1230 per 10CFR50.72.

There was no method to fill the FPCW heat exchangers identified in Station Procedure SP 23.118.01, RBCLCW System Operation. A procedure change has been initiated to include the venting and filling of these heat exchangers to prevent recurrence. The Plant Manager has directed that a review be performed of all Station Procedures used for the venting, filling and draining of safety related equipment, to ensure that proper venting, filling and draining is achieved. In addition, the head tank low level switch (1042-41ØLS-ØØ5AY) was repaired.