

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Millstone Nuclear Power Station Unit 3	DOCKET NUMBER (2) 0 5 0 0 0 4 2 3	PAGE (3) 1 OF 0 2
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TITLE (4)
Reactor Trip Due to Low Steam Generator Level

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0	2	12	8	6	0	0	1	5	0	0
0	2	12	8	6	0	0	3	1	0	0
0	2	12	8	6	0	0	3	1	0	0

OPERATING MODE (8) 1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following) (11)									
POWER LEVEL (10) 0 1 2 1 9	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.408(c)	<input checked="" type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 50.36(a)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 50.36(a)(2)	<input type="checkbox"/> 50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)		TELEPHONE NUMBER	
NAME	AREA CODE	NUMBER	
David McDaniel, Reactor Engineer	21013	4141 7 -1171911	

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	
D	S	W		N						

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE:)			<input checked="" type="checkbox"/> NO			

ABSTRACT (Limit to 1400 spaces, i.e. approximately fifteen single-space typewritten lines) (16)

At 1044 on 2/12/86, while operating at 29% Reactor Power, the plant received a reactor trip from low steam generator level. Operators were in the process of shifting from a turbine driven main feed pump to a motor driven main feed pump. Plant Operators verified that all rods were fully inserted after the trip and that the reactor trip breakers opened. All safety systems actuated properly.

Subsequent investigation revealed an error in the procedure covering operation of main feedwater pumps which caused a main feed pump trip and a subsequent reactor trip on a low steam generator level. The affected procedure has been changed to correct the error and additional operator training will take place.

This report is being submitted in accordance with 10 CFR 50.73(a)(2)(iv).

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Millstone Nuclear Power Station Unit 3	DOCKET NUMBER (2) 05000423	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		86	015	00	02	OF 02

TEXT (if more space is required, use additional NRC Form 366A's) (17)

At 1044 on 2/12/86, while operating at 29% of Rated Thermal Power, the plant received a reactor trip due to Low Steam Generator Level. The operators were in the process of shifting from a turbine driven feed pump to a motor driven feed pump, in anticipation of taking the main turbine off line, due to high vibration. Prior to starting the motor driven feed pump, the feedwater/condensate system had one turbine driven feed pump and one condensate pump in operation, supplying the steam generators with approximately 6500 gpm. When the motor driven feed pump was started, its recirc valve opened to provide the pump with a minimum flow of 5000 gpm. The operator then began to decrease the turbine driven feed pump's speed. As the feed pump was decreased, the recirc valve on the turbine driven feed pump opened to provide it with minimum flow. At this point total flow through both feed pumps was estimated to be approximately 12,000 gpm. As the capacity of one condensate pump is only about 10,500 gpm, feed flow now exceeded condensate flow. As a result the motor driven feed pump tripped by design on low suction pressure.

When the motor driven pump tripped, the operator tried to recover by increasing speed on the turbine driven feed pump and start supplying steam generators from it.

However, the operator was not able to recover speed on the turbine driven pump fast enough. As a result, steam generator levels decreased due to lack of feed flow, and a reactor trip occurred, due to low steam generator level.

The root cause of this event was procedural inadequacy. The operating procedure for the feed water system did not require starting an additional condensate pump prior to starting a motor driven feed pump. The procedure did require the starting of an additional condensate pump prior to starting an additional Steam Turbine Driven Feed Pump. Operating procedures have been modified to require starting an additional condensate pump prior to starting any additional main feed pumps. Additional training for operators will also occur to emphasize the importance of providing the feed pumps adequate suction flow.

There were no safety implications to the public, as all equipment performed its safety function.

This report is being submitted as required by 10 CFR 50.73(a)(2)(IV).

NORTHEAST UTILITIES



THE CONNECTICUT LIGHT AND POWER COMPANY
EASTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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MP-8805

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-015-00

Gentlemen:

This letter forwards Licensee Event Report 86-015-00 to be submitted within thirty days pursuant to 10CFR50.73 (a) (2) (iv), any event or condition that resulted in manual or automatic actuation of any Engineered Safety Feature (ESF).

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/DTM:pdm

Attachment: LER 86-015-00

cc: Dr. T. E. Murley, Region I

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