NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/88

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	1 OF 0 2

TITLE (4)
Area Temperature Monitoring - CS-01

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 2	0 8	8 6	8 6	0 1 6	0 0	0 3	1 0	8 6		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	3	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10)	0 0 0	20.402(b)	20.405(c)	50.73(a)(2)(iv)	73.71(b)
		20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	X OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	SPECIAL REPORT
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Thomas Cleary, Associate Engineer	AREA CODE 2 0 3 4 4 4 - 5 5 7 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	I	M	S 4 2 0	N					

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

This Special Report is being submitted within thirty days pursuant to Plant Technical Specifications 3.7.14b and 6.9.2. Plant Technical Specification 3.7.14b requires that a Special Report be submitted to the Nuclear Regulatory Commission if one or more areas exceeds the specified temperature limit by less than 20 degrees Fahrenheit for more than 8 hours. Containment Area CS-01 (inside crane wall) has exceeded the 120 degrees Fahrenheit specified limit. Subsequent investigation reveals that the pressurizer cubicle is the only area exceeding the limits.

On 2/7/86 the pressurizer cubicle reached a temperature of 121.2 degrees Fahrenheit and the plant entered the Action Statement at 0224 hours on 2/8/86. All environmentally qualified equipment was verified to be operable.

This Special Report is being submitted pursuant to Plant Technical Specifications 3.7.14b and 6.9.2.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Millstone Nuclear Power Station Unit 3	0 5 0 0 0 4 2 3	8 6	0 1 6	0 0	0 2 OF 0 2

TEXT (If more space is required, use additional NRC Form 366A's) (17)

This Special Report is being submitted within thirty days pursuant to Plant Technical Specifications 3.7.14b and 6.9.2. Plant Technical Specification 3.7.14b requires that a Special Report be submitted to the NRC if one or more areas exceeds the specified temperature limit by less than 20 degrees Fahrenheit for more than 8 hours. Containment Area CS-01 (inside crane wall) has exceeded the 120 degrees Fahrenheit specified limit. Subsequent investigation reveals that the pressurizer cubicle is the only area exceeding the limits.

On 2/7/86 the pressurizer cubicle reached a temperature of 121.2 degrees Fahrenheit and the plant entered the Action Statement at 0224 hours on 2/8/86. Further investigation reveals that the pressurizer cubicle temperature has been varying between approximately 119 degrees Fahrenheit to 125 degrees Fahrenheit consistently since 2/7/86. Therefore, an analysis has been performed for continued operability for a sustained temperature of 125 degrees Fahrenheit within the pressurizer cubicle. The following is a list of affected equipment.

3RCS*MV8000A - Pressurizer Relief Isolation Valve
3RCS*MV8000B - Pressurizer Relief Isolation Valve
3RCS*PCV455A - Pressurizer Pressure Control Valve
3RCS*PCV456 - Pressurizer Power Relief Valve
3RCS*SV8095A - Reactor Vessel Head Vent Isolation Valve
3RCS*SV8095B - Reactor Vessel Head Vent Isolation Valve
3RCS*SV8096A - Reactor Vessel Head Vent Isolation Valve
3RCS*SV8096B - Reactor Vessel Head Vent Isolation Valve

Calculations have been performed for all affected equipment. Review of these calculations demonstrate continued operability of this equipment. The thermal life characteristics for each component has been adjusted accordingly.

This Special Report is being submitted pursuant to Plant Technical Specifications 3.7.14b and 6.9.2.

NORTHEAST UTILITIES

THE CONNECTICUT LIGHT AND POWER COMPANY
WESTERN MASSACHUSETTS ELECTRIC COMPANY
HOLYOKE WATER POWER COMPANY
NORTHEAST UTILITIES SERVICE COMPANY
NORTHEAST NUCLEAR ENERGY COMPANY

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March 10, 1986
MP-8804

U. S. Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Reference: Facility Operating License No. NPF-49
Docket No. 50-423
Licensee Event Report 50-423/86-016-00

Gentlemen:

This letter forwards Licensee Event Report 86-016-00, a Special Report required to be submitted within thirty days pursuant to Plant Technical Specifications 3.7.14b and 6.9.2. The plant entered the Action Statement for area temperature monitoring on February 8, 1986.

Yours truly,

NORTHEAST NUCLEAR ENERGY COMPANY

Wayne D. Romberg
Station Superintendent
Millstone Nuclear Power Station

WDR/TC:se

Attachment: LER 86-016-00

cc: Dr. T. E. Murley, Region I

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