NRC Form 366 (9-83)					LIC	LICENSEE EVENT REPORT (LER)					U.S. NUCLEAR REGULATORY COMMISSION APPROVED OMB NO. 3150-0104 EXPIRES: 8/31/86					
FACILITY NAME (1)										10	OCKET NUMBER	1 (2)	PAGE (3)			
Wolf Creek Generating					ating St	Station				0 5 0 0	10141812	1 OF 0 13				
TITLE (4																
			hnica						sed Ho	ourly Fire						
EVE	ENT DATE	(5)	LER NUMBER (6) REPORT DATE (7)					OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL	REVISION NUMBER	MONTH	DAY	YEAR		FACILITY NAMES		DOCKET NUMBER(S)				
												0 5 0 0	10111			
0 2	1 1	8 6	8 6	- 000	1-00	0 3	111	8 6				0 5 0 0	101 1 1			
	RATING		THIS REP	ORT IS SUBMITTE	ED PURSUANT	TO THE R	EQUIREM	ENTS OF 10	CFR 8: /	Check one or more o	of the following) (11)				
MODE (6) 1 POWER LEVEL (10) 1 0 0		20,402(b) 20,405(a)(1)(i) 20,405(a)(1)(ii) 20,405(a)(1)(iii) 20,405(a)(1)(iv) 20,405(a)(1)(iv)		X	20.405(e) 50.36(e)(1) 50.36(e)(2) (50.73(a)(2)(i) 50.73(a)(2)(iii) 50.73(a)(2)(iiii)		50.73(a)(2)(iv) 50.73(a)(2)(vii) 50.73(a)(2)(viii)(A) 50.73(a)(2)(viii)(A) 80.73(a)(2)(viii)(B) 60.73(a)(2)(xiii)(B)			73.71(b) 73.71(c) OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
						ICENSEE	CONTACT	FOR THIS	LER (12)							
NAME	Merli	in G.	Will	iams - Su ar	perintend Admin					Quality	AREA CODE	TELEPHONE NUM				
			P108(4)	COMPLETE	ONE LINE FOR	EACH CO	OMPONEN	T FAILURE	DESCRIBE	D IN THIS REPOR		And the balance of the same				
CAUSE	SYSTEM	COMPO	DNENT	MANUFAC- TURER	REPORTABLE TO NPRDS			CAUSE	SYSTEM	COMPONENT	MANUFAC- TURER	REPORTABLE TO NPROS	-			
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On February 13, 1986, at approximately 1410 CST, a violation of Technical Specification 3.7.11 was discovered due to a failure to initiate an hourly fire watch patrol of three areas containing inoperable fire barriers. Technical Specification 3.7.11 requires, in part, the establishment of an hourly fire watch patrol for areas containing inoperable fire barriers.

SUPPLEMENTAL REPORT EXPECTED (14)

On February 11, 1986, craft personnel had removed fireproofing material from structural steel in the Auxiliary Building, but through a cognitive personnel error, failed to recognize that sufficient quantities of fireproofing material had been removed to necessitate the initiation of a Fire Protection Impairment Control Permit (which is utilized to establish fire watch patrols).

Upon discovery, an hourly fire watch patrol was established in accordance with Technical Specification 3.7.11. During this time period, the unit operated in Mode 1, Power Operation, at power levels up to 100 percent reactor power. To prevent recurrence, a knowledgeable individual has been assigned the responsibility of overseeing the removal and reinstallation of fireproofing material.

There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop that might have posed a threat to the public health or safety.

A previous similar occurrence is discussed in Licensee Event Report

85-077-00.

YES (If yes, complete EXPECTED SUBMISSION DATE)

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

8603170379 860311 PDR ADDCK 05000482 S PDR LE 22

MONTH

EXPECTED SUBMISSION DATE (16) DAY

YEAR

		361	
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NU. 3150-0104 EXPIRES 8/31/85

FACILITY NAME (1)	DOCKET NUMBER (2)		L	ER NUMBER (6)		PAGE (3)		
		YEAR	F	SEQUENTIAL NUMBER	REVISI NUMB	ER.		
Wolf Creek Generating Station	0 5 0 0 0 4 8 2	816	-	0 10 1 4	-01	0 0 12	OF	0 3

On February 13, 1986, at approximately 1410 CST, a violation of Technical Specification 3.7.11 was discovered. This situation was the result of the removal of portions of fireproofing material from structural steel in the Auxiliary Building [NF] without establishing the required hourly fire watch patrols. Technical Specification 3.7.11 requires, in part, the establishment of an hourly fire watch patrol of areas containing an inoperable fire barrier.

The potential for the existence of this situation was identified by an NRC Resident Inspector, who questioned the removal of fireproofing material without issuance of a Fire Protection Impairment Control Permit. It was subsequently determined that craft personnel had removed sufficient quantities of fireproofing material from structural steel at three locations in the Auxiliary Building to render the areas as inoperable fire barriers.

Upon discovery, an hourly fire watch patrol was established for the affected areas in accordance with the requirements of Technical Specification 3.7.11. The fireproofing material was re-installed in the three affected areas on February 14, 1986.

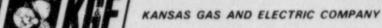
Investigations revealed that the craft personnel had removed the fireproofing material from these locations on February 11, 1986, in support of other work activities. Therefore, this Technical Specification violation existed from that time until the hourly patrol was established on February 13. During this time period, the unit operated in Mode 1, Power Operation, at power levels up to 100 percent reactor power.

A previous engineering evaluation had identified the maximum allowable amount of fireproofing material that could be removed without affecting the performance of the fireproofing material. The craft personnel were aware of the criteria and of the necessity to initiate a Fire Protection Impairment Control Permit (utilized to establish fire watch patrols) if the criteria were exceeded. However, through a cognitive personnel error, the craft personnel failed to recognize that the fireproofing material removal critera had been exceeded in these three areas.

A contributing factor to this occurrence was the fact that this crew of craft personnel was relatively new and may not have fully understood the requirements. Cross training of craft personnel on fire protection program requirements was in progress, and these personnel had not yet received this training. These personnel have now received training on the fireproofing material removal criteria. A field inspection of the areas in which they had been working revealed no other instances of noncompliance.

To prevent future occurrences of this nature, a knowledgeable individual has been assigned the responsibility of overseeing the removal and replacement of fireproofing material in all areas by craft personnel.

U.S. NUCLEAR REGULATORY COMMISSION LICENSEE EVENT REPORT (LER) TEXT CONTINUATION APT OVED OMB NO. 3150-0104 EXPIRES 8/31/85 FACILITY NAME (1) DOCKET NUMBER (2) LER NUMBER (6) PAGE (3) SEQUENTIAL REVISION NUMBER 0 |5 |0 |0 |0 |4 |8 | 2 |8 |6 |- 0 | Wolf Creek Generating Station There was no damage to plant equipment or release of radioactivity as a result of this event, and at no time did conditions develop that may have posed a threat to the health or safety of the public. Licensee Event Report 85-077-00 discusses one previous similar occurrence of a Technical Specification violation resulting from a failure to initiate a required fire watch when rendering a fire barrier penetration inoperable.





GLENN L KOESTER

March 11, 1986

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555

Mr. E.H. Johnson, Director Division of Reactor Safety and Projects U.S. Nuclear Regulatory Commission Region IV 611 Ryan Plaza Drive, Suite 1000 Arlington, Texas 76011

KMLNRC 86-042

Re: Docket No. STN 50-482

Subj: License Event Report 86-004-00

Gentlemen:

The attached Licensee Event Report is submitted pursuant to 10 CFR 50.73 (a) (2) (i) concerning a Technical Specification violation.

John a Sailey for

Glenn L. Koester

Vice President - Nuclear

GLK: see

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

IEZL