



Northern States Power Company

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August 24, 1992

Report Required by 10 CFR Part 50, Section 50.73

U.S. Nuclea. Regula ory Commission Attn: Document Control Desk Washington, DC 20555

> MONTICELLO NUCLEAR GENERATING PLANT Docket No. 50-263 License No. DPR-22

Missed Surva llance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program

The Licensee Event Report for thi occurrence is attached. Please contact us if you require further information.

Thomas M Parker

Manager

Nuclear Support Services

c: Regional Administrator - III NRC Sr Resident Inspector, NRC NRR Fcoject Manager, NRC State of Minnesots, Attn: Kris Sanda

Attachment

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APPROVED OME NO. 3150-0104 EXPIRES. 4/30/97

ESTIMATED BURDEN PER RESIDNSE TO COMPLY WITH THIS INFORMATION COLLECTION FROUEST 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH 19-5301. U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104) OFFICE OF MANAGEMENT AND BUDGET WASHINGTON DC 70503.

# LICENSEE EVENT REPORT (LER)

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Safety Systems Engineering  COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBE  COMPONENT MANUFAC STEPORTABLE TO APPROS CAUSE SYSTEM	Missed Surveillance Requirement Caused by Failure to Inservice Water Valves in Section XI Testing Program  (8)	Missed Surveillance Requirement Caused by Failure to Inclu Service Water Valves in Section XI Testing Program  (8)	Missed Surveillance Requirement Caused by Failure to Include Service Water Valves in Section XI Testing Program  (S) LER HUMBER (8) REPORT DATE (7) OTHER FACILITIES (8) PAGE OF THE FACILITIES (9) OTHER (9) OTHER FACILITIES (9) OTHER FACILIT	Missed Surveillance Requirement Caused by Failure to Include Emer Service Water Valves in Section XI Testing Program  (5)	Missed Surveillance Requirement Caused by Failure to Include Emerge Service Water Valves in Section XI Testing Program  (8) LER HUMBER (8) REPORT DATE (7) OTHER FACILITIES (NVOLVED YEAR YEAR) SELVENT, NUMBER MONTH DAY YEAR FACILITY NAMES DOUGH NUMBER NUMBER MONTH DAY YEAR FACILITY NAMES DOUGH NUMBER NUMBER MONTH DAY YEAR FACILITY NAMES DOUGH NUMBER NUMBER NOT THE REQUIREMENTS OF 10 CFR \$ (Check use or more or me fellowing) (11) 20 405(4) 1110 \$0.36(4) 50.36(	Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program  (8)	Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program    Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program   Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program   Surveillance Report Expected In Section Program   Surveillance Report Expected In Testing P	Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program  Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program  Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program  Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program  Missed Surveillance Requirement Caused by Failure to Include Emergency  Service Water Valves in Section XI Testing Program  Missed Surveillance Requirement Caused by Failure to Include Emergency  Missed Surveillance Requirement Caused by Failure to Include Emergency  Missed Surveillance Requirement Caused by Failure to Include Emergency  Missed Surveillance Requirement Caused by Failure to Include Emergency  Missed Surveillance Requirement Caused by Failure to Include Emergency  Missed Surveillance Requirement Section XI Testing Program  Missed Surveillance Requirement Failure Description Testing Program  Missed Surveillance Requirement Failure Description This Report Include Include Emergency  Missed Surveillance Requirement Pailure Description This Report Include In	Monticello Nuclear Generating Plant    O   S   O   O   2   6   3   1   OF

On July 25, 1992, during a Design Bases Document review of the Emergency Service Water system it was identified that four valves used to mitigate the consequences of a High Energy Line Break were not included in the ASME Section XI Testing Program as required by Technical Specifications. The cause of this event was failure to consider ASME Section XI pump and valve testing requirements during preparation, review, and approval of abnormal operating procedures. The applicable abnormal operating procedure was revised to provide direction if the valves failed to operate, a test was performed to confirm adequate cooling would be provided when the Emergency Service Water system is cross connected, and a review of the High Energy Line Break in the feedwater pump area is being conducted to identify other components to be included in the ASME Section XI program. Administrative controls will be revised to require a review of abnormal operating procedures for ASME Section XI components.

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#### U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 4/30/92

# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

ESTIMATED EURDEN PER RESPONSE TO COMPLY WTH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURGEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-30). U.S. NOCLEAR REGULATORY COMMISSION. WASHINGTON, OC 2056S. AND TO THE FARRENOR REDUCTION PROJECT (3150-0104). DEFICE OF MANAGEMENT AND SUDGET, WASHINGTON, OC 20503.

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## DESCRIPTION

On July 25, 1992, at 100% power, during the performance of a Design Bases Document review of the Emergency Service Water (EIIS System: BI) system it was determined that four valves which would be used to mitigate a postulated High Energy Line Break Accident were not tested as required by Technical Specifications.

Technical Specification 4.15.B states, in part, that "Inservice Testing of Quality Group A, B, and C pumps and valves shall be performed ...". The four valves not tested are ESW-1-1 (11 ESW Pump Discharge Check; included in Section XI testing program but not for stoppage of reverse flow), ESW-2-2 and ESW-2-1 (ESW System Cross-ties), and ESW-5-1 (12 ESW Pump Strainer Outlet). These valves are used to mitigate a postulated High Energy Line Break in the Feedwater Pump (EIIS System: SJ) area. During this postulated event a loss of off-site power, a failure of 12 Diesel Generator, and damage to the 11 Emergency Service Water Pump power supply must be assumed. The abnormal procedure directs the operators to cross-tie 12 Emergency Service Water Pump discharge to supply cooling water to the 11 Emergency Diesel Generator (See Figure 1, attached). ESW-1-1, ESW-2-1, ESW-2-2, and ESW-5-1 would be required to function to insure adequate cooling to 11 Emergency Diesel Generator. None of the four valves are tested for these specific functions in the ASME Section XI Testing Program.

During the investigation of this event, other High Energy Line Break mitigation equipment was identified which will require additional components to be included in the ASME Section XI testing program. This includes components of the normal Service Water system (EIIS System: KG).

This event is reportable as a condition prohibited by Technical Specification because of the failure to perform a required surveillance.

### CAUSE

The cause of this event was failure to consider ASME Section XI test requirements during preparation, review, and approval of abnormal procedures.

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#### U.S. NUCLEAR REGULATORY COMMISSION

APPROVED OMB NO. 3150-0104 EXPIRES: 4/30/92

# TEXT CONTINUATION

ESTIMATED SURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: SO,0 MRS. FORK ARC COMMENTS REGARDING BURDEN ESTIMATE TO THE RECURDS AND REPORTS WANGGEMENT BRANCH [P-530]. U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20595, AND TO THE PAPERWORK REDUCTION PROJECT [1315-0]-1041 OFFICE OF MANAGEMENT AND SUDGET, WASHINGTON, DC 20503.

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## ANALYSIS

A test was performed on August 23, 1992, which demonstrated the ability of the components to perform the required functions. Based on the test results there were no consequences to the health and safety of the public.

### CORRECTIVE ACTIONS

The following actions have been completed:

- The procedure for loss of AC power with a High Energy Pipe Break near the Reactor Feedwater Pumps was revised on July 31,1992, to include installation of blank flanges and/or removal of valve internals if needed to cope with valve failures.
- A test was performed to confirm adequate cooling could be provided to the 11 Emergency Diesel Generator using 12 Emergency Service Water Pump and the normal Service Water system.

The following actions will be completed:

- Administrative controls associated with abnormal procedure writing will be revised to require a review for ASME Section XI requirements.
- A review of all abnormal procedures will be conducted to identify any additional ASME Section XI components that should be included in the Inservice Testing program. Any components so identified will added to the program.

### ADDITIONAL INFORMATION

Failed Component Identification

None

Previous Similar Events

None

MRC FORM, 386A

U.S. MUCLEAR REGULATORY COMMISSION

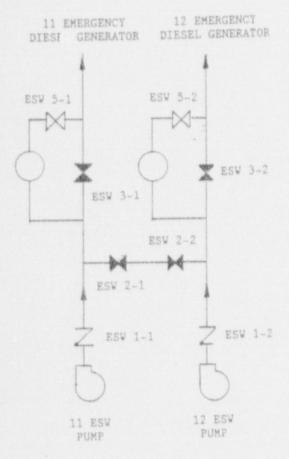
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# LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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TEXT (If more space is required, use additional NRC Form 3664's) (17)



NORMAL LINEUP

DIESEL GENERATOR ESW 5-2 ESW 5-1 ESW 3-2 ESW 3-1 ESW 2-2 ESW 2-1 ESW 1-2 ESW 1-1 12 ESW

11 EMERGENCY

HELB SCENERIO

PUMP

11 ESW

PUMP

Figure 1