



Northern States Power Company

414 Nicollet Mall
Minneapolis, Minnesota 55401-1927
Telephone (612) 330-5500

August 24, 1992

Report Required by
10 CFR Part 50, Section 50.73

U.S. Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

MONTICELLO NUCLEAR GENERATING PLANT
Docket No. 50-263 License No. DPR-22

Missed Surveillance Requirement Caused by Failure to Include
Emergency Service Water Valves in Section XI Testing Program

The Licensee Event Report for this occurrence is attached. Please contact us
if you require further information.

TE Com for

Thomas M Parker
Manager
Nuclear Support Services

c: Regional Administrator - III NRC
Sr Resident Inspector, NRC
NRR Project Manager, NRC
State of Minnesota,
Attn: Kris Sanda

Attachment

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PDR ADOCK 05000263
S PDR

IF22
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LICENSEE EVENT REPORT (LER)

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (P-530), U.S. NUCLEAR REGULATORY COMMISSION WASHINGTON, DC 20555, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET WASHINGTON, DC 20503.

FACILITY NAME (1) DOCKET NUMBER (2) PAGE (3)
 Monticello Nuclear Generating Plant 0 5 0 0 0 2 6 3 1 OF 0 4

TITLE (4)
 Missed Surveillance Requirement Caused by Failure to Include Emergency Service Water Valves in Section XI Testing Program

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)																										
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)																								
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OPERATING MODE (9) N

POWER LEVEL (10) 1 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)

<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.408(e)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
<input type="checkbox"/> 20.408(a)(1)(ii)	<input type="checkbox"/> 80.73(a)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(a)
<input type="checkbox"/> 20.408(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)	<input type="checkbox"/> 50.73(a)(2)(iv)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
<input type="checkbox"/> 20.408(a)(1)(iv)	<input checked="" type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)	
<input type="checkbox"/> 20.408(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 80.73(a)(2)(vii)(B)	
<input type="checkbox"/> 20.408(a)(1)(vi)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME TELEPHONE NUMBER
 Patrick Tobin, Supt. Safety Systems Engineering 6 1 2 2 9 5 - 1 2 9 6

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)

MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces - i.e., approximately fifteen single-space typewritten lines) (16)

On July 25, 1992, during a Design Bases Document review of the Emergency Service Water system it was identified that four valves used to mitigate the consequences of a High Energy Line Break were not included in the ASME Section XI Testing Program as required by Technical Specifications. The cause of this event was failure to consider ASME Section XI pump and valve testing requirements during preparation, review, and approval of abnormal operating procedures. The applicable abnormal operating procedure was revised to provide direction if the valves failed to operate, a test was performed to confirm adequate cooling would be provided when the Emergency Service Water system is cross connected, and a review of the High Energy Line Break in the feedwater pump area is being conducted to identify other components to be included in the ASME Section XI program. Administrative controls will be revised to require a review of abnormal operating procedures for ASME Section XI components.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 30.0 HRS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F-530), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 2	0 1 0	0 0	0 2	OF 0 4

TEXT (if more space is required, use additional NRC Form 356A's) (17)

DESCRIPTION

On July 25, 1992, at 100% power, during the performance of a Design Bases Document review of the Emergency Service Water (EIS System: BI) system it was determined that four valves which would be used to mitigate a postulated High Energy Line Break Accident were not tested as required by Technical Specifications.

Technical Specification 4.15.B states, in part, that "Inservice Testing of Quality Group A, B, and C pumps and valves shall be performed ...". The four valves not tested are ESW-1-1 (11 ESW Pump Discharge Check; included in Section XI testing program but not for stoppage of reverse flow), ESW-2-2 and ESW-2-1 (ESW System Cross-ties), and ESW-5-1 (12 ESW Pump Strainer Outlet). These valves are used to mitigate a postulated High Energy Line Break in the Feedwater Pump (EIS System: SJ) area. During this postulated event a loss of off-site power, a failure of 12 Diesel Generator, and damage to the 11 Emergency Service Water Pump power supply must be assumed. The abnormal procedure directs the operators to cross-tie 12 Emergency Service Water Pump discharge to supply cooling water to the 11 Emergency Diesel Generator (See Figure 1, attached). ESW-1-1, ESW-2-1, ESW-2-2, and ESW-5-1 would be required to function to insure adequate cooling to 11 Emergency Diesel Generator. None of the four valves are tested for these specific functions in the ASME Section XI Testing Program.

During the investigation of this event, other High Energy Line Break mitigation equipment was identified which will require additional components to be included in the ASME Section XI testing program. This includes components of the normal Service Water system (EIS System: KG).

This event is reportable as a condition prohibited by Technical Specification because of the failure to perform a required surveillance.

CAUSE

The cause of this event was failure to consider ASME Section XI test requirements during preparation, review, and approval of abnormal procedures.

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HRS. FOR ARC COMMENTS REGARDING BURDEN ESTIMATE TO THE RECORDS AND REPORTS MANAGEMENT BRANCH (F&SD), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20585, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 0 5 0 0 0 2 6 3	LER NUMBER (3)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		9 2	0 1 0	0 0	3 3	OF 0 4

TEXT IF MORE SPACE IS REQUIRED, USE ADDITIONAL NRC FORM 366A's (17)

ANALYSIS

A test was performed on August 23, 1992, which demonstrated the ability of the components to perform the required functions. Based on the test results there were no consequences to the health and safety of the public.

CORRECTIVE ACTIONS

The following actions have been completed:

1. The procedure for loss of AC power with a High Energy Pipe Break near the Reactor Feedwater Pumps was revised on July 31, 1992, to include installation of blank flanges and/or removal of valve internals if needed to cope with valve failures.
2. A test was performed to confirm adequate cooling could be provided to the 11 Emergency Diesel Generator using 12 Emergency Service Water Pump and the normal Service Water system.

The following actions will be completed:

1. Administrative controls associated with abnormal procedure writing will be revised to require a review for ASME Section XI requirements.
2. A review of all abnormal procedures will be conducted to identify any additional ASME Section XI components that should be included in the Inservice Testing program. Any components so identified will added to the program.

ADDITIONAL INFORMATION

Failed Component Identification

None

Previous Similar Events

None

LICENSEE EVENT REPORT (LER)
TEXT CONTINUATION

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FACILITY NAME (1) Monticello Nuclear Generating Plant	DOCKET NUMBER (2) 050002639	LER NUMBER (5)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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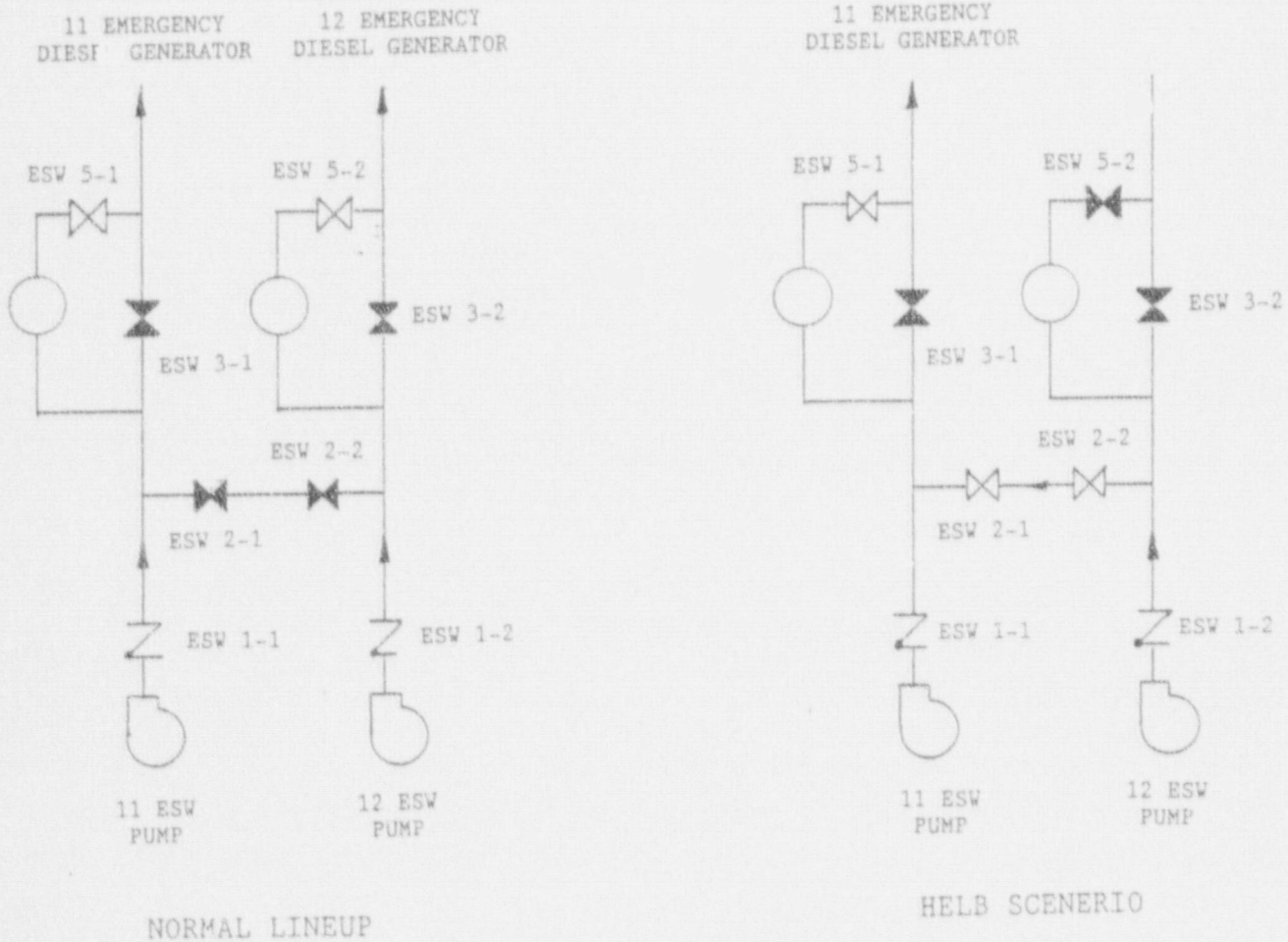


Figure 1