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Gary J. Taylor
Vice President
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May 20, 1997
RC-97-0107

Document Control Desk
U. S. Nuclear Regulatory Commission
Washington, DC 20555

Gentlemen:

Subject: VIRGIL C. SUMMER NUCLEAR STATION (VCSNS)
DOCKET NO. 50/395
OPERATING LICENSE NO. NPF-12
Licensee Event Report, (LER 970001)

Attached is Licensee Event Report No. 970001 for the Virgil C. Summer Nuclear Station . This report is submitted pursuant to the requirements of 10CFR50.73(a)(2)(iv).

Should you have any questions, please call Mr. Jeffrey Pease at (803) 345-4124.

Very truly yours,

Gary J. Taylor

JWP/GJT/nkk

Attachment
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RTS (LER 970001, CER 970353)
File (818.05, 818.07)
DMS (RC-97-0107)

IE22

NRC FORM 366 (4-95) U.S. NUCLEAR REGULATORY COMMISSION

APPROVED BY OMB NO. 3150-0104
EXPIRES 04/30/98
ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33), U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Virgil C.Summer Nuclear Station	05000395	1 OF 3

TITLE (4)
Manual Reactor Trip

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
										05000
0 4	2 2	9 7	9 7	-- 001	-- 0 0	0 5	2 0	9 7	FACILITY NAME	DOCKET NUMBER
										05000

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)	100	20.2201(b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)	
		20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)	
		20.2203(a)(2)(i)		20.2203(a)(3)(ii)0		50.73(a)(2)(iii)		73.71	
		20.2203(a)(2)(ii)		20.2203(a)(4)	X	50.73(a)(2)(iv)		OTHER	
		20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below	
		20.2203(a)(2)(iv)		50.36(c)(2)		50.73(a)(2)(vii)		or in NRC FORM 366A	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER (Include Area Code)
April R.Rice, Manager, Nuclear Licensing & Operating Experience	(803) 345-4232

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
EB	TG	PIPEXXA		N					

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE).	X	NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

On April 22, 1997, plant personnel conservatively initiated a manual reactor trip in response to a leak on the Main Turbine Electro-Hydraulic Control (EHC) system. It was subsequently determined that an "O" ring on the shutdown servo to Combined Intercept Valve (CIV) #1 had failed. Although Feedwater Isolation was achieved, one Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design. Repairs were accomplished and the plant restarted.

FACILITY NAME (1)	DOCKET NUMBER	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
V. C. Summer Nuclear Station	05000395	9 7	--001--	0 0	2 OF 3

TEXT *(If more space is required, use additional copies of NRC Form 366A)* (17)

PLANT IDENTIFICATION

Westinghouse - Pressurized Water Reactor

EQUIPMENT IDENTIFICATION

Hydraulic "O" ring in Electro-Hydraulic Control System
EIIS System Code - TG

IDENTIFICATION OF EVENT

Manual Reactor Trip

EVENT DATE

April 22, 1997

REPORT DATE

May 20, 1997
This report was initiated by CER 970353

CONDITIONS PRIOR TO EVENT

MODE 1 - 100% Reactor Power

DESCRIPTION OF EVENT

On April 22, 1997, plant personnel conservatively initiated a manual reactor trip in response to a leak on the Main Turbine Electro-Hydraulic Control (EHC) system. It was subsequently determined that an "O" ring on the shutdown servo to Combined Intercept Valve (CIV) #1 had failed. Although Feedwater Isolation was achieved, one Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design. Repairs were accomplished and the plant restarted.

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TEXT *(If more space is required, use additional copies of NRC Form 366A)* **(17)**

CAUSE OF EVENT

The cause of the manual reactor trip is attributed to conservative operator action in anticipation of a Turbine trip due to the mechanical failure of a Hydraulic "O" ring in the Main Turbine Electro-Hydraulic Control (EHC) system.

ANALYSIS OF EVENT

One Feedwater Regulating Valve (FRV) was slow to completely close in response to a Feedwater Isolation Signal. All other systems functioned per design.

IMMEDIATE CORRECTIVE ACTIONS:

The hydraulic "O" ring in the EHC system was replaced. Mechanical grooming of the FRV was accomplished. The outage package was reviewed for additional items which could be worked to enhance plant performance. These items were identified and worked prior to plant restart.

ADDITIONAL CORRECTIVE ACTIONS:

SCE&G is evaluating the failure of the hydraulic "O" ring to determine whether additional corrective actions will be required for this non-safety related component. This determination will be completed prior to the end of refueling outage 10, scheduled to begin in October 1997.

PRIOR OCCURRENCES:

None