

YANKEE ATOMIC ELECTRIC COMPANY



Rowe, Massachusetts 01367

April 10, 1986

U.S. Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, Pennsylvania 19406

Attention: Dr. Thomas E. Murley, Regional Administrator

Subject: Licensee Event Report 50-29/86-002

Insufficient Implementation Procedures for the Offsite Dose
Calculation Manual

Dear Sir:

In accordance with 10 CFR 50.73(a)(2)(i), the attached Licensee Event
Report is hereby submitted.

Very truly yours,

Normand N. St. Laurent
Plant Superintendent

ELM/nm
Enclosure

cc: [3] NSARC Chairman (YAEC)
[1] Institute of Nuclear Power Operations (INPO)

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LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Yarkee Nuclear Power Station, Rowe, Massachusetts										DOCKET NUMBER (2) 0 5 0 0 0 0 2 9 1				PAGE (3) 1 OF 0 1		
TITLE (4) Insufficient Implementation Procedures for the Offsite Dose Calculation Manual																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENT AL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES				DOCKET NUMBER(S)			
0 3	1 1	8 6	8 6	0 0 2	0 0 0	0 4	1 0	8 6					0 5 0 0 0			
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR 5. (Check one or more of the following): (11)														
1		20.402(b)				20.405(c)				50.73(a)(2)(iv)				73.71(b)		
POWER LEVEL (10)		20.405(a)(1)(i)				50.38(c)(1)				50.73(a)(2)(v)				73.71(c)		
1 0 0		20.405(a)(1)(ii)				50.38(c)(2)				50.73(a)(2)(vi)				OTHER (Specify in Abstract below and in Text, NRC Form 306A)		
		20.405(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(vii)(A)						
		20.405(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(vii)(B)						
		20.405(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(k)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME Edwin L. May, Senior Engineer										TELEPHONE NUMBER						
										AREA CODE 4 1 3 4 2 4 - 5 2 6 1 1						
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)												X NO				
ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)																

During normal steady state Mode 1 operation, at 100 percent power, it was discovered by a Corporate QA auditor that contrary to Technical Specifications Section 6.8.1.c the Offsite Dose Calculation Manual (ODCM) was being implemented by insufficient written procedures. Calculation Method II of the ODCM is performed with the aid of a computer program. This computer program is an approved methodology but an implementing procedure had not been developed to describe its use. The Method II calculations are performed at the Corporate Headquarters by Radiological Engineering Group personnel. The Method II calculations are used as a backup for the Method I calculations, which are performed by plant personnel, using an approved procedure.

This event has been attributed to personnel error. An implementation procedure will be written for the Method II calculations by April 30, 1986. To prevent similar events in the future an interface program is being formulated which will be used to notify the appropriate Nuclear Services Division personnel of any responsibilities imposed on them by future plant Technical Specification changes. This program is expected to be in place by September 1, 1986.

This is the first event of this nature. There was no adverse effect on the health or safety of the public as a result of this occurrence.

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