(6-78		J.S. NUCLEAR REGULATORY	COMMISSION		APPLICATION FOR: eck and/or complete as appropriate)			
	APPLICATION FOR	BYPRODUCT MATER	IAL LICENSE	x	. NEW LICENSE			
Comp	trached instructions for details.	duplicate with the Division of F	uel Cycle and Material Safety.		b. AMENDMENT TO:			
Washii	ngton, DC 20555 or applicatio	nd Safeguards, U.S. Nuclear Re ns may be filed in person at th C. or 7915 Eastern Avenue, Si	e Commission's office at		C. RENEWAL OF:			
2. API	PLICANT'S NAME (Institution,	firm, person, etc.)	3. NAME OF PERSON TO BE	CON	TACTED REGARDING THIS			
	TERLAKE, INC.		J. Zbos					
	PHONE NUMBER: AREA CO	DE - NUMBER EXTENSION	312 933-5000	REA	CODE - NUMBER EXTENSION 5077			
4. API	22 933-5000 PLICANT'S MAILING ADDRES 0730 Burley Avenue nicago, Illinois 6		5. STREET ADDRESS WHERE LICENSED MATERIAL WILL BE USED (Include Zip Code) 10730 Burley Avenue Chicago, Illinois 60617					
	HE MODE SPACE I	S NEEDED FOR ANY ITEM	USE ADDITIONAL PROPE	RLY	KEYED PAGES.)			
6. IN	DIVIDUAL(S) WHO WILL	USE OR DIRECTLY SUPER	VISE THE USE OF LICENS	DM	ATERIAL			
/5	FULL N	training and experience of each in	Sivility named delicary	Т	ITLE			
. J.	Zbos		Assistant Blast Furnace Superintendent					
b.								
c.								
7	bert J. Oblon	CER	Attach a resume of person's tr 16 and 17 and describe his resp		and experience as outlined in Items ilities under Item 15.			
		8. LICENSE	DMATERIAL					
LINE	ELEMENT AND MASS NUMBER	CHEMICAL AND/OR PHYSICAL FORM	NAME OF MANUFACTUR AND MODEL NUMBER (If Sealed Source)	ER	MAXIMUM NUMBER OF MILLICURIES AND/OR SEALED SOURCES AND MAXIMUM ACTI- VITY PER SOURCE WHICH WILL BE POSSESSED AT ANY ONE TIME			
NO.	A	8	С		D			
(1)	Cesium - 137	Sealed Source	Kay-Ray 7700-G		2 each 1,000 mCi			
(2)	Americium-241	Sealed Source	Kay-Ray 7700-F		2 each 500 mCi			
(3)								
(4)								
	DESCRIBE USE OF LICENSED MATERIAL							
(1)	Measure Material Density							
(2)	Measurement of M	oisture in Coke						
(3)		With the best of the		74				
(4)	A Section							
FORM	NRC-313 (6-78)				1/6/62			

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. 1				F SEALED SOURCE	MANUFACTURER	MODEL NUMBER
-Z#0	SOURCE WILL BE	OR DEVICE IN WHICH E STORED OR USED.	ACH SEALED	8.		c.
(1)	Source Housing		Kay-Ray, Inc.		7051	
2)	Source Housing			Kay-Ray, Inc.		7100
3)						
(4)	H.H.					
		10. RA	DIATION DET	ECTION INSTRU		
J-2m0.	TYPE OF INSTRUMENT	MANUFACTURER'S NAME	MODEL NUMBER	NUMBER AVAILABLE D	RADIATION DETECTED (e)phe, bete, gemme, neutron) E	RANGE (millicoentgens/hour or counts/minute)
(1)	G.M. Tube Survey	Victoreen	#491	2	Alpha, Beta Gamma	0 - 100 mr/hr
(2)						
(3)						
(4)						
		11. CALIDA	ATION OF INS	TRUMENTS LIST	ED IN ITEM 10	
	101 Woodland eveland, Ohio	44104	RSONNEL MOI	NITORING DEVIC	CES	
	(Check and/or comple			SUPPLIER (Service Company)	EXCHANGE FREQUENCY	
	THERMOLUMINES		necessary documenta	monitoring for support tion. See a to Item 15-V	OUARTERLY	
(3	OTHER (Specify):		Tesponse to real 15 vi			OTHER (Specify):
-						
-	13. FACILITIES	AND EQUIPMENT (C	heck were appro	opriate and attach	annotated sketch(es)	and description(s).
00	STORAGE FACILI	CILITIES, PLANT FACIL TIES, CONTAINERS, SPE NG TOOLS OR EQUIPME	NT, ETC. NA	G (fixed and/or temp		
		AL WASTE DISPOSAL SE	14. WAS	TE DISPOSAL		
. 19	AME OF COMMERCI	AL HAD E DISTUSAL SE	THE EMPLOY			
b. IF	COMMERCIAL WAS	TE DISPOSAL SERVICE	IS NOT EMPLOY	ED, SUBMIT A DET	AILED DESCRIPTION	F METHODS WHICH WILL

INFORMATION REQUIRED FOR ITEMS 15, 16 AND 17

Describe in detail the information required for Items 15, 16 and 17. Begin each item on a separate page and key to the application as follows:

- 15. RADIATION PROTECTION PROGRAM. Describe the radiation protection program as appropriate for the material to be used including the duties and responsibilities of the Radiation Protection Officer, control measures, bioassay procedures (if needed), day-to-day general safety instruction to be followed, etc. If the application is for sealed source's also submit leak testing procedures, or if leak testing will be performed using a leak test kit, specify manufacturer and model number of the leak test kit.
- 16. FORMAL TRAINING IN RADIATION SAFETY. Attach a resume for each individual named in Items 6 and 7. Describe individual's formal training in the following areas where applicable. Include the name of person or institution providing the training, duration of training, when training was received, etc.
 - a. Principles and practices of radiation protection.
 - Radioactivity measurement standardization and monitoring techniques and instruments.
 - Mathematics and calculations basic to the use and measurement of radioactivity.
 - d. Biological effects of radiation.
- 17. EXPERIENCE. Attach a resurce for each individual named in Items 6 and 7. Describe individual's work experience with radiation, including where experience was obtained. Work experience or on-the-job training should be commensurate with the proposed use. Include list of radioisotopes and maximum activity of each used.

18. CERTIFICATE

(This item must be completed by applicant)

The applicant and any official executing this certificate on behalf of the applicant named in Item 2, certify that this application is prepared in conformity with Title 10, Code of Federal Regulations, Part 30, and that all information contained herein, including any supplements attached hereto, is true and correct to the best of our knowledge and belief.

WARNING.-18 U.S.C., Section 1001; Act of June 25, 1948; 62 Stat. 749; makes it a criminal offense to make a willfully felse statement or representation to any department or agency of the United States as to any matter within its jurisdiction.

b. CERTIFYING OFFICIAL (Signature)		
c. NAME (Type or print) F. K. Armour		
d. JITLE Vice President - Engrg. & Research		
•. DATE		

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- 0
- 15. This addendum describes the procedures to be followed as part of our radiation safety program. This addendum is sectioned as follows:
- I. Source Locations.
- II. Radiation Survey source housing maintenance.
- III. Control Measures.
- IV. Leak Testing.
- V. Procedure to be followed if source housing is damaged.
- VI. Worst case personnel radiation exposure calculation.
- VII. Notice to Employees.
- The enclosed sketches show the location of the source heads on the hoppers as well as the source locations in relation to other building areas.
- Initial radiation survey, wipe test, installation, servicing, maintenance, relocation and repair of the source holder will be performed by Kay-Ray. The initial radiation survey will be used to confirm the calculations shown in Section 15-VI of this item.
- If maintenance is required within the hopper vessel a lockout procedure will be employed to assure that the source
 head controls have been locked in the store position before
 employee hopper entry. The radiation protection officer
 will hold the gauge lockout keys. A chain barrier will
 be erected at all hopper entrances and will not be removed
 until the gauge source heads have been locked in the store
 position. All hopper entrance will be posted with a sign
 bearing the following words: "Positively no entrance to
 hopper until Kay-Ray gauge heads are locked out by radiation
 protection officer".
- 15-IV Kay-Ray will perform the leak testing on the source holder. The leak test kit used by Kay-Ray is either the General Radioisotope products WT-4 kit, or Kay-Ray, Inc. Model A kit, which have been approved by the NRC for use in the source wiping of Kay-Ray source holders. We wish to have our license worded to allow a 3 year source wipe interval on the device listed. An extension has been granted to Kay-Ray allowing a three-year interval for source wiping and we wish to have our license reflect this extended test period.

- The following procedure will be implemented in the event of damage to the source housing:

 1) Immediately evacuate the room containing the gauges.

 2) Inform the Radiation Protection Officer (Ext. 2715).

 3) Notify Kay-Ray (312-259-5600).
 - Inform by phone or telegram the regional NRC office of the accident.
 - 5) Limit access to stock house until a radiation survey and source wipe can be performed by qualified personnel or a representative of Kay-Ray.
- The attached calculation indicates a worst case operator 15-VI exposure of 3.0 millirems per year per cesium source. This exposure is based on the nearest operator location to the source housing and is less than 500 millirems per year which is well below the limits set in 10 CFR 20 for personel me 'toring equipment. The calculated radition exposure rate one 'll receive at the detector is approximately 0.5 millirems er hour or less. These low levels drop off according to the inverse square law and result in negligible operator exposure a few feet from the detector. These radiation exposures will be varified at the time of start-up. This will include the effects of radiation scattering if applicable. These provisions will be taken to verify that no one will receive a worst case exposure of 500 millirems per year at the detector side. The procedure for performing the above calculations, as supplied by Kay-Ray, Inc. has been included for reference.

D (minimum distance from gauge) = 10 feet

T (maximum time at minimum distance) = 4 hours

K (MR/HR at 10 feet) = .003

S (millicuries) = 1000

X = worst case exposure MR/YR

X = (K) (S) (T) (.25) = 3.0 millirems/year

15-VII Employees authorized to perform maintenance work within the hopper will be instructed in the following:

1) The location of the source heads.

2) The gauge source head lockout procedure.

- 3) The emergency procedure in the event of source head damage.
- 4) Not to perform work on or damage the source heads.

- The individual named in item #6 has 5 years experience in the operation and safety procedures of the Kay-Ray moisture gauging system previously installed at the Interlake stockhouse. At the time of reinstallation of the Kay-Ray coke moisture gauging system a review of the specific training necessary for the safe operation of the system will be implemented.
- 16-II The individual named in item #7 has had the following formal training in the use or radioactive materials:
 - 1. Indiana University
 B.A. Physics 1971
 Purdue University 1978
 Nuclear Reactor Engineering
 Total of 10 credit hours

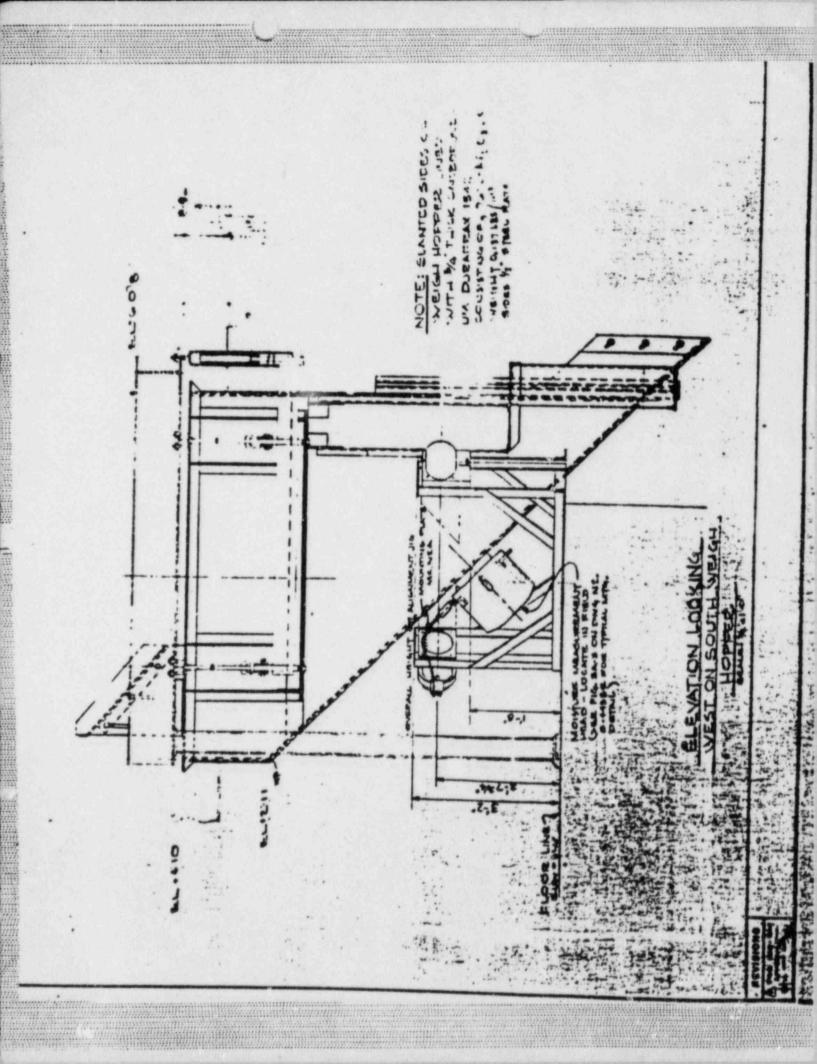
Course training in ionizing radiation and radioactivity control including:

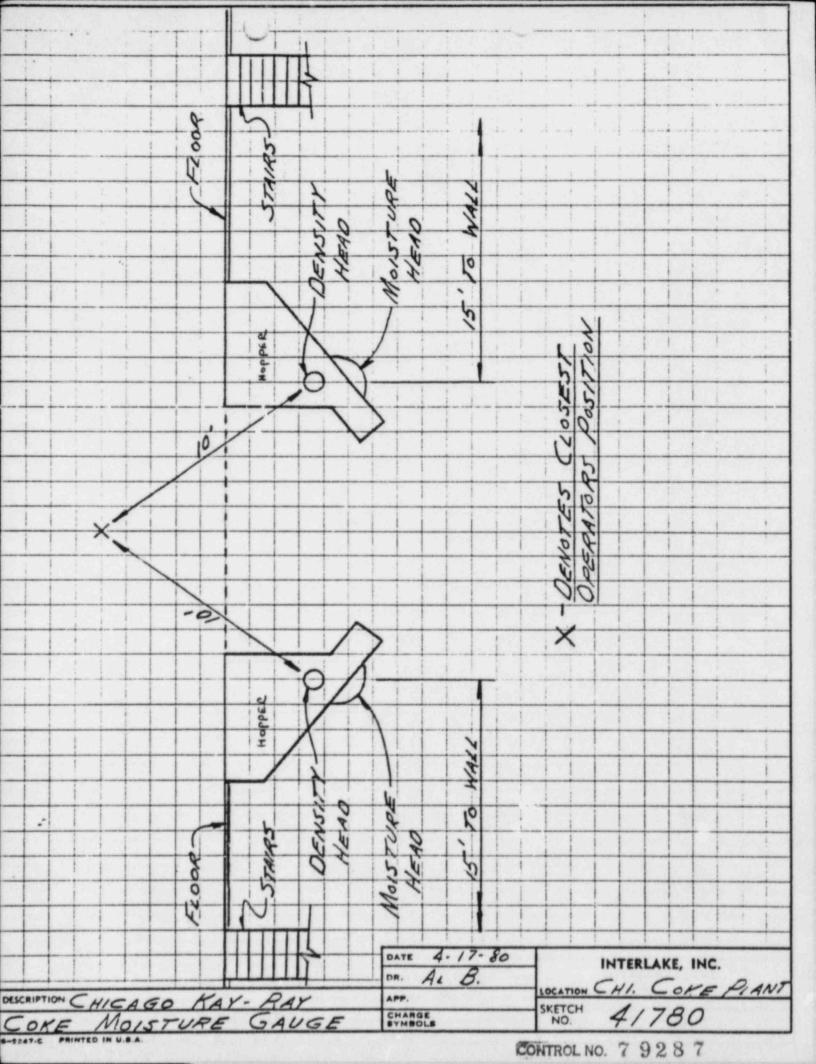
- A. Radioactive decay
- B. Attenuation of particle and photon radiation
- C. Methods of detection of Alpha, Beta and Gamma radiation (including energy dependence)
- D. Mathematics and calculations basic to use and measurement of radioactivity.
- American Iron and Steel Institute Basic Radiation Safety in the Steel Industry. Pittsburgh, Pennsylvania - 1978
 Classroom hours

Course training including:

- A. Physics of Radiation
- B. Shielding Methods
- C. Attenuation of Radiation
- D. Radiation Detection Devices and Methods (including energy dependence)
- E. Leak Test Methods
- F. Radiation Survey Techniques
- G. Mathematics and Calculations Basic to Use and Measurement of Radioactivity
- H. Biological Effect of Ionizing Radiation

The individual named in item 6 has had 5 years experience in 17-I the operation and safety procedures of the Kay-Ray coke moisture gauging system previously installed at the Interlake stock house. The individual named in item 7 has been responsible for the 17-II radiation protection program at Interlake, Inc. since June of 1976. He has been responsible for radiation safety in the use of sealed source thickness gauges containing the following by-product material. Cesium - 137 1. Americium -241 Strontium - 90 3. As well as X-Ray thickness gauges





PROCEDURE FOR CALCULATION OF WORST CASE RADIATION EXPOSURE TO OPERATING PERSONNEL

- Step 1. From the sketch drawn for item 15-I showing relationship of gauge to the operating area, note the minimum distance and maximum time that any single person will be in the vicinity of the gauge. Define this distance as D in feet and T in hours per day. If D is greater than 20 feet, use the 20 ft. value in the table.
- Step 2. From the attached find the table corresponding to the model number of the measurement head that is to be used in your application.
- Step 3. Round down to the next lower value of D found in Step 1 and note the normalized value of K.
- Step 4. Use the formula below to calculate X the worst case operator exposure per year.

 $X = K \times S \times T \times .25$

Where X = worst case exposure in mR/year

K = figure from Table 1

- S = Source size in millicuries that is used in the source head in your application. This factor corrects X for the activity in mc supplied with the respective source head.
- T = Hours/day
- .25 = Normalizing factor which converts the mR/hr figure to a yearly figure. This factor assumes a 40 hour week, 50 weeks per year, and 1000 mc source in each respective source head.

The Kay-Ray equipment supplied in your application has been specified to provide less than 500 mR/year exposure to any operating personnel. If this number is exceeded in your calculations, notify Kay-Ray where a review of your calculations will be made.

Example Calculation:

Suppose your application required a 7062 source housing with a 100 mc source. An operator stands within 5.75 feet of the gauge for a worse case average of 7 hours per day.

Following the step by step procedure above, the worst case exposure for this operator in mR/year is:

Step 1. D = 5.75T = 7

Step 2. Choose table corresponding to 7062 source head.

Worst Case Radiation Procedure (continued)

Step 3. Round down to next lower value of D or 5 feet.
The corresponding value of K from the table is
.59

Therefore: D = 5K = .59

Step 4. The value of S is 100 mc in this example.

Therefore: $X = K \times S \times T \times .25 = .59 \times 100 \times 7 \times .25$

X = 103.25 mR/year

Place this value of X in the space provided in Item 15-VI. Supply a copy of these calculations including the table supplied by Kay-Ray.

VALU OF D AND K FOR KAY-RAY IRCE HOLDER

			GAMMA SOURCE	HEADS		
KR Model	No. 7056	7050B	7051B	7062		
Allia.	7057	7060B	7061B	7062P	7063	7063P
1 (ft.)	K (mr/hr)	K (mr/hr)	K (mr/hr)	K (mr/hr)	K (mr/hr)	K (mr/hr)
0	0.63	12.5	3.0	500	50 .	15.0
1	0.05	0.59	0.17	11.22	2.3	0.82
2	0.02	0.18	0.06	3.31	0.74	0.26
3	0.009	0.09	0.03	1.56	0.34	0.12
4	0.006	0.05	0.02	0.90	. 0.20	0.07
5	0.004	0.03	0.01	0.59	0.14	0.05
6	0.003	0.02	0.008	0.41	0.09	0.03
7	0.002	0.02	0.006	. 0.31	0.08	0.03
8 9	0.002	0.01	0.005	0.24	0.06	0.02
9	0.001	0.01	0.004	0.19	0.06	0.02
10	0.001	0.009	0.003	0.15	0.03	0.01
11	0.0008	0.008	0.002	0.13	0.03	0.01
12	0.0007	0.006	0.002	0.11	0.02	0.009
13	0.0006	0.005	0.002	. 0.09	0.02	0.008
14	0.0005	0.005	0.002	0.08	0.02	0.007
15	0.0005	0.004	0.001	0.07	0.01	0.006
16	0.0004	0.004	0.001	0.06	. 0.01	0.005
17	0.0004	0.003	0.001	0.05	0.01	0.004
18	0.0003	0.003	0.001	0.05	0.01	0.004
19	0.0003	0.003	0.0009	0.04	0.01	0.004
20	0.0003	0.002	0.0008	0.04	0.008	0.003

KR Model				7067	7068
	7064	7064P	7065	7067P	7069
D (ft.)	K (mr/hr)				
0	15	3.25	120	5	7.5
	1.02	0.22	5.62	0.48	0.38
2	0.36	0.03	1.77	0.17	0.12
. 3	0.18	0.04	0.85	0.08	0.06
4	0.12	0.03	0.50	0.05	0.03
5	0.06	0.01	0.33	0.03	0.02
	0.05	0.01	0.23	0.02	0.01
7 8 9	0.04	0.007	0.17	0.02	0.01
8	0.03	0.006	0.13	0.01	0.009
	0.02	0.005	0.11	0.01	0.007
10	0.02	0.004	0.09	0.009	0.006
11	0.01	0.003	0.07	0.007	0.005
. 12	0.01	0.003	0.06	0.006	0.004
13	0.01	0.003	0.05	0.006	0.003
14	0.01	0.002	0.04	0.005	0.003
15	0.006	0.002	0.04	0.004	0.003
16	0.006	0.002	0.03	0.004	0.002
17	0.006	0.001	0.03	0.003	0.002
18	0.006	0.001	0.03	0.003	0.002
9	0.005	0.001	0.02	0.002	0.002
20	0.005	0.001	0.02	0.002	0.001