



UNITED STATES  
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

May 19, 1997

50-285

Mr. S. K. Gambhir  
Division Manager - Production Engineering  
Omaha Public Power District  
Fort Calhoun Station FC-2-4 Adm.  
Post Office Box 399  
Hwy. 75 - North of Fort Calhoun  
Fort Calhoun, Nebraska 68023-0399

SUBJECT: FORT CALHOUN STATION, UNIT NO. 1 - COMPLETION OF LICENSING ACTION  
FOR GENERIC LETTER 95-03, "CIRCUMFERENTIAL CRACKING OF STEAM  
GENERATOR TUBES," DATED APRIL 28, 1995 (TAC NO. M92243)

Dear Mr. Gambhir:

On April 28, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-03, "Circumferential Cracking of Steam Generator Tubes," to all holders of operating licenses or construction permits for pressurized-water reactors. The NRC issued GL 95-03 for three principal reasons:

- (1) Notify addressees about the safety significance of the recent steam generator tube inspection findings at Maine Yankee Atomic Power Station.
- (2) Request that all addressees implement the actions described within the generic letter.
- (3) Require that all addressees submit to the NRC a written response regarding implementation of the requested actions.

In addition, GL 95-03 alerted addressees to the importance of performing comprehensive examinations of steam generator tubes using techniques and equipment capable of reliably detecting the types of degradation to which the steam generator tubes may be susceptible. The staff also noted that the performance of steam generator tube examinations is controlled, in part, by Appendix B to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR Part 50).

In GL 95-03, the NRC staff also requested that licensees take the following actions:

- (1) Evaluate recent operating experience with respect to the detection and sizing of circumferential indications to determine the applicability to their plants.
- (2) On the basis of the evaluation in Item (1) above, as well as past inspection scope and results, susceptibility to circumferential cracking, threshold of detection, expected or inferred crack growth rates, and other relevant factors, develop a safety assessment justifying continued operation until the next scheduled steam generator tube inspections.

DFOI/0

NRC FILE CENTER COPY

250000  
9705270164 970519  
PDR ADDCK 05000285  
P PDR

May 19, 1997

- (3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

To document the outcome of these actions, the NRC staff requested that addressees prepare and submit the following:

- (1) A safety assessment justifying continued operation, predicated on the evaluation performed in accordance with requested actions 1 and 2 (above).
- (2) A summary of the inspection plans developed in accordance with requested action 3 (above) and a schedule for the next planned inspection.

In response to GL 95-03, OPPD provided letters dated June 23, 1995, and July 31, 1996, for Fort Calhoun Station. These submittals provided the information requested by GL 95-03; therefore TAC No. M92243 is closed. For your information, the staff's findings regarding this issue are contained in NUREG-1604, "Circumferential Cracking of Steam Generator Tubes." If you have any questions regarding this matter, please contact Raynard Wharton at (301) 415-1396.

Sincerely,

Original Signed By

L. Raynard Wharton, Project Manager  
Project Directorate IV-2  
Division of Reactor Projects - III/IV  
Office of Nuclear Reactor Regulation

Docket No. 50-285

DISTRIBUTION:

Docket File	EPeyton
PUBLIC	RWharton
PDIV-2 Reading	KKarwoski
JRoe	SDembek
EGAl	OGC
WBateman	PGwynn, RIV
ACRS	WJohnson, RIV
JStrosnider	

Document Name: FCS92243.LTR

OFC	PDIV-2/PM	PDIV-2/LA
NAME	RWharton:ye	EPeyton
DATE	5/19/97	5/16/97

OFFICIAL RECORD COPY

Mr. S. K. Gambhir

- 3 -

May 19, 1997

cc:

Winston & Strawn

ATTN: Perry D. Robinson, Esq.

1400 L Street, N.W.

Washington, DC 20005-3502

Mr. Jack Jensen, Chairman

Washington County Board

of Supervisors

Blair, Nebraska 68008

Mr. Wayne Walker, Resident Inspector

U.S. Nuclear Regulatory Commission

Post Office Box 309

Fort Calhoun, Nebraska 68023

Regional Administrator, Region IV

U.S. Nuclear Regulatory Commission

611 Ryan Plaza Drive, Suite 1000

Arlington, Texas 76011

Ms. Cheryl Rodgers, LLRW Program Manager

Environmental Protection Section

Nebraska Department of Health

301 Centennial Mall, South

P.O. Box 95007

Lincoln, Nebraska 68509-5007

Mr. James W. Chase, Manager

Fort Calhoun Station

Post Office Box 399

Fort Calhoun, Nebraska 68023

Mr. James W. Tills

Manager - Nuclear Licensing

Omaha Public Power District

Fort Calhoun Station FC-2-4 Adm.

Post Office Box 399

Hwy. 75 - North of Fort Calhoun

Fort Calhoun, Nebraska 68023-0399