Tennessee Valley Authority, Post Office Box 2000, Soddy-Daisy, Tennessee 37379-2000

July 14, 1997

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

Gentlemen:

In the Matter of) Docket Nos.50-327
Tennessee Valley Authority) 50-328

SEQUOYAH NUCLEAR PLANT (SQN) - JUNE 1997 MONTHLY OPERATING REPORT

Enclosed is the June 1997 Monthly Operating Report as required by SQN Technical Specification 6.9.1.10.

If you have any questions concerning this matter, please telephone me at (423) 843-7170 or James D. Smith at (423) 843-6672.

Sincerely,

Pedro Salas
Site Licensing and Industry Affairs Manager

Enclosure
cc: See page 2

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TENNESSEE VALLEY AUTHORITY

SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT

TO THE

NUCLEAR REGULATORY COMMISSION

JUNE 1997

UNIT 1

DOCKET NUMBER 50-327

LICENSE NUMBER DPR-77

UNIT 2

DOCKET NUMBER 50-328

LICENSE NUMBER DPR-79

OPERATIONAL SUMMARY
JUNE 1997

UNIT 1

Unit 1 generated 829,950 megawatthours (MWh) (gross) electrical power during June with a capacity factor of 99.8 percent. On June 28 at 1852 EDT, a power reduction to 62 percent was initiated for planned maintenance on the 1B-B main feed pump to repair an oil leak. Reactor power increase was initiated on June 30 at 0745 EDT, and Unit 1 was operating at 100 percent reactor power at 2230 EDT that day and through the end of June.

UNIT 2

Unit 2 generated 793,742 megawatthours (MWh) (gross) electrical power during June with a capacity factor of 95.4 percent. On June 5 at 1347 EDT, an alarm was received on letdown heat exchanger flow temperature abnormal. Investigation showed that a letdown flow control valve had closed. Excess letdown was established. The flow control valve closed as a result of a degraded actuator diaphragm. On June 6 at 1633 EDT, a power decrease was initiated and reactor power was stabilized at approximately 28 percent on June 7 at 1557 EDT. Reactor power increase was initiated on June 8 at 1638 EDT, and Unit 2 was operating at 100 percent power on June 9 at 2330 EDT.

On June 18 at 2007 EDT, a reactor power decrease was initiated to remove 2B main feed pump from service to repair a leaking seal water injection header that could not be isolated. Power was stabilized at 56 percent on June 19 at 0155 EDT and repairs were completed. Reactor power increase was initiated on June 19 at 0515 EDT, and Unit 2 was operating at 100 percent power on June 19 at 1727 EDT.

Unit 2 continued to operate at 100 percent reactor power through the end of June.

SUMMARY OF POWER-OPERATED RELIEF VALVES (PORVs) AND SAFETY VALVES CHALLENGES

No PORVs or safety valves were challenged in June.

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-327 UNIT NO. One DATE: 07-07-97

COMPLETED BY: T. J. Hollomon TELEPHONE: (423) 843-7528

MONTH: JUNE 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1144	17	1142
2	1143	18	1142
3	1136	19	1142
4	1133	20	1140
5	1133	21	1144
6	1143	22	1141
7	1146	23	1139
8	1149	24	1138
9	1144	25	1138
10	1143	26	1137
11	1145	27	1139
12	1147	28	1079
13	1144	29	674
14	1144	30	876
15	1144	31	NA
16	1142		

AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-328 UNIT NO. Two DATE: 07-07-97

COMPLETED BY: T. J. Hollomon TELEPHONE: (423) 843-7528

MONTH: JUNE 1997

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1142	17	1136
2	1133	18	1116
3	1143	19	943
4	1140	20	1136
5	1137	21	1136
6	1072	22	1135
7	379	23	1130
8	261	24	1134
9	1003	25	1133
10	1137	26	1127
11	1140	27	1127
12	1140	28	1126
13	1139	29	1127
14	1136	30	1128
15	1138	31	NA
16	1138		

OPERATING DATA REPORT

DOCKET NO. 50-327
DATE 07/07/97
COMPLETED BY T. J. Hollomon
TELEPHONE (423) 843-7528

OPERATING STATUS

1. Unit Name: Sequoyah Unit One
2. Reporting Period: June 1997
3. Licensed Thermal Power (MWt): 3411.0
4. Nameplate Rating (Gross (MWe): 1220.6
5. Design Electrical Rating (Net MWe): 1148.0
6. Maximum Dependable Capacity (Gross MWe): 1155.0
7. Maximum Dependable Capacity (Net MWe): 1117.0
8. If Changes Occur in Capacity Ratings (Item Numbers 3 Through 7) Since Last Report, Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any: N/A

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	720	4,343	140,256
12. Number of Hours Reactor Was Critical	720.0	3,141.3	80,464
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	720.0	3,104.3	78,675.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWh)	2,399,755.2	9,977,676.1	256,332,663
17. Gross Electrical Energy Generated (MWh)	829,950	3,473,223	87,265,363
18. Net Electrical Energy Generated (MWh)	803,462	3,348,875	83,726,513
19. Unit Service Factor	100.0	71.5	56.1
20. Unit Availability Factor	100.0	71.5	56.1
21. Unit Capacity Factor (Using MDC Net)	99.9	69.0	53.4
22. Unit Capacity Factor (Using DER Net)	97.2	67.2	52.0
23. Unit Forced Outage Rate	0.0	0.0	31.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE 07/07/97
COMPLETED BY T. J. Hollomon
TELEPHONE (423) 843-7528

OPERATING STATUS

1. Unit Name: Sequoyah Unit Two
2. Reporting Period: June 1997
3. Licensed Thermal Power (MWt): 3411.0
4. Nameplate Rating (Gross (MWe): 1220.6
5. Design Electrical Rating (Net MWe): 1148.0
6. Maximum Dependable Capacity (Gross MWe): 1155.0
7. Maximum Dependable Capacity (Net MWe): 1117.0
8. If Changes Occur in Capacity Ratings (Item Numbers 3 Through 7) Since Last Report, Give Reasons:

9. Power Level to Which Restricted, If Any (Net MWe): N/A
10. Reasons for Restrictions, If Any: N/A

	This Month	Yr-to-Date	Cumulative
11. Hours in Reporting Period	720	4,343	132,216
12. Number of Hours Reactor Was Critical	720.0	4,343.0	83,945
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	720.0	4,343.0	82,015.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy Generated (MWh)	2,315,918.4	14,620,226.3	262,359,880
17. Gross Electrical Energy Generated (MWh)	793,742	5,050,609	89,249,666
18. Net Electrical Energy Generated (MWh)	767,646	4,885,157	85,564,876
19. Unit Service Factor	100.0	100.0	62.0
20. Unit Availability Factor	100.0	100.0	62.0
21. Unit Capacity Factor (Using MDC Net)	95.4	100.7	57.9
22. Unit Capacity Factor (Using DER Net)	92.9	98.0	56.4
23. Unit Forced Outage Rate	0.0	0.0	30.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Unit 2 Cycle 8 Refueling Outage is scheduled to begin in October 3, 1997 with a duration of 50 days.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

DOCKET NO.: 50-327
UNIT NAME: One
DATE: 07/07/97
COMPLETED BY: T. J. Hollomon
TELEPHONE: (423) 843-7528

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JUNE 1997

No.	Date	Type[1]	Outage Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
3	970628	S	N/A	B	5	N/A	N/A	N/A	On June 28 at 1852 EDT, a power reduction to 62 percent was initiated for planned maintenance on the 1B-B main feed pump to repair an oil leak.

[1]F: Forced
S: Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing Outage
5-Reduction
9-Other

[4]Exhibit G-Instructions for Preparation of Data Entry sheets for Licensee Event Report (LER) File (NUREG-1022)

[5]Exhibit I - Same Source

DOCKET NO.: 50-328
UNIT NAME: Two
DATE: 07/07/97
COMPLETED BY: T. J. Hollomon
TELEPHONE: (423) 843-7528

UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH: JUNE 1997

No.	Date	Type[1]	Outage Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
1	970605	F	N/A	A	5	N/A	N/A	N/A	On June 5 at 1347 EDT, an alarm was received on letdown heat exchanger flow temperature abnormal. Investigation showed that the letdown flow control valve 2-FCV-62-69 had closed. Excess letdown was established. It was determined that the diaphragm had failed on the valve. The valve was repaired and the system was returned to service.
2	970618	F	N/A	A	5	N/A	N/A	N/A	On June 18 at 2007 EDT, a reactor power decrease was initiated to remove 2B main feed pump from service. The seal water injection header was leaking and could not be isolated. Repair of the line was performed and the system was returned to service.

[1]F: Forced
S: Scheduled

[2]Reason:
A-Equipment Failure (Explain)
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training and License Exam
F-Administrative
G-Operational Error (Explain)
H-Other (Explain)

[3]Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuation of Existing Outage
5-Reduction
9-Other

[4]Exhibit G-Instructions for Preparation of Data Entry sheets for Licensee Event Report (LER) File (NUREG-1022)

[5]Exhibit I - Same Source