



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

May 19, 1997

Mr. Roy A. Anderson
Senior Vice President,
Nuclear Operations
Florida Power Corporation
ATTN: Manager, Nuclear Licensing
Crystal River Energy Complex
15760 W Power Line Street
Crystal River, Florida 34428-6708

SUBJECT: CRYSTAL RIVER UNIT 3 - COMPLETION OF LICENSING ACTION FOR GENERIC
LETTER 95-03, "CIRCUMFERENTIAL CRACKING OF STEAM GENERATOR TUBES,"
DATED APRIL 28, 1995 (TAC NO. M92237)

Dear Mr. Anderson:

On April 28, 1995, the U.S. Nuclear Regulatory Commission (NRC) issued Generic Letter (GL) 95-03, "Circumferential Cracking of Steam Generator Tubes," to all holders of operating licenses or construction permits for pressurized-water reactors. The NRC issued GL 95-03 for three principal reasons:

- (1) Notify addressees about the safety significance of the recent steam generator tube inspection findings at Maine Yankee Atomic Power Station.
- (2) Request that all addressees implement the actions described within the generic letter.
- (3) Require that all addressees submit to the NRC a written response regarding implementation of the requested actions.

In addition, GL 95-03 alerted addressees to the importance of performing comprehensive examinations of steam generator tubes using techniques and equipment capable of reliably detecting the types of degradation to which the steam generator tubes may be susceptible. The staff also noted that the performance of steam generator tube examinations is controlled, in part, by Appendix B to Title 10, Part 50, of the *Code of Federal Regulations* (10 CFR Part 50).

In GL 95-03, the NRC staff also requested that licensees take the following actions:

- (1) Evaluate recent operating experience with respect to the detection and sizing of circumferential indications to determine the applicability to their plants.
- (2) On the basis of the evaluation in Item (1) above, as well as past inspection scope and results, susceptibility to circumferential cracking,

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threshold of detection, expected or inferred crack growth rates, and other relevant factors, develop a safety assessment justifying continued operation until the next scheduled steam generator tube inspections.

- (3) Develop plans for the next steam generator tube inspections as they pertain to the detection of circumferential cracking. The inspection plans should address, but not be limited to, scope (including sample expansion criteria, if applicable), methods, equipment, and criteria (including personnel training and qualification).

To document the outcome of these actions, the NRC staff requested that addressees prepare and submit the following:

- (1) A safety assessment justifying continued operation, predicated on the evaluation performed in accordance with requested actions 1 and 2 (above).
- (2) A summary of the inspection plans developed in accordance with requested action 3 (above) and a schedule for the next planned inspection.

In response to GL 95-03, you provided letters dated June 22, 1995 and July 25, 1996. These submittals provided the information requested by GL 95-03; therefore the technical assignment control (TAC) No. 92237 under which our effort relating to the GL was performed is closed. For your information, the staff's findings regarding this issue are contained in NUREG-1604, "Circumferential Cracking of Steam Generator Tubes."

If you have any questions regarding this matter, please contact me at 301/415-1471.

Sincerely,
Original signed by
L. Raghavan, Project Manager
Project Directorate II-3
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

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Docket File

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GENERATING PLANT

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