PUBLIC SERVICE COMPANY OF COLORADO
FORT ST. VRAIN NUCLEAR GENERATING STATION

MONTHLY OPERATIONS REPORT

NO. 146

March, 1986

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This report contains the highlights of the Fort St. Vrain, Unit No. 1, activities operated under the provisions of the Nuclear Regulatory Commission Operating License DPR-34. This report is for the month of March, 1986.

# 1.0 NARRATIVE SUMMARY OF OPERATING EXPERIENCE AND MAJOR SAFETY RELATED MAINTENANCE

The reactor remained critical throughout the month of March, 1986. The rise in power continued as allowed by primary coolant moisture level. The Interlock Sequence Switch (ISS) was placed in the "low power" position on March 13, 1986. On March 18, 1986, all four helium circulators were brought to speed and the core was orificed for equal region outlet temperatures. A maximum power level of 12.9% was achieved on March 29, 1986. Power is limited to 35% power pending completion of Environmental Oualification work.

On March 13, 1986, during a planned surveillance inspection of PCRV tendons, approximately twenty-five gallons of water was found in Tendon CM 4.6 when the end cap was loosened. This event will be investigated and reported to the Nuclear Regulatory Commission in Licensee Event Report 84-005 (revised report).

Spent fuel shipping activities continued.

The turbine valves have been successfully stroked in anticipation of rolling the turbine in April.

Preparations are continuing for the upcoming Environmental Qualification shutdown.

2.0 SINGLE RELEASES OF RADIOACTIVITY OR RADIATION EXPOSURE IN EXCESS OF 10% OF THE ALLOWABLE ANNUAL VALUE

None

3.0 INDICATION OF FAILED FUEL RESULTING FROM IRRADIATED FUEL EXAMINATIONS

None

4.0 MONTHLY OPERATING DATA REPORT

Attached

#### OPERATING DATA PEPONT

50-267 DOCKET NO. April 10, 1986 DATE Frank J. Novachek

COMPLETED BY (303) 785-2224 ext. 201

RATING STATE		N	OTES	
Unit Name: Fort St	. Vrain, No. 1			
Reporting Period: 860301	through 860331			
Licensed Thermal Power (MFt):	842			
Nameplate Rating (Gross Mwe):	342			
Design Electrical Rating (Net	: Mie): 330			
Maximum Dependable Canacity	(Gross Mie): 342			
Maximum Dependable Capacity	(Net 15/e): 330			
If Changes Occur in Capacity None	Ratings (Items Number 3	Through 7) Sin	ce Last Report, Give	heasons:
Power Level To Frich Restrict Reasons for Restrictions, If Environmental Qualifi	Any: Limited to 35		ding completion	n of
	This h		ear to Date C	
Hours in Reporting Period		744.0	2,160.0	59.185.
Number of Hours Reactor Was (	ritical	744.0	1,090.5	29,171.
Reactor Reserve Foundown House	78	0.0	0.0	0.
Hours Generator (n-Line		0.0	0.0	18,468
Unit Reserve Shutzovn Rours		0.0	0.0	0.
Gross Thermal Energy Generate	id (ME)	43,760.6	45,734.6	9,940,817
Gross Electrical Energy Gener	raced (MWH)	0.0	0.0	3,248,594
Net Electrical Energy General	ed (1978)	-4,125.0	-10,144.0	2,886,129
Unit Service Factor		0.0	0.0	31.
		-		
Unit Availability Parcor		0.0	0.0	31
Unit Availability Factor Unit Capacity Factor (Using )	-		0.0	
	DC Net)	0.0		14,
Unit Capacity Factor (Using )	DC Net)	0.0	0.0	14. 14.
Unit Capacity Fatter (Using ) Unit Capacity Fatter (Uring )	DC Net)  CD Net)  t f hunths (Type, Date,	0.0 0.0 0.0 0.0	0.0	31. 14. 14. 59. mental
Unit Capacity Factor (Using ) Unit Capacity Factor (Oring I Unit Forced Outage Rate Shutdowns Scheduled Over Next	DC Net)  OER Net)  of hunths (Type, Date,  1986, 2928 hours	0.0 0.0 0.0 0.0 and Duration of	0.0 0.0 0.0 Each): Environ	14, 14, 59,
Unit Capacity Factor (Using ) Unit Capacity Factor (Uring ) Unit Forcer Outage hate Shutdowns Scheduled Over News Qualification, May 31 Li Shut Down at End of Report	DC Net)  If Net)  If funths (Type, Date, , 1986, 2928 hours Feriod, Estimated Date	0.0 0.0 0.0 0.0 and Duration of S.	0.0 0.0 0.0 Each): Environ	14, 14, 59,
Unit Capacity Factor (Using S Unit Capacity Factor (Uring I Unit Forced Outage Rate Shutdowns Scheduled Over News Qualification, May 31 16 Shut Down at End of Report Units In Test Status (Frior	DC Net)  If Net)  If funths (Type, Date, , 1986, 2928 hours Feriod, Estimated Date	0.0 0.0 0.0 0.0 and Duration of S.	0.0 0.0 0.0 Each): Environ	14, 14, 59, mental

N/A

N/A

### AVERAGE DAILY UNIT POWER LEVEL

Docket No.	50-267	
Unit	Fort St. Vrain No.	1
Date	April 10, 1986	
Completed By	Frank J. Novachek	
Telephone	(303) 785-2224 ext.	201

March, 1986  AVERAGE DAILY POWER LEVEL	DAY	AVERAGE DAILY POWER LEVE
(MWe-Net)		(MWe-Net)
0.0	17	0.0
0.0	18	0.0
0.0	19	0.0
0.0	20	0.0
0.0	21	0.0
0.0	22	0.0
0.0	23	0.0
0.0	24	0.0
0.0	25	0.0
0.0	26	0.0
0.0	27	0.0
0.0	28	0.0
0.0	29	0.0
0.0	30	0.0
0.0	31	0.0
0.0		

<sup>&</sup>quot;Generator on line but no net generation.

UNIT SHUTDOWNS AND POWER REDUCTORS

INCRET IN. 50-267

IMIT NAME Fort St. Vrain No. 1

DATE April 10, 1986

COMPLETED BY Frank J. Novachek

TELEFTHONE (303) 785-2224 ext. 201

REFORT FORTH March, 1986

110.	DATE	TYPE	DURATION	REASON	PETHOD OF SHUTTING DOWN REACTOR	LER #	SYSTEM	COREONERI	CAUSE AND CORRECTIVE ACTION TO PREVENT RECURBING.
6-001	860207	F.	744.0	Н	N/A	N/A	AB	XXXXXX	Primary Coolant Cleanup
								•	
		. :							
								1.3	

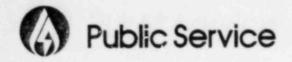
## REFUELING INFORMATION

1.	Name of Facility	Fort St. Vrain Unit No. 1		
2.	Scheduled date for next refueling shutdown	October 1, 1987		
3.	Scheduled date for restart   following refueling.	November 1, 1987		
4.	Will refueling or resumption of operation thereafter require a technical specification change or other license amendment?	No		
	If answer is yes, what, in general, will these be?			
	If answer is no, has the reload! fuel design and core configura-! tion been reviewed by your !! Plant Safety Review Committee ! to determine whether any unre-! viewed safety questions are associated with the core reload! (Reference 10 CFR Section 50.59)?	No		
	If no such review has taken   place, when is it scheduled?	1986		
5.	Scheduled date(s) for submit- ting proposed licensing action   and supporting information.			
6.	Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures.			
7.	The number of fuel assemblies (a) in the core and (b) in the spent fuel storage pool.	a) 1482 HTGR fuel elements b) 40 spent fuel elements		

#### REFUELING INFORMATION (CONTINUED)

8.	The present licensed spent fuell pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned, in number of fuel assemblies.	Capacity is limited in size to about one-third of core (approximately 500 HTGR elements). No change is planned.
9.	The projected date of the last   refueling that can be distinct charged to the spent fuel pool assuming the present licensed capacity.	1992 under Agreements AT(04-3)-633 and DE-SC07-79ID01370 between Public Service Company of Colorado, and General Atomic Company, and DOE.*

\* The 1992 estimated date is based on the understanding that spent fuel discharged during the term of the Agreements will be stored by DOE at the Idaho Chemical Processing Plant. The storage capacity has evidently been sized to accommodate eight fuel segments. It is estimated that the eighth fuel segment will be discharged in 1992.



**Public Service** Company of Colorado

16805 WCR 19 1/2, Platteville, Colorado 80651

April 15, 1986 Fort St. Vrain Unit No. 1 P-86287

Office of Inspection and Enforcement ATTN: Document Control Desk U. S. Nuclear Regulatory Commission Washington, D.C. 20555

Docket No. 50-267

SUBJECT: Monthly Operations

Report For March, 1980

REFERENCE: Facility Operating

License No. DPR-34

Dear Sir:

Enclosed, please find the Monthly Operations Report for the month of March, 1986.

If you have any questions, please contact Mr. Mike Holmes at (303) 480-6960.

Sincerely,

J. W. Gahm

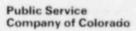
Manager, Nuclear Production Fort St. Vrain Nuclear Generating Station

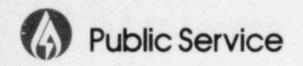
Enclosure

cc: Mr. R. D. Martin,

Regional Administrator, Region IV

JWG:djm





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