UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

January 2, 1986

Docket No. 50-293

Mr. William D. Harrington
Senior Vice President, Nuclear
Boston Edison Company
800 Boylston Street
Boston, Massachusetts 02199

Dear Mr. Harrington:

SUBJECT: GENERIC LETTER 84-11, INSPECTION OF STAINLESS STEEL PIPING

Re: Pilgrim Nuclear Power Station

Our Generic Letter 84-11 (GL 84-11) requested licensees to address their current plans relative to inspections of recirculation and residual heat removal piping which is subject to intergranular stress corrosion cracking (IGSCC). The letter also discussed leakage detection measures. We reviewed your response dated June 4, 1984 and found that it provided the appropriate information.

However, we noted your response to Item 1 of GL 84-11 that "If appropriate, an augmented inspection program of piping susceptible to IGSCC will be developed and submitted to the NRC for review prior to RFO 7." If you have concluded that an augmented inspection program is appropriate, please submit it for NRC review as soon as possible, so that any NRC comments can be resolved without affecting the outage schedule.

Recently reported ultrasonic inspection results by various utilities have shown that the performance by qualified UT personnel lacks uniformity and consistency. To resolve this concern, the NRC has determined that all UT personnel performing detection and evaluation should be qualified by an upgraded program at the EPRI NDE Center. Therefore, we suggest that you contact the EPRI NDE Center to arrange for requalification of the UT personnel to be used during the 1986 outage.

The UT personnel performing inspection of overlay repaired welds should be trained and qualified at the NDE Center, using the demonstrated procedures and techniques.

The allowable flaw sizes in Code IWB-3640 are currently being revised to take account of the low toughness associated with flux welds. Pending publication of the revised IWB-3640, the staff has developed the following interim guidelines for use of the present IWB-3640 to derive the allowable flaw sizes for flux welds:

(i) The secondary stresses (mainly thermal stresses) should be included in the calculation of the stress ratio.

(ii) The flaw size limits for the flux welds are 2/3 of the Code IWB-3640 allowables at a stress ratio as modified by (i).

Using the above guidelines, overlay repair may now be required for some unrepaired welds that were previously justified for continued service.

In the event you wish to discuss any of the above comments, please contact your NRC project manager.

Sincerely,

Original signed by:

John A. Zwolinski, Director
BWR Project Directorate #1
Division of BWR Licensing

cc: See next page

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