UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W., SUITE 2900
ATLANTA, GEORGIA 30323-0199

March 27, 1996

Florida Power Corporation
Crystal River Energy Complex
Mr. P. M. Beard, Jr. (SA2A)
Sr. VP, Nuclear Operations
ATTN: Mgr., Nuclear Licensing
15760 West Power Line Street
Crystal River, FL 34428-6708

SUBJECT: PLANT PERFORMANCE REVIEW (PPR) - CRYSTAL RIVER 3

Gentlemen:

On February 28, 1996, the regional staff completed the semiannual Plant Performance Review (PPR) of Crystal River 3. The staff conducts these reviews for all operating nuclear power plants to develop an integrated understanding of safety performance. The results are used by regional management to facilitate planning and allocation of inspection resources. The PPR for Crystal River 3 involved the participation of all technical divisions in evaluating inspection results and safety performance information for the period September 1995 through February 1996. PPRs provide regional management with a current summary of licensee performance and serve as inputs to the NRC Systematic Assessment of Licensee Performance (SALP) and senior management meeting (SMM) reviews.

This letter advises you of our planned inspection effort resulting from the Crystal River 3 PPR review. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability to be resolved in advance of inspector arrival onsite. The enclosure details our inspection plan for the next six (6) months. The rationale or basis for each inspection outside the core inspection program is provided so that you are aware of the reasons for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

During each NRC inspection planned during this period, specific attention will be given to the verification of selected UFSAR commitments. Applicable portion(s) of the UFSAR that relate to the inspection activities will be reviewed and verification made that the UFSAR commitments have been properly implemented into plant practices, procedures and/or parameters. The goal is to determine the accuracy of the UFSAR regarding existing plant practices and conditions by providing specific attention to the UFSAR when performing various reactor inspections. Inspectors will not be judging the overall completeness of the UFSAR; rather, the inspections will focus on identifying differences between the UFSAR description and the plant.

We will inform you of any changes to the inspection plan. If you have any questions, please contact Kerry Landis at 404-331-5509.

Sincerely,

Kerry D. Landis

Kerry D. Landis, Chief
Reactor Projects Branch 3
Division of Reactor Projects

Docket No. 50-302
License Nos. DPR-72

Enclosure: Inspection Plan

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CRYSTAL RIVER INSPECTION PLAN

INSPECTION PROCEDURE/ TEMPORARY INSTRUCTION	TITLE/PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
	INITIAL OPERATOR EXAMINATION	3	03/11/96	PREPARATION
IP 73753	INSERVICE INSPECTION	1	03/11/96	ISI - REACTOR VESSEL 10 YEAR ISI. SERGE LINE FLAW. HPI NOZZLES INSPECTION
IP 73753	INSERVICE INSPECTION (INTEGRATED SG INSPECTION)	3	03/11/96	REGIONAL INITIATIVE -INTEGRATES SG INSPECTION, SG TUBE ISSUES
IP 92702	FOLLOWUP ON CORRECTIVE ACTIONS FOR VIOLATIONS AND DEVIATIONS	1	3/18/96	SETPOINTS
IP 83750	OCCUPATIONAL RADIATION EXPOSURE	1	03/18/96	RP CORE INSPECTION
IP 81700	PHYSICAL SECURITY PROGRAM FOR POWER REACTORS	1	03/18/96	SAFEGUARDS
	INITIAL OPERATOR EXAM	3	03/25/96	ADMINISTER EXAMINATION
IP 92703 IP 92720	FOLLOWUP - ENGINEERING CORRECTIVE ACTIONS	3	04/08/96	CCHE REGIONAL INITIATIVE CORRECTIVE ACTION PROGRAM
IP 71500 IP 62700	BALANCE OF PLANT INSPECTION MAINTENANCE PROGRAM IMPLEMENTATION	1	04/08/96	MAINTENANCE TO SUPPORT 24 MONTH REFUELING INTERVAL
IP 37550	ENGINEERING	2	04/22/96	CORE INSPECTION - FOCUS ON 50.72s/LERs, OPERABILITY, 50.59 EVALUATIONS, ENGINEERING SUPPORT TO OPS AND MAINT, SELF ASSESSMENT. ELECTRICAL MODIFICATIONS

INSPECTION PROCEDURE/ TEMPORARY INSTRUCTION	TITLE/PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 61726 IP 62703	SURVEILLANCE OBSERVATIONS MAINTENANCE OBSERVATIONS	1	05/20/96	CORE INSPECTION
IP 37550	ENGINEERING	1	06/10/96	DESIGN BASIS & OPERABILITY SUPPORT TO OPS AND MAINT
IP 71500 IP 62700	BALANCE OF PLANT INSPECTION MAINTENANCE PROGRAM IMPLEMENTATION	1	06/24/96	NEW PROCEDURES TO SUPPORT 24 MONTH REFUELING INTERVAL
IP 73753	INSERVICE INSPECTION (INTEGRATED SG INSPECTION)	2	07/08/96	REGIONAL INITIATIVE - INTEGRATED SG INSPECTION, SG TUBE ISSUES
IP 92720	CORRECTIVE ACTIONS	3	07/15/96	CAP
IP 81700	PHYSICAL SECURITY PROGRAM FOR POWER REACTORS	1	07/22/96	SAFEGUARDS - REGIONAL INITIATIVE
IP 93808	INTEGRATED PERFORMANCE ASSESSMENT PROCESS (IPAP)	5	07/22/96 07/29/96 08/19/96 08/26/96	IPAP PREP ONSITE IPAP PREP ONSITE IPAP INSPECTION IPAP INSPECTION
IP 92702	FOLLOWUP ON CORRECTIVE ACTIONS FOR VIOLATIONS AND DEVIATIONS	1	09/16/96	SET POINTS & MAKEUP TANK CURVE
IP 61726 IP 62703	SURVEILLANCE OBSERVATIONS MAINTENANCE OBSERVATIONS	1	09/16/96	CORE INSPECTION
IP 92903	FOLLOWUP - ENGINEERING	1	09/23/96	SERVICE WATER FOLLOWUP
IP 42001	EMERGENCY OPERATING PROCEDURES	3	10/07/96	EOP UPGRADE PROGRAM INSPECTION
IP 71500 IP 62700	BALANCE OF PLANT INSPECTION MAINTENANCE PROGRAM IMPLEMENTATION	1	10/07/96	NEW PROCEDURES TO SUPPORT 24 MONTH REFUELING INTERVAL

INSPECTION PROCEDURE/ TEMPORARY INSTRUCTION	TITLE/PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 82301 IP 82302	EVALUATION OF EXERCISES FOR POWER REACTORS REVIEW OF EXERCISE OBJECTIVES AND SCENARIOS FOR POWER REACTORS	3	10/16/96	CORE INSPECTION
IP 71001	LICENSED OPERATOR REQUALIFICATION PROGRAM EVALUATION	2	11/04/96	OPERATOR REQUALIFICATION

SENIOR MANAGEMENT BRIEFING - ST LUCIE

A. Plant Performance (See Attached Performance Power Profiles)

The unit was shutdown on August 1, 1995, as a result of Hurricane Erin. A series of problems including RCP seal failure, both PORVs inoperable due to incorrect assembly, SDC relief valve problems, associated problem with several other relief valves, inadvertent spraydown of the containment, catastrophic piston failure on 1B emergency diesel generator, and a leaking flange on a pressurizer safety valve have prevented the unit from restarting. With the unit down the licensee planned to correct a large number of operator-work-arounds and other plant deficiencies. The next refueling outage is scheduled for April 4, 1996.

Unit 2 was shutdown on April 25, 1995, for approximately seven hours to replace a main turbine digital electro hydraulic power supply. Unit 2 was shutdown on August 1 as a result of Hurricane Erin and restarted on August 4. The refueling outage began October 9 and was expanded from 30 to 48 days.

B. SALP

	SALP ending January 1994	SALP ending May 1992
Operations	1	1
Maintenance	1	1
Engineering	1	1
Plant Support	1	
Rad. Con.		1
Security		1
Emerg. Prep.		1
SA/QV		1

C. INPO

INPO assessment July 1995 - Category 1
INPO/WANO assessment April 1994 - Category 1

D. Precursor Events None

E. PRA Insights

CE 2 loop plant
Internal CDF - 2.6E-6
Containment Type - Dry, Ambient Pressure

Principal Risk Contributor -	LOCA	49%
	Transients	20%
	SBO	10%

IPEEE Submittal Date - Unit 1: December 1994 (Non-seismic), September 1992 (Seismic); Unit 2: February 1995 (Seismic)

Enforcement History

No escalated enforcement in 1994. Enforcement conference on inoperable PZR PORV, September 25, 1995, with SLIII and base CP yet to be issued. Pending escalated enforcement on SDC relief valve lift with 4000 gallons of RCS inventory spilled in the pipe tunnel. Enforcement panel is yet to be scheduled.

G. **Allegation Status**

H. **Performance Indicators (See Attached)**

The previous six months has shown an increase in personnel errors involving, the failure to follow procedures, inattention to detail, and the failure to maintain awareness of equipment status. The above occurrences and events that have taken place since July 1995, indicated a decrease in over all plant performance.

I. **PPR Results**

1. Trends - The recent six months has shown an increase in personnel errors involving the failure to follow procedures, inattention to detail, failure to maintain awareness of equipment status, and inadequate procedures. Operations performance overall has declined. Operators respond well to events but do not always have a questioning attitude, appear to have lapses in procedural use and compliance, and are not identifying concerns and holding plant support organizations accountable to correct plant deficiencies. The PPR panel agreed that all disciplines should review procedure adequacy and management oversight as part of each inspection. NRC performed an Organizational and Programmatic ~~Diagnostic~~ assessment of St Lucie and presented it to the licensee on August 29, 1995, identifying lack of detail and scope in procedures as the primary root cause with contribution from lack of management commitment to program implementation. The licensee performed an internal and independent external assessment and is implementing a corrective action program. The PPR panel agreed that the resident staff should continue to focus on the licensee's corrective action program (CAP). Since most of the licensee's CAP is to be completed by the end of 1995, DRS will conduct two weeks of regional initiative inspection effort to review the CAP by January, 1996.

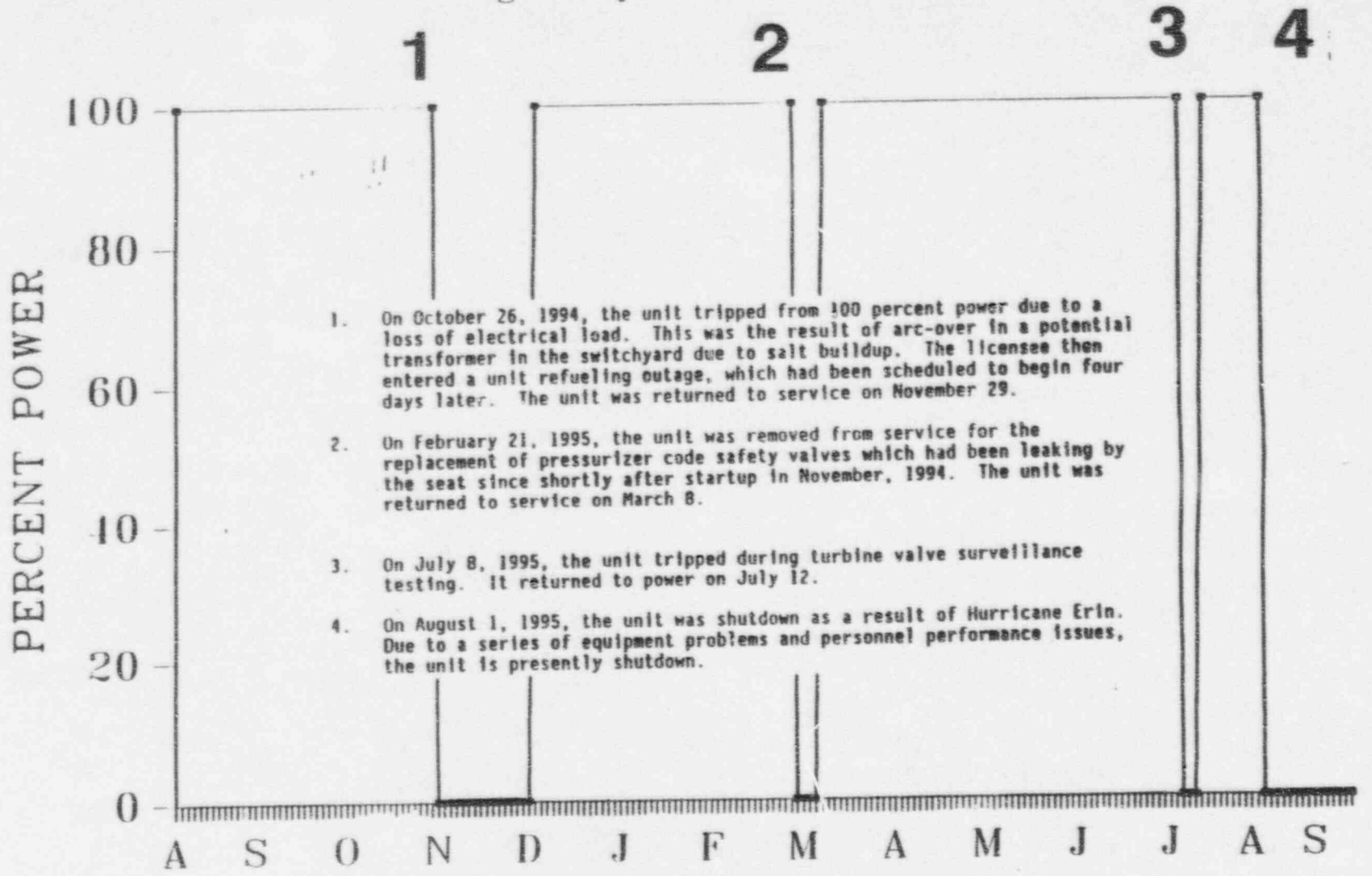
2. Planned Inspections (See Attached Letter to Licensee)
3. Site Integration Matrix (See Attached)

J. **Organizational Charts (See Attached)**

FAC	ACTIVITY ID	NAME	RESP ORG	LEAD EMP	TM SZ	ACTIVITY TARGT DT	09 OCT	16	23	30	06 NOV	13	20	27	04 DEC	11	18	25
STL1	Z008	INPO OUTAGE MANAGEME	2201	KDL	9	10/09/95	*****	*****										
STL1	A601	REC INSPECTION	2340	CQR	1	10/16/95		*****										
STL1	S000	END CYCLE 11			0	01/06/96												
STL1	A754	EP EXERCISE	2340	KFB	1	02/05/96												
STL1	A800	FRP CORE (83750)	2340	WGR	1	02/26/96												
STL1	A837	REC INSPECTION	2340	CQR	1	02/26/96												
STL1	A400	INITIAL PREP	2330	GOH	3	03/11/96												
STL1	A401	INITIAL EXAM	2330	GOH	3	03/25/96												
STL1	A801	FRP CORE (83750)	2340	WGR	1	06/17/96												
STL1	A900	SFGDS INSP	2340		1	07/22/96												

ST. LUCIE 1

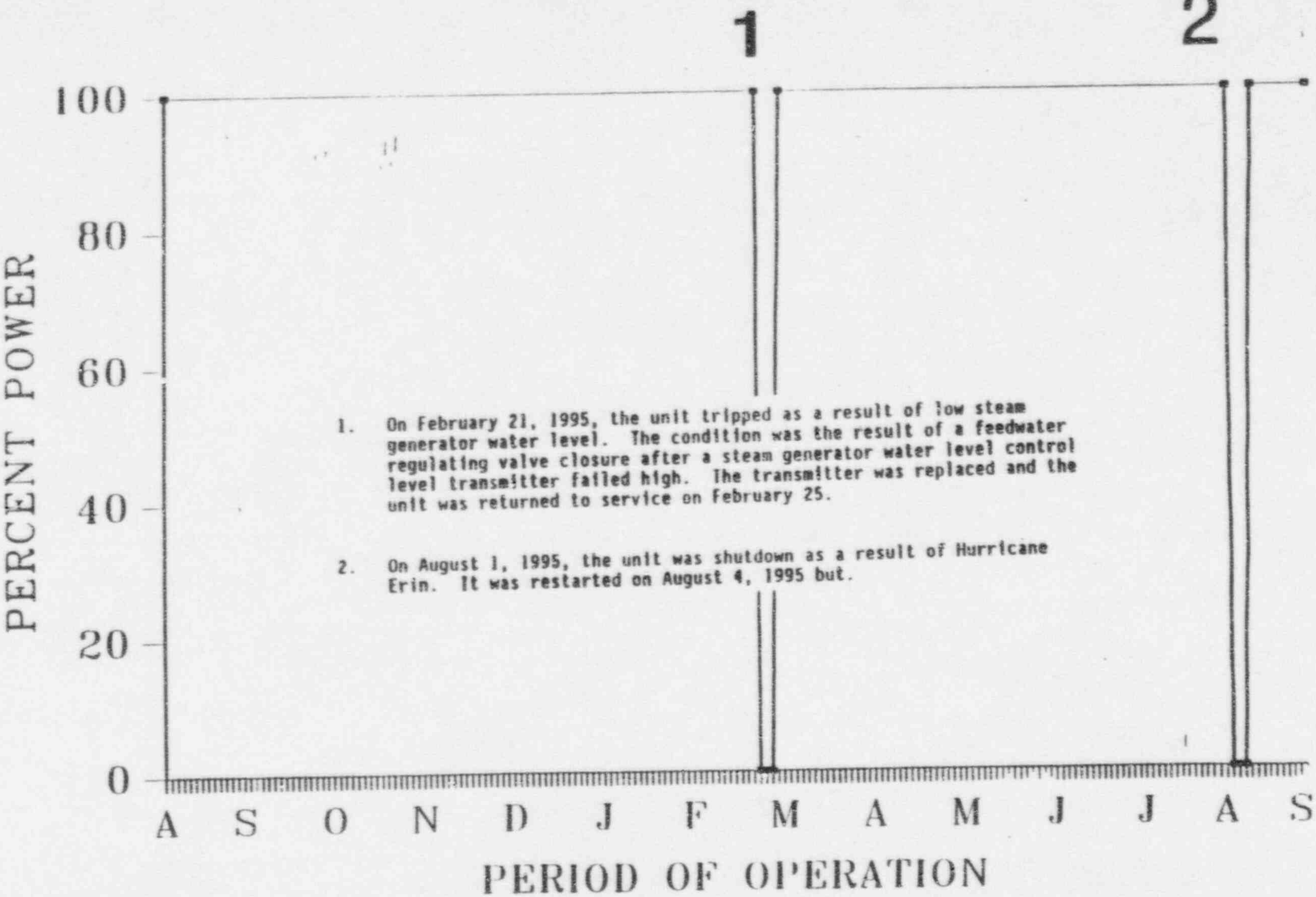
Operational Period August 1, Through September 12, 1995



Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

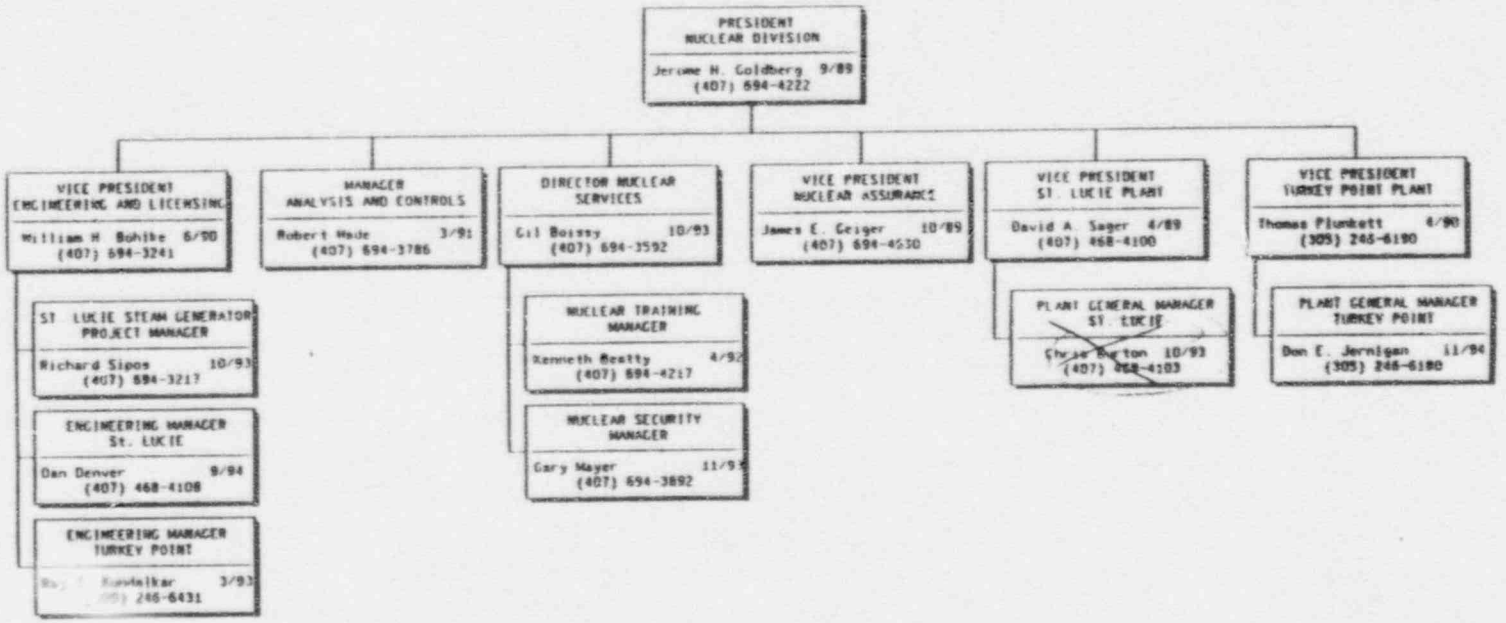
ST. LUCIE 2

Operational Period August 1, Through September 12, 1995



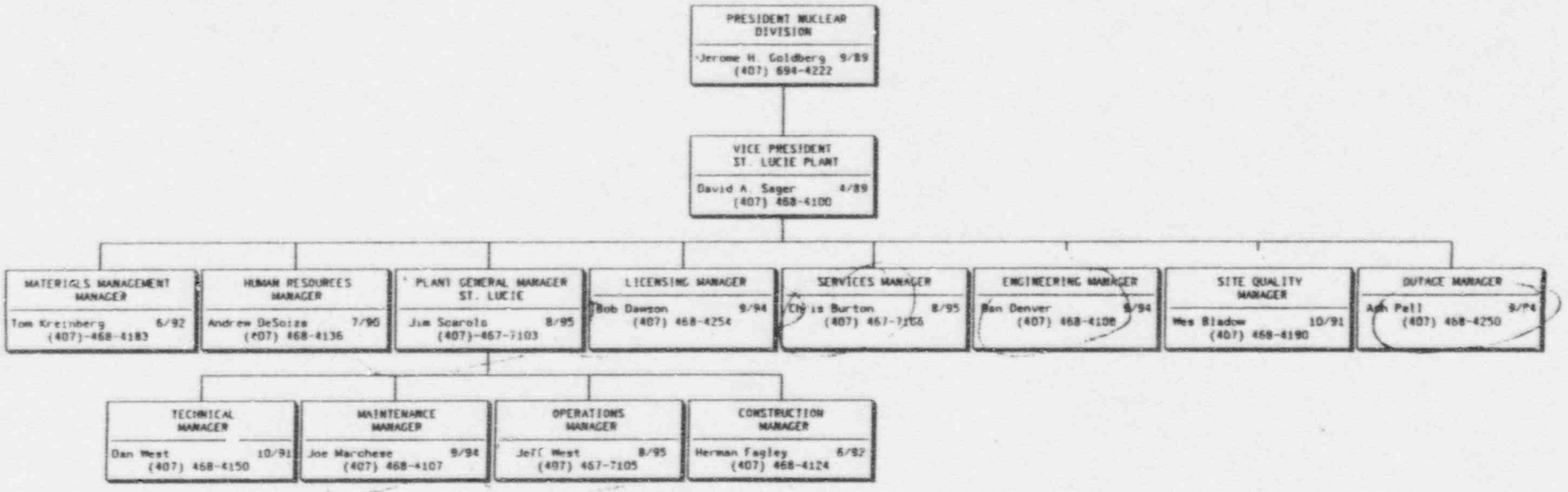
Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

FLORIDA POWER AND LIGHT
MANAGEMENT OVERVIEW



5/31/95

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE UNITS 1 & 2



10/10/95

October 6, 1995

ST LUCIE

Site Integration Matrix

Date	Salp F.A.	Ref.	Cause	Identified	Description
9/28/95	MS	IR 95-18	Equipment Failure	Self Identifying	Leaking PZR SVs extended forced outage - problems with tailpipe alignment.
9/20/95	MS	IR 95-18	Equipment Failure	Self Identifying	Grounds in EDG 1A/1B governor control wiring resulted in load oscillations.
9/15/95	OPS/ MS	IR 95-18	Failure to Follow Procedures	Self Identifying	Maint/Ops did not provide clearance for work on condenser waterbox cover. Vacuum severed worker's finger.
9/14/95	PS	IR 95-18	Failure to Follow Procedure	Licensee	Security failed to take correct compensatory action on computer failure.
9/10/95	OPS	IR 95-18	Failure to Use Correct Procedure	Self Identifying	SG blowdown sent to incorrect system on RAB roof. Operator used wrong procedure. When identified did not back out of procedure correctly.
9/9/95	MS	IR 95-15	Weakness in Work Screening and Planning	Self Identifying	Leak on SV 1201 flange extended outage, identified one month earlier but not worked.
9/7/95	OPS	IR 95-15	Personnel Error/ Inoperable Equipment/OWA	Licensee	Unit 2 Main Generator overpressurized while filling with H2. Inattention by operators.
9/2/95	OPS	IR 95-15 VIO 95-15	Personnel Error	NRC	Weaknesses identified in logs relating to abnormal equipment conditions and out of service equipment not logged (multiple examples).
8/31/95	MS	IR 95-15	Personnel Error	Self Identifying	Damaged cylinder and head on 1B EDG due to loose lash adjustment.
8/30/95	PS	IR 95-15	Management and QC weaknesses	NRC	Containment closure walkdowns by management were inadequate and depended heavily on QC involvement to identify deficiencies.
8/30/95	MS	IR 95-15	Supervisory oversight and worker attitude	NRC	Maintenance personnel not using procedures for work in progress.
8/29/95	OPS	IR 95-15 VIO 95-15	Personnel Error	Licensee	Started 1B LPSI pump with suction valve closed. (No damage to pump)
8/29/95	MS	IR 95-15	Procedure Use	NRC	Maintenance journeyman not signing off procedure steps as work completed (previously identified as a weakness in May 1995).
8/23/95	MS	IR 95-15	Equipment Failure/ Inadequate Corrective Action	Self Identifying	2A HDP trip due to relay failure. Eight HDP trips in past year. Engineering solution available but not implemented.
8/22/95	PS	IR 95-15	Personnel Error	NRC	QA failed to document a deficiency on containment spray valve surveillance identified in an audit.
8/19/95	OPS	IR 95-15	Operator Error/ Operator Workaround	Self Identifying	verfill of PWT. Spilled approx. 10K gallons on ground inside RCA. Operator work around on level control system and inattention to filling process by operator caused error.

8/18/95	MS	IR 95-15	Procedural Weakness	NRC	Procedural weakness involving supervisory oversight and journeyman qualification.
8/17/95	OPS	LER U1 95-007 VIO 95-15	Procedural Inadequacy and Weakness/ Operator-Work-Around	Self Identifying	Spraydown of Unit 1 containment. STAR process did not assign accountability for corrective action. Valve surveillance prelube not documented on STAR.
8/9/95	MS	LER U1 95-005	Maintenance/ Testing errors	Licensee	Inoperable Unit 1 PORVs due to maintenance error/testing inadequacy. (Valves assembled incorrectly) (Used acoustic data only)
8/6/95	ENG	LER U1 95-006 VIO 95-15	Corrective Action/Procedural Weakness	Self Identifying	Lifting of Unit 1 SDC thermal relief due to procedural revision from previous corrective action. Inoperable equipment not logged.
8/2/95	OPS	LER U1 95-004 VIO 95-15	Procedural Weakness/Failure to Follow Procedures	Licensee	1A2 RCP seal failure due to "restaging" at high temperature.
8/2/95	OPS	LER U1 95-04 VIO 95-15	Operator Error	Self-Identifying	Operator failed to block MSIS actuation during cooldown.
7/29/95	MS	IR 95-14	Prodcedural Weakness	Self Identifying	I&C personnel attempt to test a level switch circuit h could not actuate given system conditions.
7/29/95	OPS	IR 95-14	Operator Error/Procedural Weakness	Self Identifying	.oine/Reactor Trip due to test error.
7/29/95	MS	IR 94-14	Root Cause Pending	Self Identifying	Catastrophic failure of Unit 2 B train CEDM cooling fan.
7/3/95	PS	IR 95-14	Security Weakness	Self Identifying	Automobile passed through normally closed security gate to plant intake/discharge canals at beach. Subsequent accident resulted in vehicle lodged in discharge canal piping.
7/1/95	OPS	IR 95-12	Weak Log Keeping	NRC	Weaknesses identified in logs relating to battery jumper installation and out-of-service equipment.
7/1/95	MS	IR 95-12	Maintenance	Self Identifying	Corrosion in transformer fire protection deluge system results in multiple failures.
7/1/95	PS	IR 95-12 NCV 95-12-02	Personnel Error	NRC	Three pieces of SNM found improperly tagged.
7/1/95	PS	IR 95-12	Program Weaknesses	NRC	Fire Protection program weaknesses identified in fire-fighting techniques and respirator qualification program.
7/1/95	MS	IR 95-12 NCV 95-12-01	Personnel Error	NRC	M&TE found installed across battery cell without J/LL authorization.
6/3/95	MS	IR 95-10	Procedural Adequacy/ Adherance	NRC	Several examples of weak adherance to procedures, including step signoffs and independent verification, identified.
6/3/95	MS	IR 95-10	Poor Communication	Licensee	Poor communication/lack of detailed instruction leads to improper 1B EDG governor installation.
6/3/95	MS	IR 95-10	Poor Maintenance/Procedures	NRC	HVAC systems for both units poorly maintained/Operating procedures contained numerous deficiencies.
6/3/95	MS	IR 95-10 NCV 95-10-01	Poor Surveillance Tracking System	Licensee	Missed several surveillances (7 day) on EDG.
4/29/95	MS	IR 95-09 NCV 95-09-01	Personnel Error	Licensee	Failure to perform personnel air lock testing on time.

4/28/95	OPS	IR 95-05	Corrective Action Program Weakness	NRC	STAR/NCR program did not address evaluating past operability
4/28/95	MS	IR 95-05	Maintenance Error	Licensee	Incore Instruments at ICI Flange 8 miswired - ICI output signals directed to wrong computer points.
4/28/95	OPS	IR 95-05	Weakness in Temp Mod Procedure	NRC	Weakness in addressing how mods would affect control room drawings.
4/28/95	ENG	IR 95-05 NCV 95-05-04	Failure to Implement Corrective Action Program	NRC	Failure to document nonconformance regarding ICI flange 8 conditions.
4/28/95	MS	IR 95-05 VIO 95-05-01	Design Implementation Discrepancy	NRC	Installation of wrong overload heater models in switchgear.
4/1/95	OPS	IR 95-07 NCV 95-07-02	Apparent Personnel Error	Licensee	Unit 1 experienced an approximate 14 minute loss of shutdown cooling while shifting from one shutdown cooling loop to the other. The root cause was the closing of the wrong SDC suction isolation valve (the valve for the operating, vice idle, pump) on the part of the operator.
4/1/95	MS	IR 95-07 NCV 95-07-02	Poor Adherence to J/LL and Maintenance Procedures	Licensee	Jumper left installed in ECCS ventilation damper after work complete.
4/1/95	OPS	IR 95-07	Weak Annunciator Response	NRC	Weak annunciator response by ROs contributed to loss of shutdown cooling event.
3/26/95	MS	IR 94-09	Procedural Weakness	NRC	LPSI mechanical seal housing outer cap misinstalled.
3/26/95	OPS	IR 94-09	Operator Error/Proced-ural Weakness	NRC	Operator failure to recognize out-of-sight high indication on EDG cooling water tank. Failure of procedure to include instructions on draining tank.
3/04/95	ENG	IR 95-04	Design	Licensee	SDC suction relief valve lift due to water hammer.
3/04/95	OPS	IR 95-04	House-keeping	NRC	Loose plastic debris found in Unit 2 fuel pool area.
2/27/95	MS	IR 95-04	Equipment Failure	Self Identifying	Unit 1 was shut down for the replacement of 3 pressurizer code safety valves. The valves were leaking by the seat.
2/21/95	OPS	IR 95-04	Equipment Failure	Self Identifying	Unit 2 trip due to failure of a SGWL control level transmitter. Transmitter failed high, resulting in closure of the FRV and a subsequent trip on low SGWL. (95-04)
2/20/95	OPS	IR 95-04	Equipment Anomaly	Self Identifying	2B LPSI pump found air-bound during surveillance testing. The licensee has theorized that the migration of air in the system resulted in the condition as a result of previous surveillance testing. The pumps are not self-venting.
2/17/95	MS	IR 95-02	Physical Condition	NRC	Numerous areas of corrosion identified in Unit 1/2 CCW areas.
2/17/95	PS	IR 95-03	Personnel Error /Training Weakness	NRC	In two observed exercises, ECs failed to notify states within 15 minutes.
2/16/95	MS	IR 95-04	Maintenance Error/ Procedural Weakness	Self Identifying	Load shed of the 1A3 1E 4160 bus due to inadvertent jumper contact while replacing a degraded voltage relay.

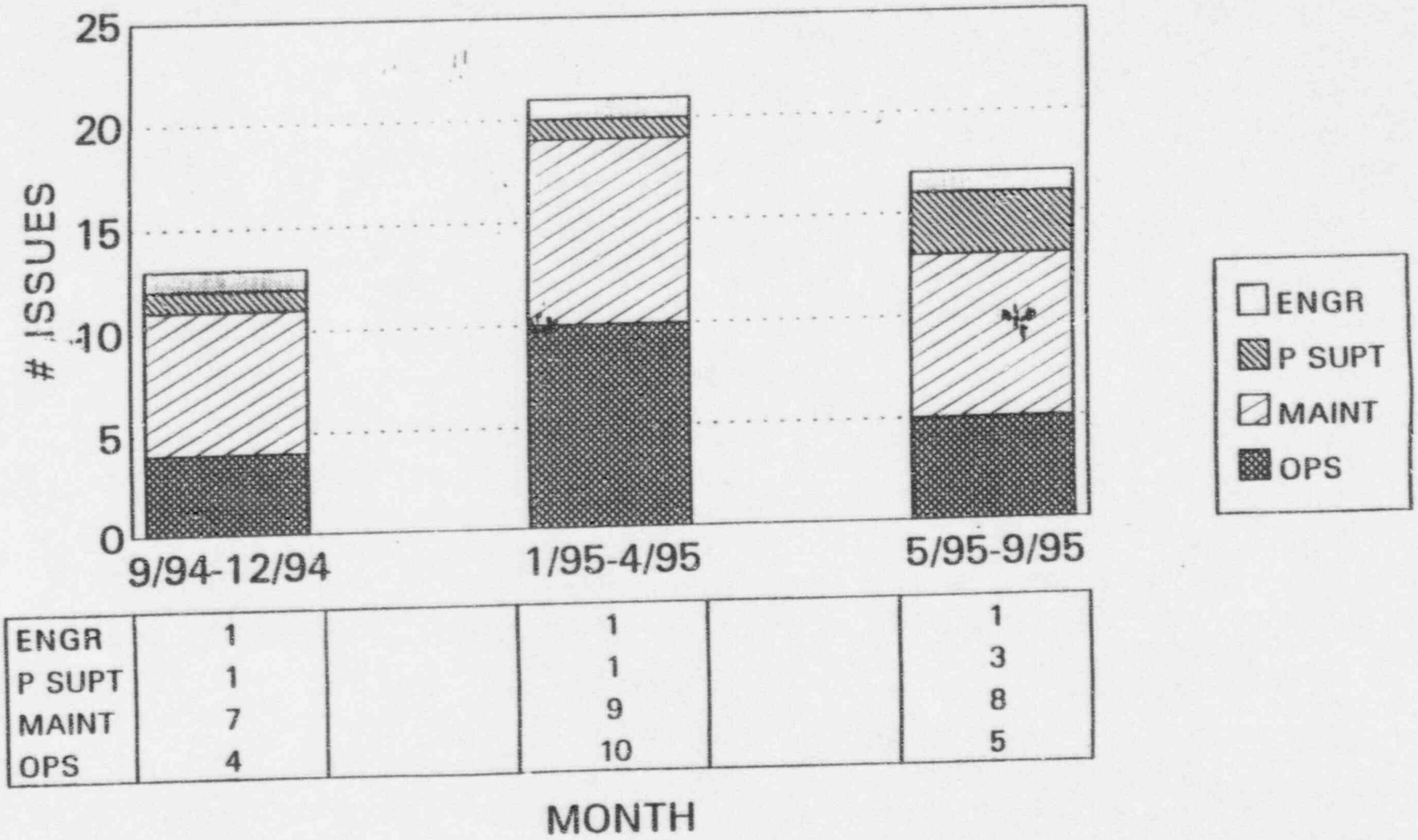
2/4/95	OPS	IR 95-01 VIO 95-01-01	Operator Error/Communications	Licensee	Failure to sample SIT within TS required time frame following volume addition. Second occurrence in 2 years.
2/4/95	OPS	IR 95-01	Poor Communications	NRC	Failure to identify and analyze Unit 1 hot leg flow stratification
2/4/95	MS	IR 95-01 VIO 95-01-02	Personnel Error/ Program Weakness	Self Identifying	Inadequate independent verification resulted in CVCS letdown control valve failing to respond due to reversed leads. Resulted in a cessation of letdown flow.
12/31/94	ENG	IR 94-25 NCV 94-25-01	Engineering Design Error	Self Identifying	Inadequate design control of NaOH cross-connection between ECCS trains.
12/3/94	PS	IR 94-24 NCV 94-24-01	Procedure Review Inadequacy	Licensee	Failure to perform TS-required periodic procedure reviews.
12/3/94	MS	IR 94-24 VIO 94-24-02	Maintenance Procedures Inadequacy	NRC	Inadequate process for changes to vendor technical manuals.
11/25/94	MS	IR 94-22	Program weakness	Licensee	The licensee's QA organization identified numerous weaknesses in the implementation of the site's welding program. As a result, the Maintenance Manager placed a stop work order on welding activities. The stoppage lasted one week.
11/24/94	MS	IR 94-24	Procedure weakness	Self-Identifying	Unit 1 B side SIAS actuation due to a bistable module which had not been adequately withdrawn from the ESFAS cabinet during maintenance.
11/23/94	MS	IR 94-24	Equipment Failure	Self Identifying	Unit 1 SIAS with unit in mode 5 due to common mode failure of Rosemount transmitters used for pressurizer pressure channels.
11/5/94	OPS	IR 94-22 NCV 94-22-03	Operations, Maintenance Errors	Licensee	Waste gas release on Sept. 10, 1993, with meteorological instruments out of service.
10/26/94	MS	IR 94-22 LER	Weather-Related/ Maintenance	Self-Identifying	Unit 1 automatically tripped due to arc-over from a potential transformer due to salt buildup on switchyard insulators.
9/30/94	OPS MS	IR 94-20	Inconsistent Expectations	NRC	Local valve position indicators not maintained accurate. Procedures/training provided to operators on verifying valve position found weak.
9/30/94	OPS	IR 94-20	Operations, Maintenance Deficiency	NRC	Plant personnel not trained on IPE and not using it for work planning and scheduling.
9/30/94	OPS	IR 94-19	Operations Weakness	NRC	During requal exam, a licensed operator exhibited an apparent disregard for EOPs.
9/30/94	MS	IR 94-20	Personnel Error	Licensee	Maintenance personnel begin to work the wrong RWT isolation valve, threatening the operability of both trains of ECCS.
9/30/94	OPS	IR 94-19 NCV 94-19-01	Operations Error	Licensee	Failure to notify the NRC of changes in status of licensed operators' medical conditions.
8/29/94	OPS	IR 94-20 VIO 94-22-01 VIO 94-22-02	Operations Errors	NRC	Operators placed 1A EDG in an electrical lineup for which TS-required surveillance tests had not been performed (with the safety-related swing bus powered from it). Also, related control room log entries appeared to be inaccurate.
8/28/94	OPS	IR 94-20	Equipment Failure	Licensee	Unit 1 was taken off line (Mode 2) to repair a DEH leak. The unit was returned on line later the same day.

8/12/94	OPS	IR 94-18	Operations/ Maintenance Error and Lack of Engineering Drawings/In-spection Criteria	NRC	The licensee was unloading new fuel for Unit 1 with a hoist grapple that was missing the safety latch sleeve locating pin. The safety sleeve functioned by friction only.
7/14/94	MS	IR 94-15 LER U-2 94-06 VIO 94-15-01	Equipment Failure/Poor Management Decision	Licensee/NRC	During surveillance test, TCB 5 failed to open due to mechanical binding (licensee). The licensee failed to recognize the condition as requiring a shutdown per TS (NRC).
7/9/94	OPS	IR 94-15	Equipment Failure	Licensee	Unit 2 turbine was shut down and reactor power reduced to Mode 2 because the 2B1 RCP lower oil level indication showed a leak. The indication was later shown to be erroneous.
7/8/94	OPS	IR 94-15 LER U2 94-05	Operator Error	Licensee	TS 3.0.3 entry due to placing 2A1 LPSI pump and 2B charging pump OOS at the same time.
6/28/94	MS	IR 94-14 NCV 94-14-01 LER U-2 94-04	Personnel Error/ Procedural Weakness	Licensee	Inoperable Unit 2 RAB ventilation exhaust WRGM due to failure to connect sample lines.
6/6/94	OPS	IR 94-14	Weather	Licensee	Unit 1 trip from 100% power during a severe thunderstorm due to debris blown across two main transformer output terminals.
5/28/94	PS	IR 94-13 DEV 94-13-01	Poor Corrective Action	NRC	Emergency supplies in control room less that stated in FSAR.
5/6/94	ENG	IR 94-11 VIO 94-11-01	Engineering Error	NRC	Inadequate corrective action for MOVs which stalled during surveillances.
4/23/94	OPS	IR 94-12 LER U-2 94-03	Mfg. Error	Self Identifying	Unit 2 auto reactor trip from 30% power caused by RPS cabinet wiring error for trip bypass circuit, from original unit construction.
4/23/94	MS	IR 94-12	Equipment Failure	Self-Identifying	Following unit 2 trip, steam bypass system operated unexpectedly and dropped RCS temp by seven degrees F, pressurizer heaters turned off.
4/21/94	OPS	IR 94-12	Operator Inattentiveness	Licensee	Unit 2 reactor power increased from 26 to 31% due to positive MTC.
4/7/94	MS	IR 94-10 VIO 94-10-01	Maintenance Error	NRC	Contractor personnel made and contractor QC accepted pressurizer nozzle weld prep that did not meet procedural requirements for bevel angle. Licensee engineering had specified overly tight tolerances.
4/3/94	OPS	IR 94-12 LER U1 94-04	Operations Procedure Error (Lack of sufficient depth in review)	Self-Identifying	Unit 1 auto reactor trip due to unusual electrical lineup (isochronous EDG paralleled with offsite power through TCBs).
4/3/94	ENG	IR 94-12 VIO 94-12-01	Surveillance Error	Licensee	Licensee discovered that the 4160 V (AB Bus) swing bus components (C ICW Pump and C CCW Pump) would not strip from the bus upon undervoltage if the bus were aligned to the B bus due to a missing wire.
3/28/94	MS	IR 94-09 LER U1 94-03	Personnel Error	Self Identifying	Unit 1 auto reactor trip. Maintenance foreman opened generator exciter breaker on wrong unit.
3/16/94	ENG	IR 94-08 VIO 94-08-01 VIO 94-08-02	Engineering Corrective Action	NRC	Regional inspector had two Unit 2 SL4 violations: 1) corrective action for an 11/24/92 water hammer event was done without documented instructions or procedures, resulting in operating until 3/94 with five snubbers on the SRV and PORV tailpipes inoperable. 2) Failure to write a nonconformance report for a damaged pipe support in March 1994.

3/16/94	ENG	IR 94-10 LER U-294-02	Equipment Failure	Licensee	A Unit 2 pressurizer instrument nozzle that had been repaired a year ago was found leaking while the unit was in Mode 5. The unit remained shut down for repairs.
3/4/94	ENG	IR 94-06 NCV-94-06-02	Engineering Design Error	Licensee	Inadequate design controls on Unit 2 sequencer charging pump loading block.
3/4/94	ENG	IR 94-06 NCV 94-06-01	Engineering Error	Licensee	Failure to report an EDG failure.
2/28/94	ENG	IR 94-09 NCV 94-04-01	Refueling procedure & operator error	Licensee/NRC	Inadequate grappling of a fuel assembly caused by error in Recommended Move List and operator error in following procedure. (IR 94-09)
2/17/94	OPS	IR 94-05 NCV 94-05-01 LER U2 94-01	Operator Error	Licensee	Pressurizer aux. spray isolation valve had been locked closed (vice open) since 3/27/93.
2/11/94	PS	IR 94-02 NCV 94-02-01	Security Error	Licensee	Failure to provide required compensatory measures in response to a security computer system failure.
1/13/94	OPS	IR 94-01 LER U1 94-02	Surveillance Procedure Weakness/ Component Faliure	Self Identified	UV relay test resulted in load shed of the 1A3 4160 volt bus and a 1A EDG auto start
1/9/94	OPS	IR 94-01 LER U1 94-01	Equipment Failure	Self Identified	Manual reactor trip - feed pump control circuit failure.
1/2/94					SALP period 11 began

ST LUCIE

of ISSUES vs TIME



ENGR	1	1	1
P SUPT	1	1	3
MAINT	7	9	8
OPS	4	10	5

9/94 TO 9/95

August 3, 1995

IMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE PNO-II-95-047

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by Region II staff (Atlanta, Georgia) on this date.

Facility
Florida Power & Light Co.
Saint Lucie 1
Hutchinson Island, Florida
Dockets: 50-335

Licensee Emergency Classification
X Notification of Unusual Event
Alert
Site Area Emergency
General Emergency
Not Applicable

Subject: STARTUP OF ST. LUCIE UNIT 1 POSTPONED TO REPAIR PUMP SEAL

On August 2, 1995, at 9:25 p.m. (EDT) St. Lucie Unit 1 declared an Unusual Event due to a 2 GPM leak from the 1A2 reactor coolant pump seal to the containment sump. (See Event Number 29153.) Unit 1 is being cooled down to Mode 5 and depressurized for repairs. At the time of this event, Unit 1 was shut down in Mode 3 preparing for restart after Hurricane Erin had passed. The licensee estimates the outage duration to be about seven days. The NRC resident inspectors are following the event.

State of Florida has been notified of this event.

The NRC received notification of this event at 9:58 p.m. (EDT) on August 2 by phone call to the NRC duty officer.

This information is current as of 9:30 a.m. on August 3, 1995.

Contact: R. SCHIN
(404)331-5561

IMINARY NOTIFICATION OF EVENT OR UNUSUAL OCCURRENCE PNO-II-95-039

This preliminary notification constitutes EARLY notice of events of POSSIBLE safety or public interest significance. The information is as initially received without verification or evaluation, and is basically all that is known by Region II staff (Atlanta, Georgia) on this date.

Facility
Florida Power & Light Co.
Saint Lucie 1 2
Jensen Beach,Florida
Dockets: 50-335,50-389

Licensee Emergency Classification
Notification of Unusual Event
Alert
Site Area Emergency
General Emergency
X Not Applicable

Subject: MEDIA INTEREST - AUTOMOBILE DRIVES INTO DISCHARGE CANAL

At about 3:30 p.m. on Sunday, July 9, 1995, a sport utility vehicle drove through the owner controlled area and into the discharge canal near the beach head wall. The three vehicle occupants safely exited the vehicle and no one was seriously injured. The vehicle is now completely submerged. The vehicle occupants were teenagers and were apparently looking for a place to surf.

The vehicle entered the discarge canal on the opposite side of the A1A highway from the nuclear units and about one-half mile from the nuclear s. The area was fenced and posted for restricted access but an ss gate was open. There was no threat to plant safety. There has been local media interest.

The licensee notified the U.S. Coast Guard, Florida Marine Patrol, and the National Department of Transportation Response Center.

The NRC Senior Resident Inspector received initial notification of this event from the licensee at about 3:30 p.m. July 9, and responded to the site. The NRC residents are following this issue.

The State of Florida has been notified.

This information is current as of 9:30 a.m. on July 10, 1995.

Contact: R. SCHIN
(404)331-5561

UNITED STATES
NUCLEAR REGULATORY COMMISSION
REGION II
101 MARIETTA STREET, N.W., SUITE 2900
ATLANTA, GEORGIA 30323-0199

October 10, 1995

Florida Power and Light Company
ATTN: Mr. J. H. Goldberg
President - Nuclear Division
P. O. Box 14000
Juno Beach, FL 33408-0420

SUBJECT: PLANT PERFORMANCE REVIEW (PPR) - ST. LUCIE

Dear Mr. Goldberg:

On September 21, 1995, the NRC staff completed the semiannual Plant Performance Review (PPR) of St. Lucie. The staff conducts these reviews for all operating nuclear power plants to develop an integrated understanding of safety performance. The results are used by regional management to facilitate planning and allocation of inspection resources. The PPR for St. Lucie involved the participation of all Regional Technical Divisions and NRR in evaluating inspection results and safety performance information for the period from March 1995 to September 1995. PPRs provide regional management with a current summary of licensee performance and serve as inputs to the NRC Systematic Assessment of Licensee Performance (SALP) and senior management meeting (SMM) reviews.

This letter advises you of our planned inspection effort resulting from the St. Lucie PPR review. It is provided to minimize the resource impact on your staff and to allow for scheduling conflicts and personnel availability issues to be resolved in advance of inspector arrival onsite. The enclosure details our inspection plan for the next 6 months. The rationale or basis for each inspection outside the core inspection program is provided so that you are aware of the reason for emphasis in these program areas. Resident inspections are not listed due to their ongoing and continuous nature.

We will inform you of any changes to the inspection plan. If you have any questions, please contact K. Landis at (404) 331-5509.

Sincerely,

Ellis W. Merschoff, Director
Division of Reactor Projects

Docket No.: 50-335, 50-389
License No.: DPR-67, NPF-16

Enclosure: Inspection Plan

cc w/encl: (See page 2)

9510230304 SPP

cc w/encl:
D. A. Sager
Vice President
St. Lucie Nuclear Plant
P. O. Box 128
Ft. Pierce, FL 34954-0128

H. N. Paduano, Manager
Licensing and Special Programs
Florida Power and Light Company
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Juno Beach, FL 33408-0420

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Department of Health and
Rehabilitative Services
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Jack Shreve
Public Counsel
Office of the Public Counsel
c/o The Florida Legislature
111 West Madison Avenue, Room 812
Tallahassee, FL 32399-1400

cc w/encl: Continued see page 3

cc w/encl: Continued
Joe Myers, Director
Division of Emergency Preparedness
Department of Community Affairs
2740 Centerview Drive
Tallahassee, FL 32399-2100

Thomas R. L. Kindred
County Administrator
St. Lucie County
2300 Virginia Avenue
Ft. Pierce, FL 34982

Charles B. Brinkman
Washington Nuclear Operations
ABB Combustion Engineering, Inc.
12300 Twinbrook Parkway, Suite 3300
Rockville, MD 20852

ST LUCIE
INSPECTION PLAN

IP - Inspection Procedure
TI - Temporary Instruction
Core Inspection - Minimum NRC Inspection Program (mandatory all plants)

INSPECTION PROCEDURE	TITLE/ PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 84750 IP 86750	Radioactive Waste Systems, Water Chemistry Confirmatory Measurements and Radiological Environmental Monitoring Solid Radioactive Waste Management and Transportation of Radioactive Materials	1	10/16-20/95	Core Inspection
IP 38701 IP 38702 IP 38703	Procurement Program; Receipt, Storage, and Handling of Equipment and Materials Program Commercial Grade Procurement Inspection	1	12/04-15/95	Regional Initiative - to address recent concerns in the procurement area. Jim Moorehead
IP 40500 IP 92720	Safety Assessment Corrective Action Program	3	12/95 or 01/96 1-week	Regional Initiative Inspection to review the adequacy and status of the ongoing Safety Assessment and Corrective Action Program at St Lucie.

INSPECTION	TITLE/ PROGRAM AREA	NUMBER OF INSPECTORS	PLANNED INSPECTION DATES	TYPE OF INSPECTION - COMMENTS
IP 61726 IP 62703	Surveillance Observation Maintenance Observation	1	11/95 1-week, 01/96 1-week, 03/96 1-week	Core Inspection
IP 82701 IP 82301 IP 82302	Operational Status of the Emergency Preparedness Program Evaluation of Exercises for Power Reactors Review of Exercise Objectives and Scenarios for Power Reactors	2	02/05-09/96	Core Inspection - EP Exercise
IP 83750 IP 84750 IP 86750	Occupational Radiation Exposure Radioactive Waste Systems, Water Chemistry Confirmatory Measurements and Radiological Environmental Monitoring Solid Radioactive Waste Management and Transportation of Radioactive Materials	2	02/26-03/01/96	Core Inspection
TI 2515/109	Inspection Requirements for Generic Letter 89-10, Safety-Related Motor-Operated Valve Testing and Surveillance	1 or 2	TBD - 01-03/96	Area of Emphasis - Inspection to close out Generic Letter 89-10
	Licensing Examination Preparation	3	03/11-15/96	Operator Exam Preparation
	Initial Licensing Examination	3	03/25-29/96	Scheduled Operator Exam

DATE: 12/04/96
TIME: 00/13/07

FLORIDA POWER & LIGHT

PLANT: PSL
RPTID: PMPRC30R

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

ACCESS RESTRICTED REPORT - ACTIVE PERSONNEL

DEPT	PID #	NAME	CODE	REASON DESCRIPTION	OPID	EFFECTIVE DATE	END DATE
017	[redacted]		HP1	SEE UNIT 1 HEALTH PHYSICS SHIFT SUPERVISOR	MOURING	09/04/96	
041			TRN	NEEDS TRAINING	DUNNE	12/03/96	
044			WBC	NEEDS BODY COUNT	DUNNE	12/03/96	
044			MED	MEDICAL	DUNNE	11/13/96	
013			WBC	NEEDS BODY COUNT	DUNNE	12/03/96	
017			MED	MEDICAL	VASILAS	09/12/96	
046			TRN	NEEDS TRAINING	DUNNE	12/03/96	
061			TRN	NEEDS TRAINING	DUNNE	12/03/96	
052			TRN	NEEDS TRAINING	DUNNE	12/03/96	
047			WBC	NEEDS BODY COUNT	DUNNE	12/03/96	
041			TRN	NEEDS TRAINING	DUNNE	12/03/96	
044			NTC	NEEDS TRAINING CLASS & BODY COUNT	VASILAS	10/02	
030			LOA	LEAVE OF ABSENCE	DUNNE	11/21	
013			NTC	NEEDS TRAINING CLASS & BODY COUNT	VASILAS	09/03/9[illegible]	
095			MED	MEDICAL	DUNNE	10/31/96	
044			NTC	NEEDS TRAINING CLASS & BODY COUNT	VASILAS	10/02/96	
014			HPA	SEE DOSIMETRY	MOORE	11/20/96	
039			TRN	NEEDS TRAINING	DUNNE	12/03/96	
083			MED	MEDICAL	MOORE	11/26/96	

5/8/12

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 004 - NPS ENERGY SERVICES

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			984	16	16	1000	4500	16	0	0	0.00
			997	3	3	1000	4500	3	0	0	0.00
			986	14	14	1000	4500	14	0	0	0.00
			912	88	88	1000	4500	88	0	0	0.00
			954	46	46	1000	4500	89	0	0	0.00
			698	302	302	1000	4500	340	0	0	0.00
			947	53	53	1000	4000	53	0	0	0.00
			988	12	12	1000	4500	12	0	0	0.00
			963	37	37	1000	4500	37	0	0	0.00
	X		952	48	48	1000	4000	48	0	0	0.00
	XCG		470	530	530	1000	4500	530	0	0	0.00
			980	33	20	1000	4500	20	0	0	0.00
			981	19	19	1000	4500	19	0	0	0.00
	X		810	190	190	1000	4500	190	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			767	233	233	1000	4500	233	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			775	225	225	1000	4500	225	0	0	0.00
			727	273	273	1000	4500	291	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		960	40	40	1000	4500	40	0	0	0.00
	X		836	164	164	1000	4500	189	0	0	0.00
			869	131	131	1000	4500	131	0	0	0.00
			846	154	154	1000	4500	172	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			958	42	42	1000	4500	42	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		538	1212	1212	1750	4500	1369	0	0	0.00
			988	12	12	1000	4500	12	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		713	287	287	1000	4500	287	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			850	150	150	1000	4500	150	0	0	0.00
			992	25	8	1000	4500	8	0	0	0.00
			671	329	329	1000	4500	353	0	0	0.00
	X		936	64	64	1000	4500	64	0	0	0.00
			1000	0	83	1000	4500	83	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			868	132	132	1000	4500	132	0	0	0.00
			473	527	527	1000	4500	598	0	0	0.00
	X		622	878	878	1500	4500	938	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 004 - NPS ENERGY SERVICES

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			929	71	71	1000	4500	84	0	0	0.00
			945	55	134	1000	4500	134	0	0	0.00
			1000	0	15	1000	4500	15	0	0	0.00
	XG		914	86	86	1000	4500	86	0	0	0.00
			875	125	125	1000	4500	135	0	0	0.00
			719	281	281	1000	4500	281	0	0	0.00
			939	61	61	1000	4500	61	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		938	62	62	1000	4500	62	0	0	0.00
			988	12	12	1000	4500	12	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			937	63	150	1000	4500	150	0	0	0.00
	X		934	66	131	1000	4500	131	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 007 - COMBUSTION ENG

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	981	19	19	1000	4500	19	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT 010 - ELECTRICAL MAINT

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
	SX		921	79	79	1000	4500	79	0	0	0.00
	SX		763	237	237	1000	4500	248	0	0	0.00
	SX		978	22	22	1000	4500	22	0	0	0.00
	SX		997	3	3	1000	4500	3	0	0	0.00
	SXG		743	257	257	1000	4500	257	0	0	0.00
	SX		985	15	15	1000	4500	15	0	0	0.00
	SXG		976	24	24	1000	4500	24	0	0	0.00
	SX		747	253	253	1000	4500	253	0	0	0.00
	SX		910	90	90	1000	4500	93	0	0	0.00
	SX		908	92	92	1000	4500	92	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			923	77	77	1000	4500	77	0	0	0.00
	SX		903	97	97	1000	4500	100	0	0	0.00
			448	2	2	450	4500	2	0	0	0.00
			917	83	83	1000	4500	83	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
	SX		982	18	18	1000	4500	18	0	0	0.00
	SX		975	25	25	1000	4500	25	0	0	0.00
	SX		445	555	555	1000	4500	555	0	0	0.00
			935	65	65	1000	4500	65	0	0	0.00
			923	77	77	1000	4500	87	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			941	59	59	1000	4500	59	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		981	19	19	1000	4500	19	0	0	0.00
	Y		996	4	4	1000	4500	4	0	0	0.00
			792	208	208	1000	4500	226	0	0	0.00
	SX		957	43	43	1000	4500	43	0	0	0.00
	SX		998	2	2	1000	4500	2	0	0	0.00
	SXCG		748	252	252	1000	4500	272	0	0	0.00
	SX		968	32	32	1000	4500	32	0	0	0.00
	YG		385	615	615	1000	4500	638	0	0	0.00
	SX		577	423	423	1000	4500	443	0	0	0.00
	SX		957	43	43	1000	4500	43	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		861	139	139	1000	4500	139	0	0	0.00
	SX		952	48	48	1000	4500	48	0	0	0.00
	SXG		952	48	48	1000	4500	48	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
	SX		945	55	55	1000	4500	55	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 012 - HP UTILITY WORKERS

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	X	[redacted]	1215	285	285	1500	4500	302	0	0	0.00
	XG		550	950	950	1500	4500	974	0	0	0.00
	X		395	605	605	1000	4500	637	0	0	0.00
	X		457	543	543	1000	4500	543	0	0	0.00
			824	176	185	1000	4500	185	0	0	0.00
	SX		918	82	82	1000	4500	82	0	0	0.00
	X		848	152	152	1000	4500	152	0	0	0.00
	X		596	404	404	1000	4500	414	0	0	0.00
	X		884	616	616	1500	4500	631	0	0	0.00
	SX		885	115	115	1000	4500	115	0	0	0.00
	X		710	290	595	1000	4500	628	0	0	0.00
	X		658	342	342	1000	4500	354	0	0	0.00
			986	14	144	1000	4500	144	0	0	0.00

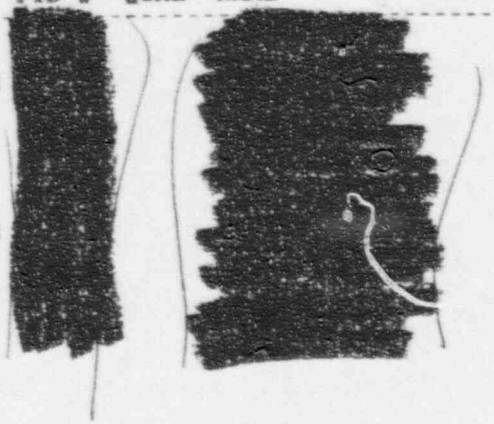
F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 013 - FPL - NON-PSL



PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
	R		1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	R		1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			330	120	120	450	4500	120	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 014 - HEALTH PHYSICS

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SX	[redacted]	402	598	598	1000	4500	618	0	0	0.00
	SX		562	438	438	1000	4500	450	0	0	0.00
	SX		354	646	646	1000	4500	707	0	0	0.00
	SX		708	292	292	1000	4500	307	0	0	0.00
			999	1	1	1000	4500	1	0	0	0.00
	SX		657	843	843	1500	4500	859	0	0	0.00
	SX		552	948	948	1500	4500	964	0	0	2.12
	SX		253	1247	1247	1500	4500	1311	0	0	1.00
	SX		970	30	30	1000	4500	30	0	0	0.00
	SXG		999	1	1	1000	4500	1	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		760	240	240	1000	4500	240	0	0	0.00
	SX		618	382	382	1000	4500	397	0	0	0.00
	SX		639	361	361	1000	4500	361	0	0	0.00
	SX		824	176	176	1000	4500	176	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		755	245	245	1000	4500	264	0	0	0.00
	SXG		571	929	929	1500	4500	944	0	0	0.00
	SX		547	453	453	1000	4500	453	0	0	0.00
	X		758	742	742	1500	4500	742	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		892	108	108	1000	4500	129	0	0	0.00
	SXG		913	87	87	1000	4500	97	0	0	0.00
	SX		841	159	159	1000	4500	163	0	0	0.00
	SX		495	1005	1005	1500	4500	1126	0	0	0.00
	SX		368	632	632	1000	4500	632	0	0	0.00
	SX		763	237	238	1000	4500	238	0	0	0.00
	SX		381	619	619	1000	4500	645	0	0	0.00
	SX		579	421	421	1000	4500	442	0	0	0.00
R	SX		664	336	336	1000	4500	336	0	0	0.00
	SX		1404	96	96	1500	4500	99	0	0	0.10
	SX		363	1131	1271	1500	4500	1306	0	0	0.00
	SX		700	300	300	1000	4500	300	0	0	0.50
	SXG		892	108	108	1000	4500	108	0	0	0.00
	SX		837	163	163	1000	4500	163	0	0	0.00
	SX		865	135	135	1000	4500	135	0	0	0.00
	SX		940	60	60	1000	4500	60	0	0	0.00
	SX		754	246	246	1000	4500	246	0	0	0.00
	K		450	0	0	450	4500	0	0	0	0.00
	SX		614	386	386	1000	4500	411	0	0	0.00
	SX		690	310	310	1000	4500	330	0	0	0.00
	SX		616	884	884	1500	4500	895	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 015 - HEALTH PHYSICS CONTR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	TEDE -- YEAR	- GUIDE/EXT - SITE	GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	X	[redacted]	976	24	24	1000	4500	24	0	0	0.00
	X		886	114	114	1000	4500	114	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

[redacted]ARTMENT: 017 - I & C

	PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM DAC HRS
	[redacted]	X		680	320	320	1000	4500	320	0	0	0.00
R		X		577	423	423	1000	4500	425	0	0	0.00
				852	148	148	1000	4500	148	0	0	0.00
		X		1000	0	0	1000	4500	0	0	0	0.00
		SX		701	399	399	1000	4500	419	0	0	0.00
				970	30	30	1000	4500	30	0	0	0.00
		SX		296	704	704	1000	4500	753	0	0	0.00
				571	429	429	1000	4500	429	0	0	0.00
				416	584	584	1000	4500	628	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
		X		1000	0	0	1000	4500	0	0	0	0.00
		SX		989	11	11	1000	4500	11	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
		Y		841	159	159	1000	4500	159	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
		SXG		577	423	423	1000	4500	433	0	0	0.00
		SX		789	211	211	1000	4500	211	0	0	0.00
				985	15	15	1000	4500	15	0	0	0.00
		X		1000	0	0	1000	4500	0	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
				669	331	331	1000	4500	331	0	0	0.00
		SXG		729	271	271	1000	4500	271	0	0	0.00
				976	24	198	1000	4500	198	0	0	0.00
		X		415	585	585	1000	4500	679	0	0	0.00
				686	314	314	1000	4500	326	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
R		SX		890	110	110	1000	4500	110	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
		YG		874	126	126	1000	4500	126	0	0	0.00
				861	139	139	1000	4500	139	0	0	0.75
		SX		1237	263	263	1500	4500	263	0	0	0.00
		SX		526	474	474	1000	4500	525	0	0	0.00
		X		687	313	313	1000	4500	327	0	0	0.00
		SX		1148	352	352	1500	4500	352	0	0	0.00
				914	86	86	1000	4500	86	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
				682	818	818	1500	4500	858	0	0	0.00
		X		581	919	919	1500	4500	972	0	0	0.00
		SX		986	14	14	1000	4500	14	0	0	0.00
				475	525	525	1000	4500	537	0	0	0.00
		SX		609	391	391	1000	4500	424	0	0	0.00
				654	346	346	1000	4500	356	0	0	0.00
				799	201	201	1000	4500	201	0	0	0.00
		SXG		694	306	306	1000	4500	328	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 017 - I & C

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	X	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		983	17	17	1000	4500	17	0	0	0.00
	SXCG		492	508	508	1000	4500	531	0	0	0.00
	SX		918	82	82	1000	4500	97	0	0	0.00
			960	40	40	1000	4500	40	0	0	0.00
	X		810	190	190	1000	4500	190	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	Y		647	853	853	1500	4500	920	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXCG		916	84	84	1000	4500	92	0	0	0.00
	SX		436	564	564	1000	4500	605	0	0	0.00
			956	44	44	1000	4500	44	0	0	0.00
	X		719	281	281	1000	4500	281	0	0	0.00
	SX		613	387	387	1000	4500	387	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		555	445	445	1000	4500	445	0	0	0.00
			581	419	419	1000	4500	436	0	0	0.00
	SXCG		940	60	60	1000	4500	60	0	0	0.00
	SX		701	299	299	1000	4500	301	0	0	0.00
	SX		706	294	294	1000	4500	331	0	0	0.00
			934	66	66	1000	4500	66	0	0	0.00
	SXG		554	446	446	1000	4500	489	0	0	0.00
			661	339	339	1000	4500	363	0	0	0.00
	SX		898	102	102	1000	4500	102	0	0	0.00
	X		749	251	251	1000	4500	264	0	0	0.00
			698	802	802	1500	4500	814	0	0	0.00
	X		720	280	280	1000	4500	301	0	0	0.00
	SX		546	954	954	1500	4500	1002	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 020 - Maintenance Services

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SXG	[redacted]	626	874	946	1500	4500	965	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			910	590	890	1500	4500	973	0	0	0.00
	X		972	28	28	1000	4500	28	0	0	0.00
			938	62	62	1000	4500	62	0	0	0.00
			971	29	29	1000	4500	29	0	0	0.00
	SX		934	66	66	1000	4500	66	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			938	562	850	1500	4500	909	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			917	83	83	1000	4500	83	0	0	0.00
	SX		601	1549	1652	2150	4500	1693	0	0	0.00
	X		950	50	585	1000	4500	649	0	0	0.00
			782	218	218	1000	4500	236	0	0	0.00
			827	173	173	1000	4500	173	0	0	0.00
			796	204	204	1000	4500	204	0	0	0.00
	SXG		999	1	1	1000	4500	1	0	0	0.00
	SX		989	11	11	1000	4500	11	0	0	0.00
			977	23	23	1000	4500	23	0	0	0.00
	SX		953	47	47	1000	4500	47	0	0	0.00
	SX		700	1450	1450	2150	4500	1524	0	0	0.00
			976	24	24	1000	4500	24	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			807	193	193	1000	4500	193	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1441	709	709	2150	4500	729	0	0	0.00
	SX		730	270	270	1000	4500	282	0	0	0.00
			953	47	47	1000	4500	47	0	0	0.00
	X		847	653	872	1500	4500	927	0	0	0.00
			827	173	173	1000	4500	173	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	Y		979	21	21	1000	4500	21	0	0	0.00
	Y		681	319	319	1000	4500	319	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			802	198	198	1000	4500	216	0	0	0.00
	Y		570	430	430	1000	4500	451	0	0	0.00
	X		1004	746	746	1750	4500	797	0	0	0.00
	X		871	129	129	1000	4500	129	0	0	0.00
	X		171	1329	1558	1500	4500	1654	0	0	0.00
	X		789	211	211	1000	4500	232	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORT: DEPT/NAME

EPARTMENT: 020 - Maintenance Services

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	S	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			920	80	80	1000	4500	80	0	0	0.00
[redacted]	X		955	45	45	1000	4500	45	0	0	0.00
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
[redacted]	SX		1000	0	0	1000	4500	0	0	0	0.00
[redacted]	SX		746	754	754	1500	4500	815	0	0	0.00
[redacted]	SX		862	138	138	1000	4500	138	0	0	0.00
[redacted]	XG		698	302	302	1000	4500	322	0	0	0.00
[redacted]	X		1000	0	0	1000	4500	0	0	0	0.00
[redacted]	SX		907	93	93	1000	4500	93	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 021 - MECH MAINT

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM DAC HRS
[redacted]	SX	[redacted]	997	3	3	1000	4500	3	0	0	0.00
	SX		998	2	2	1000	4500	2	0	0	0.00
			797	203	203	1000	4500	219	0	0	0.00
			959	41	41	1000	4500	41	0	0	0.00
			851	149	149	1000	4500	149	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			896	104	104	1000	4500	104	0	0	0.00
			895	105	105	1000	4500	105	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		886	114	114	1000	4500	114	0	0	0.00
			842	158	158	1000	4500	158	0	0	0.00
			980	20	20	1000	4500	20	0	0	0.00
			907	93	93	1000	4500	93	0	0	0.00
			882	118	118	1000	4500	132	0	0	0.00
			996	4	4	1000	4500	4	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			981	19	19	1000	4500	19	0	0	0.00
			971	29	29	1000	4500	29	0	0	0.00
			869	131	131	1000	4500	131	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

[redacted]ARTMENT: 023 - OPERATIONS NUCLEAR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SX	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		877	123	123	1000	4500	123	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500		0	0	0.00
	SXG		766	234	234	1000	4500		0	0	0.00
	SX		1000	0	0	1000	4500		0	0	0.00
	SX		866	134	134	1000	4500	134	0	0	0.00
	SX		745	255	255	1000	4500	259	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		871	129	129	1000	4500	129	0	0	0.00
	SX		804	196	196	1000	4500	197	0	0	0.00
	SX		704	296	296	1000	4500	296	0	0	0.00
	SX		941	59	59	1000	4500	59	0	0	0.00
	SX		706	294	294	1000	4500	316	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		921	79	79	1000	4500	79	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		929	71	71	1000	4500	71	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
	SX		848	152	152	1000	4500	152	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		772	228	228	1000	4500	239	0	0	0.00
	SX		1000	0	15	1000	4500	15	0	0	0.00
	SX		818	182	182	1000	4500	182	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		569	431	431	1000	4500	449	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		988	12	12	1000	4500	12	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		985	15	15	1000	4500	15	0	0	0.00
	SX		424	576	576	1000	4500	625	0	0	0.10
	SX		987	13	13	1000	4500	13	0	0	0.00
	SX		775	225	225	1000	4500	225	0	0	0.00
	SX		787	213	213	1000	4500	213	0	0	.0
	SX		910	90	90	1000	4500	90	0	0	0.00
	SX		785	215	215	1000	4500	238	0	0	0.00
	SX		869	131	131	1000	4500	131	0	0	0.00
	SXG		998	2	2	1000	4500	2	0	0	0.00
	SX		708	292	292	1000	4500	292	0	0	0.00
	SX		976	24	24	1000	4500	24	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 023 - OPERATIONS NUCLEAR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]	SX	[redacted]	696	304	304	1000	4500	304	0	0	0.00
	SX		872	128	128	1000	4500	128	0	0	0.00
	SX		931	69	69	0	0	69	0	0	0.00
	SXG		987	13	13	1000	4500	13	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0		0.00
	SX		655	345	345	1000	4500	37	0		0.00
	SX		809	191	191	1000	4500	1	0		0.00
	SX		953	47	47	1000	4500		0	0	0.00
	SX		633	367	367	1000	4500	367	0	0	0.00
	SXCG		901	99	99	1000	4500	99	0	0	0.00
	SX		932	68	68	1000	4500	68	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		981	19	19	1000	4500	19	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		8	169	169	1000	4500	184	0	0	0.00
	SX		648	352	352	1000	4500	352	0	0	0
	SX		1000	0	0	1000	4500	0	0	0	0
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		877	123	123	1000	4500	123	0	0	0.00
	SXCG		952	48	48	1000	4500	48	0	0	0.00
	SX		874	126	126	1000	4500	137	0	0	0.00
	SX		877	123	123	1000	4500	124	0	0	0.00
	SX		977	23	23	1000	4500	23	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		853	147	147	1000	4500	147	0	0	0.00
	SXG		972	28	28	1000	4500	28	0	0	0.00
	SX		785	215	215	1000	4500	226	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		600	400	400	1000	4500	416	0	0	0.00
	SX		915	85	85	1000	4500	85	0	0	0.00
	SX		979	21	21	1000	4500	21	0	0	0.00
	SX		902	98	98	1000	4500	98	0	0	0.00
	SX		802	198	198	1000	4500	198	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		427	573	573	1000	4500	584	0	0	0.00
	SX		919	81	81	1000	4500	81	0	0	0.00
	SXCG		633	367	367	1000	4500	385	0	0	0.00
			649	351	351	1000	4500	356	0	0	0.00
	XG		828	172	172	1000	4500	1	0	0	0.00
	SX		[illegible]	118	118	1000	4500		0	0	0.00
	SX		1000	0	0	1000	4500		0	0	0.00
	SX		868	132	132	1000	4500	132	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 023 - OPERATIONS NUCLEAR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SX		933	67	67	1000	4500	67	0	0	0.00
	SXCG		979	21	21	1000	4500	21	0	0	0.00
			682	318	318	1000	4500	318	0	0	0.00
	SXG		985	15	15	1000	4500	15	0	0	0.00
	SX		947	53	53	1000	4500	53	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		773	227	227	1000	4500	227	0	0	0.00
	SX		789	211	211	1000	4500	211	0	0	0.00
	SXG		999	1	1	1000	4500	1	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		964	36	36	1000	4500	36	0	0	0.00
	SX		637	363	363	1000	4500	385	0	0	0.00
	SXG		930	70	70	1000	4500	70	0	0	0.00
	SX		812	188	188	1000	4500	209	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		988	12	12	1000	4500	12	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		988	12	12	1000	4500	12	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SX		909	91	91	1000	4500	91	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
	SXG		800	200	200	1000	4500	200	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
	SX		956	44	44	1000	4500	45	0	0	0.00
	SX		932	68	68	1000	4500	68	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 024 - Ops Support

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[illegible]			1000	0	0	1000	4500	0	0	0	0.00
[illegible]	X		867	133	133	1000	4500	133	0	0	0.00
[illegible]			1000	0	0	1000	4500	0	0	0	0.00
[illegible]			983	17	17	1000	4500	17	0	0	0.00
[illegible]	XCG		998	2	2	1000	4500	2	0	0	0.00
[illegible]	X		1000	0	0	1000	4500	0	0	0	0.00
[illegible]			1000	0	0	1000	4500	0	0	0	0.00
[illegible]			992	8	8	1000	4500	8	0	0	0.00
[illegible]			1000	0	0	1000	4500	0	0	0	0.00
[illegible]			1000	0	0	1000	4500	0	0	0	0.00
[illegible]			903	97	97	1000	4500	97	0	0	0.00
[illegible]			405	595	595	1000	4500	666	0	0	0.00

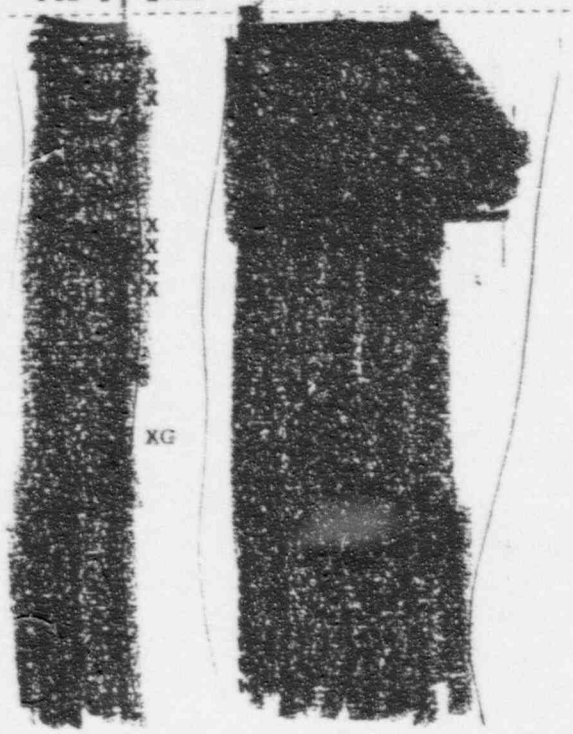
FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 025 - NUCLEAR ASSURANCE



PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			970	30	30	1000	4500	30	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		881	119	119	1000	4500	119	0	0	0.00
	X		717	283	283	1000	4500	305	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0		0	0.00
			990	10	10	1000	4500	10		0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		1755	245	245	2000	4000	245	0	0	0.00
	X		1054	946	946	2000	4500	1030	0	0	0.00
	X		1500	0	0	1500	4500	0	0	0	0.00
	X			86	86	1000	4500	86	0	0	0.00
				0	0	450	4500	0	0	0	0.00
			1865	135	135	2000	4500	135		0	0.00
			1000	0	0	1000	4500	0		0	0.00
			1000	0	1	1000	4500	1	0	0	0.00
			378	72	72	450	4500	88	0	0	0.00
			1 : 6	774	774	2150	4500	833	0	0	0.00
	XG		880	120	120	1000	4500	120	0	0	0.00
			409	41	41	450	4500	41	0	0	0.00
			990	10	10	1000	4500	10	0	0	0.00
			967	33	33	1000	4500	33	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			986	14	193	1000	4500	193	0	0	0.00
			986	14	14	1000	4500	14	0	0	0.00
			925	75	75	1000	4500	75	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			998	2	2	1000	4500	2	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 026 - Business System

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	914	86	86	1000	4500	86	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 0[illegible] - RAD [illegible]EM & CHEM

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[illegible]	SX	[illegible]	1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		970	30	30	1000	4500	30	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		936	64	64	1000	4500	64	0	0	0.00
	SXCG		983	17	17	1000	4500	17	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		925	75	75	1000	4500	75	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		972	28	28	1000	4500	28	0	0	0.00
	SX		984	16	16	1000	4500	16	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		989	11	11	1000	4500	11	0	0	0.00
	SX		876	124	124	1000	4500	136	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 029 - MAINT PROGRAMS

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	951	49	49	1000	4500	49	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	X		987	13	13	1000	4500	13	0	0	0.00
			973	27	27	1000	4500	27	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 030 - SECURITY/NON-FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	XG	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		988	12	12	1000	4500	12	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	YG		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	YG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	YG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DE[illegible]T: 030 - SECURITY/N[illegible]FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[illegible]	SXG	[illegible]	1000	0	0	1000	4500	0	0	0	0.00
			969	31	31	1000	4500	31	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			930	70	70	1000	4500	70	0	0	0.00
			894	106	106	1000	4500	106	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			861	139	139	1000	4500	139	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			896	104	104	1000	4500	104	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	SX		989	11	11	1000	4500	11	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			999	1	1	1000	4500	1	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			960	40	40	1000	4500	40	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
R			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 030 - SECURITY/NON-FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	SX		989	11	11	1000	4500	11	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	YG		999	1	1	1000	4500	1	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
	Y		1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	XCG		1000	0	0	1000	4500	0	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 031 SECURITY/FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			100	0	0	1000	4500	0	0	0	0.00
	SXG		886	114	114	1000	4500	114	0	0	0.00
	X		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

[redacted]

EPARTMENT: 033 - Material Mgmt

PID #	QUAL	NAME	MARGIN	TEDE SITE	TEDE YEAR	GUIDE/EXT SITE	GUIDE/EXT ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	987	13	13	1000	4500	13	0	0	0.00
[redacted]		[redacted]	425	25	25	450	4500	25	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	975	25	25	1000	4500	25	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	951	49	49	1000	4500	49	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

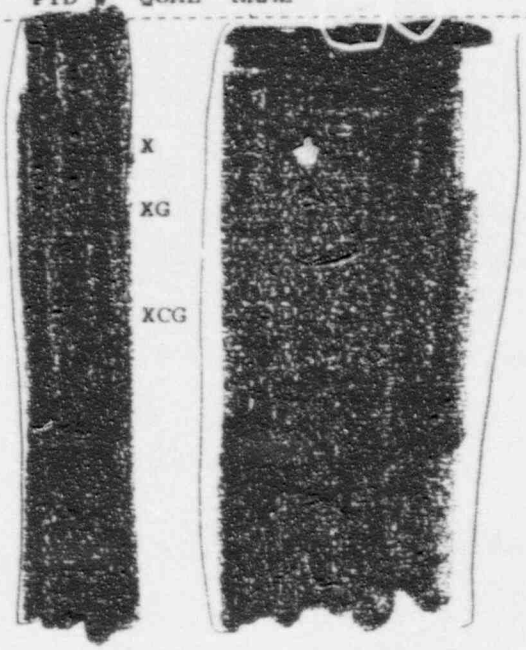
F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 034 - SYSTEM COMPONENT ENG



PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	TEDE -- YEAR	- GUIDE/EXT - SITE	GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			916	84	84	1000	4500	84	0	0	0.00
			990	10	10	1000	4500	10	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			983	17	65	1000	4500	65	0	0	0.00
	X		868	132	132	1000	4500	132	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			903	97	97	1000	4500	97	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			763	237	237	1000	4500	237	0	0	0.50
			987	13	13	1000	4500	13	0	0	0.00
			374	26	26	400	4500	26	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XCG		921	79	79	1000	4500	79	0	0	0.00
			986	14	14	1000	4500	14	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			875	125	125	1000	4500	125	0	0	0.00
			881	119	119	1000	4500	135	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			789	211	211	1000	4500	224	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			965	35	35	1000	4500	35	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			931	69	69	1000	4500	87	0	0	0.00
			916	84	84	1000	4500	84	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 035 - US NCLR REG COMM

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			438	12	12	450	4500	12	0	0	0.00
			434	16	76	450	4500	76	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	120	1000	4500	120	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 037 - VISITORS

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	994	6	6	1000	4500	6	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			945	55	56	1000	4500	56	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	9	1000	4500	9	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 039 - CONTRACTOR/PAINTERS

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	990	10	10	1000	4500	10	0	0	0.00
			964	36	36	1000	4500	36	0	0	0.00
	R		956	44	44	1000	4500	44	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 040 - MANAGEMENT - PSL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			986	14	14	1000	4500	14	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			999	1	13	1000	4500	13	0	0	0.00
			918	82	82	1000	4500	82	0	0	0.00
			987	13	13	1000	4500	13	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

[redacted]

PARTMENT: 041 - TRAINING

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	200	1000	4500	200	0	0	0.00
			954	46	46	1000	4500	83	0	0	0.00
			1037	463	463	1500	4500	495	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		943	57	57	1000	4500	57	0	0	0.00
	SXCG		933	67	67	1000	4500	67	0	0	0.00
	X		927	73	73	1000	4500	73	0	0	0.00
			1000	0	8	1000	4500	8	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		969	31	31	1000	4500	31	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			963	37	37	1000	4500	37	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			962	38	38	1000	4500	38	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			941	59	59	1000	4500	59	0	0	0.00
			900	100	100	1000	4500	100	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XCG		1000	0	0	1000	4500	0	0	0	0.00
			999	1	1	1000	4500	1	0	0	0.00
	XG		978	22	22	1000	4500	22	0	0	0.00
	X		823	677	677	1500	4500	740	0	0	0.00
	SXG		935	65	65	1000	4500	65	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 041 - TRAINING

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			957	43	43	1000	4500	43	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XCG		968	32	32	1000	4500	32	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 042 - PROTECTION SERVICES

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SXG	[redacted]	962	38	38	1000	4500	38	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			979	21	21	1000	4500	21	0	0	0.00
			395	55	55	450	4500	59	0	0	0.00
	SXCG		1000	0	0	1000	4500	0	0	0	0.00
	SX		999	1	1	1000	4500	1	0	0	0.00
	SX		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 044 - NUCLEAR ENGR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			934	66	66	1000	4500	66	0	0	0.00
			966	34	34	1000	4500	46	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			946	54	54	1000	4500	54	0	0	0.00
			989	11	11	1000	4500	11	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			999	1	12	1000	4500	12	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	R		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	R		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			420	30	30	450	4500	42	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			430	20	20	450	4500	20	0	0	0.00
			982	18	18	1000	4500	18	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	14	1000	4500	14	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			980	20	20	1000	4500	20	0	0	0.00
			986	14	14	1000	4500	14	0	0	0.00
			916	84	84	1000	4500	84	0	0	0.00
			989	11	11	1000	4500	11	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			960	40	40	1000	4500	45	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			998	2	2	1000	4500	2	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 044 - NUCLEAR ENGR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			967	33	33	1000	4500	33	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XCG		977	23	23	1000	4500	23	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		884	116	116	1000	4500	116	0	0	0.00
R			1000	0	0	1000	4500	0	0	0	0.00
			943	57	57	1000	4500	57	0	0	0.00
			994	6	6	1000	4500	11	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			989	11	11	1000	4500	11	0	0	0.00
			866	134	134	1000	4500	151	0	0	0.75
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	28	1000	4500	28	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			568	432	432	1000	4500	443	0	0	0.00
			984	16	16	1000	4500	16	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
R			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			963	37	37	1000	4500	37	0	0	0.00
			976	24	24	1000	4500	24	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			929	71	71	1000	4500	71	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			405	45	45	450	4500	45	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			434	16	16	450	4500	16	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 044 - NUCLEAR ENGR

PID	S	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			[redacted]	1000	0	0	1000	4500	0	0	0	0.00
				1000	0	166	1000	4500	166	0	0	0.00
				450	0	0	450	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEP[redacted]T: 046 - Work Cont[redacted]

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	895	105	105	1000	4500	105	0	0	0.00
[redacted]	X	[redacted]	987	13	13	1000	4500	13	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	SXCG	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	987	13	13	1000	4500	13	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	986	14	14	1000	4500	14	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	998	2	2	1000	4500	2	0	0	0.00
R [redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	826	1324	1324	2150	4500	1422	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	32	1000	4500	32	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	SX	[redacted]	577	423	423	1000	4500	423	0	0	0.00
[redacted]	X	[redacted]	962	38	38	1000	4500	38	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X	[redacted]	967	33	33	1000	4500	33	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	939	61	61	1000	4500	61	0	0	0.00
[redacted]	XG	[redacted]	980	20	20	1000	4500	20	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]	XG	[redacted]	987	13	13	1000	4500	13	0	0	0.00
[redacted]	X	[redacted]	954	46	46	1000	4500	46	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 046 - Work Control

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	SX		934	66	66	1000	4500	66	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 047 - PROT. AND CONTROL SYSTEM

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			988	12	12	1000	4500	12	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 048 - EBASCO SRV ENGR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	999	1	1	1000	4500	1	0	0	0.00

. F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 049 - FPL - NDE/I[redacted]

PID #	QUAL	NAME	-- TEDE -- MARGIN	SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]	XCG	[redacted]	450	0	0	450	4500	0	0	0	0.00
			439	11	11	450	4500	11	0	0	0.00
			369	81	172	450	4500	172	0	0	0.00
			975	25	25	1000	4500	25	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			375	75	75	450	4500	86	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 051 - HP ALARA/DECON/LAUND

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	X	[redacted]	316	684	935	1000	4500	968	0	0	0.00
			541	459	873	1000	4500	921	0	0	0.00
	X		700	794	800	1500	4500	829	0	0	0.00
	X		409	1084	2035	1500	4500	2194	0	0	0.00
	X		565	935	1379	1500	4500	1475	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 052 - CONTRACTOR MECH MAIN

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	XCG		1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
R			484	516	516	1000	4500	574	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 053 - CONTRACTOR/ELEC MAIN

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			990	10	10	1000	4500	10	0	0	0.00

DATE: 12/04/96
TIME: 00:50:45

PLANT: PSL
REPORT: DEXPC10R

F L O R I D A   P O W E R   &   L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 055 - INFORMATION SRVCS

PID #	QUAL	NAME	MARGIN	TEDE SITE	TEDE YEAR	GUIDE/EXT SITE	GUIDE/EXT ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	908	92	92	1000	4500	92	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 058 - PRIMARY VALVE CONTR

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 060 - LICENSING

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			912	88	88	1000	4500	105	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 061 - Operations Support Eng.

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			807	193	193	1000	4500	193	0	0	0.00
	SX		959	41	41	1000	4500	41	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
	X		999	1	1	1000	4500	1	0	0	0.00
			967	33	33	1000	4500	33	0	0	0.00
			890	110	110	1000	4500	110	0	0	0.00
R			984	16	16	1000	4500	16	0	0	0.00
			939	61	61	1000	4500	61	0	0	0.00
	X		897	103	103	1000	4500	103	0	0	0.00
			968	32	32	1000	4500	32	0	0	0.00
			939	61	61	1000	4500	61	0	0	0.00
			858	142	143	1000	4500	159	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			968	32	32	1000	4500	32	0	0	0.00
	X		982	18	18	1000	4500	18	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			918	82	82	1000	4500	82	0	0	0.00
	SXG		1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 070 - SGRP - NON-FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted] R W	1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 071 - SGRP - FPL

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
[redacted]	X		861	139	140	1000	4500	140	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 080 - Maint. Outage Proj.

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 081 - VALVE AND WELDING

PID #	QUAL	NAME	MARGIN	TEDE SITE	TEDE YEAR	GUIDE/EXT SITE	GUIDE/EXT ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	SX	[redacted]	893	107	107	1000	4500	107	0	0	0.00
			971	29	29	1000	4500	29	0	0	0.00
	SXG		852	148	148	1000	4500	148	0	0	0.00
			489	511	511	1000	4500	560	0	0	0.00
	SX		927	73	73	1000	4500	73	0	0	0.00
	SX		905	95	95	1000	4500	95	0	0	0.00
	SX		809	191	191	1000	4500	191	0	0	0.00
	XG		532	468	468	1000	4500	468	0	0	0.00
			517	483	483	1000	4500	507	0	0	0.00
	SX		770	230	230	1000	4500	230	0	0	0.00
			673	327	327	1000	4500	327	0	0	0.00
	SXG		830	170	170	1000	4500	183	0	0	0.00
			808	192	192	1000	4500	219	0	0	0.00
	X		990	10	10	1000	4500	10	0	0	0.00
	SX		990	10	10	1000	4500	10	0	0	0.00
			774	226	226	1000	4500	226	0	0	0.00
			820	180	180	1000	4500	223	0	0	0.00
			1000	0	12	1000	4500	12	0	0	0.00
			600	400	400	1000	4500	400	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			781	219	219	1000	4500	236	0	0	0.00
	SXCG		859	141	141	1000	4500	141	0	0	0.00
	SX		732	768	768	1500	4500	780	0	0	0.00
			915	85	85	1000	4500	85	0	0	0.00
			336	114	146	450	4500	146	0	0	0.00
	SX		856	144	144	1000	4500	144	0	0	0.00
			564	436	436	1000	4500	436	0	0	0.00
			895	605	605	1500	4500	679	0	0	0.00
	SX		875	125	125	1000	4500	125	0	0	0.00
	SXCG		383	617	812	1000	4500	839	0	0	0.00
	SX		857	143	143	1000	4500	143	0	0	0.00
			921	79	79	1000	4500	79	0	0	0.00
			937	63	63	1000	4500	63	0	0	0.00
			563	437	437	1000	4500	437	0	0	0.00
	SXG		970	30	30	1000	4500	30	0	0	0.00
			883	117	217	1000	4500	217	0	0	0.00
			1077	423	423	1500	4500	423	0	0	0.00
			928	572	572	1500	4500	645	0	0	0.00
	SX		981	19	19	1000	4500	19	0	0	0.00
	SX		807	193	193	1000	4500	193	0	0	0.00
			988	12	12	1000	4500	12	0	0	0.00
	SX		956	544	544	1500	4500	544	0	0	0.00
			568	432	432	1000	4500	478	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 083 - ROTATING EQUIP

	PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
	[redacted]	XG	[redacted]	1000	0	0	1000	4500	0	0	0	0.00
		SX		817	183	183	1000	4500	192	0	0	0.00
		SX		855	145	234	1000	4500	234	0	0	0.00
		X		841	159	159	1000	4500	170	0	0	0.00
				389	61	61	450	4500	61	0	0	0.00
		X		953	47	47	1000	4500	47	0	0	0.00
				792	208	270	1000	4500	270	0	0	0.00
		X		1000	0	0	1000	4500	0	0	0	0.00
		SX		1252	248	248	1500	4500	248	0	0	0.00
		SX		927	73	73	1000	4500	73	0	0	0.00
		SX		387	1113	1113	1500	4500	1196	0	0	0.00
		SX		990	10	10	1000	4500	10	0	0	0.00
				726	274	274	1000	4500	274	0	0	0.00
				985	15	15	1000	4500	15	0	0	0.00
				950	50	50	1000	4500	50	0	0	0.00
		XCG		779	221	221	1000	4500	232	0	0	0.00
		SX		500	1000	1000	1500	4500	1011	0	0	0.00
		SX		925	75	75	1000	4500	84	0	0	0.00
				716	284	284	1000	4500	284	0	0	0.00
		SXG		914	86	86	1000	4500	86	0	0	0.00
		SX		818	182	182	1000	4500	182	0	0	0.00
				1000	0	0	1000	4500	0	0	0	0.00
R		SX		636	364	364	1000	4500	364	0	0	0.00
				917	83	83	1000	4500	83	0	0	0.00
		SX		921	579	579	1500	4500	604	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 084 - Predictive Maint

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	987	13	13	1000	4500	13	0	0	0.00
			449	1	1	450	4500	1	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			1000	0	2	1000	4500	2	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

EPARTMENT: 090 - SPEAKOUT

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 094 - HUMAN RESOURCES

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 095 - MEDICAL FACILITY

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]	Y	[redacted]	450	0	0	450	4500	0	0	0	0.00
	X		450	0	0	450	4500	0	0	0	0.00
	X		450	0	0	450	4500	0	0	0	0.00
	X		450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			450	0	0	450	4500	0	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
R	X		450	0	0	450	4500	0	0	0	0.00
	Y		450	0	0	450	4500	0	0	0	0.00

F L O R I D A P O W E R & L I G H T

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 096 - SERVICES

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	-- TEDE -- YEAR	- GUIDE/EXT - SITE	- GUIDE/EXT - ANNUAL	SKIN	EXTREMITY UPPER	EXTREMITY LOWER	ACCUM. DAC HRS
[redacted]		[redacted]	1000	0	0	1000	4500	0	0	0	0.00

FLORIDA POWER & LIGHT

RADIATION EXPOSURE MONITORING & ACCESS CONTROL SYSTEM - REMACS

EXPOSURE SUMMARY REPORT

SORTED BY: DEPT/NAME

DEPARTMENT: 097 - FIREWATCH

PID #	QUAL	NAME	MARGIN	-- TEDE -- SITE	YEAR	- GUIDE/EXT - SITE	ANNUAL	SKIN	EXTREMITY UPPER	LOWER	ACCUM. DAC HRS
[redacted]			758	242	242	1000	4500	242	0	0	0.00
			851	149	149	1000	4500	149	0	0	0.00
			891	109	109	1000	4500	109	0	0	0.00
			983	17	17	1000	4500	17	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			962	38	38	1000	4500	38	0	0	0.00
			967	33	33	1000	4500	33	0	0	0.00
			953	47	47	1000	4500	47	0	0	0.00
			781	219	219	1000	4500	219	0	0	0.00
			1000	0	0	1000	4500	0	0	0	0.00
			986	14	14	1000	4500	14	0	0	0.00

REGION II

ATLANTA, GEORGIA

PLANT STATUS REPORT

ST. LUCIE

SEPTEMBER, 1995

PLANT STATUS REPORT FOR ST. LUCIE (9/95)

TABLE OF CONTENTS

PART 1 - FACILITY DESCRIPTION

1.1 FACILITY/LICENSEE

FACILITY: St. Lucie Units 1 and 2
PLANT LOCATION: Hutchinson Island near Port St. Lucie, Florida
LICENSEE: Florida Power and Light Co. (Corporate Office in Juno Beach, Florida)

1.2 UTILITY SENIOR MANAGEMENT

CORPORATE:

J. L. Broadhead (Jim), Chairman of the Board and CEO
J. H. Goldberg (Jerry), President, Nuclear Division

SITE:

D. A. Sager (Dave) - St. Lucie Plant Vice President
C. L. Burton (Chris) - Services Manager
L. W. Bladow (Wes) - Nuclear Assurance Manager
H. F. Buchanan (Hank) - Health Physics Supervisor
R. L. Dawson (Bob) - Licensing Manager
D. J. Denver (Dan) - Site Engineering Manager
H. L. Fagley (Herman) - Construction Services Manager
P. L. Fincher (Pat) - Training Manager
R. J. Frechette (Bob) - Chemistry Supervisor
P. Fulford (Paul) - Operations and Testing Support Supervisor
J. Marchese (Joe) - Maintenance Manager
W. L. Parks (Bill) - Reactor Engineering Supervisor
C. A. Pell (Ash) - Outage Manager
L. A. Rogers (Lee) - Systems and Component Engineering Manager
J. Scarola (Jim) - Plant General Manager
J. A. West (Jeff) - Operations Manager
C. H. Wood (Chuck) - Operations Supervisor

1.3 NRC STAFF

REGION II, Atlanta, GA:

S. D. Ebneter (Stew), Regional Administrator, (404) 331-5500
L. A. Reyes (Luis), Deputy Regional Administrator (404) 331-5610
B. A. Boger (Bruce), Acting Director DRP, (404) 331-5623
D. M. Verrelli (Dave), Branch Chief, (404) 331-5535
K. D. Landis (Kerry), Section Chief, (404) 331-5509
R. P. Schin (Bob), Project Engineer, (404) 331-5561
E. Lea (Edwin), Project Engineer, (404) 331-7096

SITE:

R. L. Prevatte (Dick), Senior Resident Inspector, (407) 464-7822
M. S. Miller (Mark), Resident Inspector, (407) 464-7822

NRR:

S. A. Varga (Steven), Director, Division of Reactor Projects-I/II, (301) 504-1403
J. A. Zwolinsky (John), Deputy Director, Division of Reactor Projects-I/II, (301) 504-1335
D. B. Matthews, Deputy Director, Project Directorate II-2, (301) 415-1490
J. A. Norris (Jan), Senior Project Manager, Project Directorate II-2, (301) 504-1483

AEOD:

S. Israel (Sandy), Reactor Operations Analysis Branch, (301) 415-7573

1.4 LICENSE INFORMATION

	Unit 1	Unit 2
Docket Nos.	50-335	50-389
License Nos.	DPR-67	NPF-16
Construction Permit Nos.	CPPR-74	CPPR-144
Construction Permit Issued	7/1/70	5/2/77
Low Power License	NA	4/83
Full Power License	3/1/76	6/10/83
Initial Criticality	4/22/76	6/2/83
1st Online	5/17/76	6/13/83
Commercial Operation	12/21/76	8/8/83

1.5 PLANT CHARACTERISTICS

Description	Units 1 and 2
Reactor Type	Combustion Engineering PWR, 2-loop
Containment Type	Freestanding Steel w/Shield Building
Power Level	830 MWe (2700 MWt)
Architect/Engineer	Ebasco
NSSS Vendor	Combustion Engineering
Constructor	Ebasco
Turbine Supplier	Westinghouse
Condenser Cooling Method	Once Through
Condenser Cooling Water	Seawater

1.6 SIGNIFICANT DESIGN INFORMATION

1.6.1 REACTOR INTEGRITY

Reactor Pressure Vessel (RPV)

With the present fuel type and management policy, Unit 1 is expected to reach a 40-year RPV life. On this unit, the fuel type and management policy have been modified to make that RPV life

span possible. Presently, a program is evolving for RPV life extension beyond the projected 40 years, potentially to 60 years, via a flux reduction program. A flux reduction program has started with the addition of eight absorbers in core corner positions, performance of vessel fluence calculations, and determination of an optimum power profile for each core load. Calculations using current methodology and uncertainty predict a significant RPV life extension, but not to 60 years. Excore dosimetry installed for the current cycle [with planned removal in October, 1994] will be used to reduce calculation uncertainty.

Due to different design and construction characteristics, Unit 2 RPV life expectancy exceeds 60 years. Low leakage core designs are now used for economic reasons, however the low leakage designs provide even greater life expectancy.

Reactor Coolant Pressure Boundary

On this CE plant, ECCS-to-RCS injection points are isolated by at least two check valves and one closed MOV. High pressure safety injection (HPSI), low pressure safety injection (LPSI), and containment spray (CS) pumps' common containment sump suctions are isolated from the containment sump by one closed MOV in conjunction with a closed seismic piping system. The CS headers are isolated from containment by one closed MOV and a check valve in conjunction with a closed seismic piping system. CVCS has the normal complement of two automatic actuation isolation valves.

1.6.2 REACTOR SHUTDOWN

Reactor Protection System

The reactor protection system provides protection for the reactor fuel and its cladding by providing automatic reactor shutdowns (8 trips) based on input from reactor power, reactor coolant pressure, coolant temperature, coolant flow, steam generator pressure, and containment pressure. The RPS is a redundant four-channel system that operates on a two-out-of-four logic.

ATWS Protection

ATWS protection, outside the normal reactor protection system, is initiated via the ESF pressurizer pressure signal. It actuates by opening contactors in the output of the CEA MG sets, thereby interrupting control element assembly power at its source. This protection has been installed on both units per CE, the NSSS, recommendations.

Remote Shutdown Facilities

These facilities are located in the switchgear rooms beneath each unit's control room.

1.6.3 CORE COOLING

Feedwater System

The main feedwater pumps are motor driven with each delivering 50 percent of the flow required for full power.

Turbine Bypass/Steam Dump Capacity

Each unit has five steam bypass valves, providing 45 percent of total capacity.

Unit 1 has one atmospheric dump valve per train (two trains) and Unit 2 has two valves per train. Each unit has the capability of dumping nine percent steam flow to the atmosphere.

Auxiliary Feedwater System

There are two motor-driven pumps on each unit with 100 percent capacity per pump. There is one steam-driven pump on each unit with 200 percent capacity. Any of the three pumps can inject to either steam generator. Automatic initiation and faulted steam generator protection are provided by each unit's Auxiliary Feedwater Actuation System provided by the NSSS.

Emergency Core Cooling System

In each unit, there are two HPSI pumps and two LPSI pumps with no unit-to-unit cross-connections. One pump of each type per unit will handle a postulated LOCA. The LPSI pumps also provide decay heat removal as required when the unit is shut down.

Decay Heat Removal

As indicated above, the LPSI pumps also provide decay heat removal as required when the unit is shut down by taking suction from the RCS (hot legs), passing the fluid through the shutdown cooling heat exchangers, and returning it to the RCS (cold legs). The heat removing medium is CCW - discussed in section 1.7.6 below. Shutdown cooling flow path overpressure protection is provided by automatic isolation valves and various relief valves in the system.

1.6.4 CONTAINMENT

Pressure Control/Heat Removal

There are two containment spray pumps and four containment fan coolers available per unit to suppress pressure spikes and cool the containment. One CS pump and two fan coolers will handle a postulated LOCA. There are no unit-to-unit cross-connections. This engineered safety feature is automatically started by ESFAS.

Hydrogen Control

Containment hydrogen control post-LOCA is accomplished on each unit by two trains of hydrogen recombiners located on the operating deck inside containment. By elevating, in a controlled manner, the temperature of containment atmosphere flowing through the recombiner, the recombiner units recombine hydrogen and oxygen to form water, thus preventing the buildup of hydrogen to potentially explosive levels.

1.6.5 ELECTRICAL POWER

Offsite AC

The station switchyard is connected to the transmission system by three independent 240 KV lines that share a right of way and interconnect with FPL's grid on the mainland approximately 10 miles West of the plant site. There are two independent offsite power feeds from the station switchyard to the emergency busses.

Onsite AC

Onsite AC power is provided by four EDGs (two per unit). EDGs are independent of other plant systems except vital DC power for control of starting. A Station Blackout (SBO) cross connection is installed and tested. This cross-connection serves the emergency busses directly and reduces cross-connect time to less than 15 minutes.

DC Power

Two trains of vital batteries per unit have been routinely tested for four-hour DC load profiles. Recently, due to cell replacement, they have been tested for three-hour battery capacity instead. The battery capacity test is harsher than the load profile test. There are four normal chargers per unit with swing chargers available for service. Non-safety batteries can be cross-connected to the safety-related swing bus if needed.

Instrumentation Power

Each unit has four inverters, two powered from each vital DC train, that provide four trains of instrumentation power.

Station Blackout Resolution Status

Unit 2 is a four-hour "DC coping" plant per the original license while Unit 1 is subject to the station blackout (SBO) rule of 10 CFR 50.63 requiring additional licensee action (unit-to-unit cross-connect of 4160V bus).

1.6.6 SAFETY-RELATED COOLING WATER SYSTEMS

Intake Cooling Water (Service Water)

Intake cooling water (ICW) for each unit originates in a common canal called the Intake Canal. The canal level varies with the tides since it is filled by a level difference between the Atlantic Ocean and the canal. One 16-foot and two 12-foot diameter pipes pass under the beach to connect the ocean and canal. The intake pipe ends in the Atlantic are covered by intake structures (rebuilt in 1991) intended to limit flow velocities, particularly vertical velocity, to reduce marine life entrapment. After use, ICW returns to the ocean through a Discharge Canal and under-beach pipes.

Each unit has two trains of ICW plus a swing pump that can be aligned to either train electrically and physically. The licensee has converted the deep draft ICW pumps from externally (water) lubricated to self-lubricated to increase reliability of the lubrication water source. The 100 percent (each) capacity pumps take suction from the intake canal via a canal intake structure using traveling screen debris protection. The intake canal structures adjacent to the ICW pump suctions are continuously injected with a hypochlorite solution to reduce marine growth in the associated piping and heat exchangers. Commencing 3/92, periodic injection of a clamicide at the intake structures, primarily to control marine growth affecting the turbine condensers, has also somewhat reduced marine growth affecting the ICW system.

The ICW pumps move water through two trains of heat exchangers that cool component cooling water (CCW) and two trains of heat exchangers that cool main turbine cooling water. During a postulated accident, water flow isolates from the turbine cooling heat exchangers. The discharge from the heat exchangers returns via the discharge canal to the ocean.

Increases in debris and silt in the heat exchangers during 1993 indicated that the intake canal needed dredging.

- As of September 1993, the utility was routinely cleaning main condenser waterboxes at reduced power and obtaining necessary dredging permits from the state and Corps of Engineers.

- The canal was dredged in December 1993 and January 1994 with immediate results of reduced waterbox fouling.

Closed Cooling Water Systems

Each unit has two trains of Component Cooling Water (CCW). The arrangement of two pumps and a swing pump mimics the ICW system.

The swing pump can be aligned to either train. The 100 percent (each) capacity pumps drive water through the CCW/ICW heat exchangers and then on to the heat loads, mainly the containment fan coolers and the shutdown cooling (decay heat) heat exchangers (which also can operate as containment spray heat exchangers). Additionally, CCW cools a variety of bearings, seals, and oil coolers for the HPSI, LPSI, and CS pumps. A non-safety-related portion of the CCW system cools reactor coolant pump seals and the spent fuel pool. This section isolates upon engineered safety features actuation.

1.6.7 SPENT FUEL STORAGE

Wet storage capability exists up to the year 2002 (Unit 2) and 2007 (Unit 1).

1.6.8 INSTRUMENT AIR SYSTEM

Instrument air compressors and driers, installed several years ago on each unit, provide all instrument air for Unit 2 and all but containment air for Unit 1. These have increased instrument air reliability. Unit 1 also has instrument air compressors inside containment.

1.6.9 STEAM GENERATORS

Each unit has two large steam generators (SGs) rather than the three or four usually seen. The licensee has begun to focus on a Unit 1 SG replacement in 1997. The SGs are under construction at the B&W Canada shops and a site organization is functioning.

1.7 EMERGENCY RESPONSE FACILITIES/PREPAREDNESS

Emergency Operations Facility:	10 miles West of site. I-95/Midway Rd. Exit
Technical Support Center:	Onsite, Adjacent to Unit 1 Control Room
Operational Support Center:	Onsite, 2nd floor of North Service Building

The last annual emergency preparedness exercise was in May, 1995. This exercise was not formally evaluated by the NRC. The next emergency preparedness exercise is scheduled for February, 1996.

Since St. Lucie site has a high probability of hurricanes, communications facilities were improved following the Turkey Point experience with Hurricane Andrew in August, 1992. Improvements include:

- High Frequency Auto-link with other FPL sites and NRC.

- Enhanced 900 MHZ System for site and mobile communications, with radios also in the licensee's EOF and county emergency facility.

- Cellular phones with hardened antennas.

- Hardened Local Government Radio antenna ties.

1.8 PRESENT OPERATIONAL STATUS (9/19/95)

The unit was shutdown on August 1, 1995, as a result of Hurricane Erin. A series of problems including RCP seal failure, both PORVs inoperable due to incorrect assembly, SDC relief valve problems, associated problems with several other relief valves, inadvertent spraydown of the containment, catastrophic failure of 1B emergency diesel generator, and a leaking flange on a pressurizer safety valve have prevented the unit from being restarted. With the unit down, a large number of operator-work-arounds and other plant deficiencies are pscheduled to be corrected. The next refueling outage is scheduled for April 4, 1996.

Unit 2 was shutdown on April 25, 1995, for approximately seven hours to replace a main turbine digital electro hydraulic power supply. The unit was down powered for several days in June and July to clean condenser water boxes. The unit was shutdown on August 1, 1995, as a result of Hurricane Erin. The unit was restarted on August 4, 1995. Power was reduced from August 17 through 29 to clean condenser water boxes and repair various secondary plant deficiencies. The next refueling outage is scheduled for October 9, 1995.

Availability Factors:

	Unit 1	Unit 2
1991	81.0	100.0
1992	96.5	75.2
1993	74.0	71.8
1994	86.8	79.6
1995 (through 7/95)	93.9	98.3
Cumulative (through 7/95)	77.7	83.7

1.8.1 UNIT 1 OPERATING HISTORY (Past Twelve Months from 8/1/94)

Unit 1 operated continuously during the past 12 months with the following exceptions:

Unit 1 reduced power and entered mode 2 on August 28, 1995, to repair a DEH leak. The unit was returned to power approximately 18 hours later on the same date.

On October 26, 1994, the unit tripped from 100 percent power due to a loss of electrical load. This was the result of arc-over in a potential transformer in the switchyard due to salt buildup. The licensee then entered a unit refueling outage, which had been

scheduled to begin four days later. The unit was returned to service on November 29, 1994.

On February 21, 1995, the unit was removed from service for the replacement of pressurizer code safety valves which had been leaking by the seat since shortly after startup in November, 1994. The unit was returned to service on March 8, 1994.

On March 4, 1995, the unit experienced a 14 minute loss of shutdown cooling. The apparent root cause was operator error by a reactor operator placing one loop of SDC in standby. The operator apparently closed the suction valve to the operating, vice standby, pump. The operator in question has denied the error. The licensee is considering disciplinary action and has relieved the operator of licensed activities.

On June 11, 1995, the unit was down powered to 40 percent to jumper out a cell on 1B safety related battery.

On July 8, 1995, the unit tripped during turbine valve surveillance testing. It returned to power on July 12, 1995.

On August 1, 1995, the unit was shutdown as a result of Hurricane Erin. Due to a series of equipment problems and personnel performance issues, the unit is presently shutdown.

1.8.2 UNIT 2 OPERATING HISTORY (Past Twelve Months from 8/1/95)

Unit 2 operated continuously during the past 12 months with the following exceptions:

On February 21, 1995, the unit tripped as a result of low steam generator water level. The condition was the result of a feedwater regulating valve closure after a steam generator water level control level transmitter failed high. The transmitter was replaced and the unit was returned to service on February 25, 1995.

On April 25, 1995, the unit was shutdown for approximately 8 hours to replace a main turbine DEH power supply.

On August 1, 1995, the unit was shutdown as a result of Hurricane Erin. It was restarted on August 4, 1995, but operated at reduced power from August 17 through 29, 1995, to clean condenser water boxes and repair equipment problems.

1.9 OUTAGE SCHEDULE AND STATUS

Unit 1's last refueling outage began on October 26, 1994, and ended on November 29, 1994. Major activities included: refueling; reactor vessel nozzle and flange weld ISI inspection; installation of a permanent cavity seal ring; replacing reed switches for several CEAs; integrated

safeguards test; steam generator tube inspection and plugging; steam generator sludge lancing; repair of refueling water storage tank; several instances of reduced inventory/ mid-loop operations; replacement of ICW/CCW LOOP logic [HFA latching relays] with pull-to-lock switches; removal [collection] of Rx vessel neutron flux dosimetry; modification of EDG skids to allow access underneath; inspection of ECCS sump area; replacement of a main transformer; modification of containment spray NaOH addition piping; and mechanical, electrical, and I&C systems maintenance.

Unit 2's last refueling outage began on February 13, 1994, and ended April 17, 1994. Major outage activities included: refueling; steam generator tube inspection and plugging; low pressure turbine blade replacement; emergency diesel generator inspection; replacement of two reactor coolant pump mechanical seals; and mechanical, electrical, and I&C systems maintenance. The next Unit 2 refueling outage is scheduled for October, 1995.

PART 2 - PLANT PERSPECTIVE

2.1 GENERAL PLANT PERSPECTIVE

A SALP presentation was conducted on February 17, 1994, covering the SALP period of May 3, 1992, through January 1, 1994. The facility was rated category 1 in all functional areas for the second consecutive SALP period.

In June 1994, St Lucie was dropped from the NRC management list of good performers after experiencing five unit reactor trips in the first half of 1994.

2.2 SALP HISTORY (Past 2 SALP Periods)

The last SALP period, SALP Cycle 10, ended on January 1, 1994. The current SALP period ends on July 1, 1995.

ASSMT. PERIOD	OPS	RAD	MNT/SURV	EP	SEC	ENG/TECH	SAQV
5/1/89 - 10/31/90	1	1	2	1	1	1	1
11/1/90 - 5/2/92	1	1	1	1	1	1	1

	PLANT OPS	MAINTENANCE	ENGINEERING	PLANT SUPPORT
5/3/93 - 1/1/94	1	1	1	1

2.3 SELECTED SALP AREA DISCUSSIONS (9/1/95)

Since July 1995, there has been a series of events that led to questioning the plants overall performance. These have included: a Unit 1 turbine trip due to procedural weakness, operator performance and supervisory oversight; the attempt to restage an RCP seal using inadequate and inappropriate procedural guidance which led to the failure of the second and third stage seals, a main steam isolation signal due to inappropriate operator response, an inadvertent reactor protection system actuation due to inattention to detail by an operator, both pressurizer relief valves being inoperable due to incorrect assembly during a refueling outage, an inoperable shutdown cooling relief valve due to incorrect setpoint margins (a generic problem involving several valves), the spray down of containment due to an inadequate procedure and operator error coupled with an existing operator-work-around. These and several other recent deficiencies involving weak procedures, a general lack of procedural compliance, equipment failures, and personnel errors clearly indicate that the plants past high level of performance has declined. Both units were shutdown on August 1, 1995, for Hurricane Erin. Unit 2 immediately restarted but Unit 1 remained shutdown. The above problems have led to several plant management changes, an overall evaluation of the recent plant problems by a plant requested independent assessment team and a root cause evaluation by the NRC. In a meeting with the NRC on August 29, 1995, the licensee committed to use the results of the independent assessment team to develop an action plan for improvement.

Plant Operations

Summary of Previous Assessment

Within the current SALP cycle, previous assessments have noted a *potential* decline in Operations' performance. Noted indicators included five reactor trips in the first six months of the cycle. No common root causes were identified. Operator actions with regard to the noted trips were generally good. Two entries into reduced inventory operations during the Unit 2 outage were noted as excellent. Procedural weaknesses which indicated a lack of rigor in the review process were noted, as was the fact that temporary changes to procedures were on the increase (indicating increasing attention to procedural adequacy).

Management activities in response to the increase in operational events was determined to be strong, with an increase in overall focus directed at plant operations. The corrective actions program was enhanced, consolidating several programs into one which involves daily management reviews of all documented conditions.

The previous assessments concluded that Operations remained strong in the current period, that management actions were aggressive in dealing with identified weaknesses, and that increased attention to procedural adequacy may be warranted.

Last Six Months

The previous six months has shown an increase in personnel errors involving, the failure to follow procedures, inattention to detail, the failure to maintain awareness of equipment status, and weaknesses in logkeeping. Only one reactor trip, a turbine trip due to operator error, occurred during this time span. Operator response to that event was excellent. Overall response to plant startups, shutdown, power maneuvers has been good. Several findings indicated weaknesses in personnel performance, procedural adequacy, inattention to detail, weak logkeeping, equipment failures, poor communications, and living with operator-work-arounds. They include:

- Overpressurizing the main generator
- Not logging equipment out of service
- Starting a LPSI pump with the suction valve closed
- Overfilling PWT
- Spray down of Unit 1 Containment
- Improper staging of RCP seal resulted in seal failure
- Failure to block MSIS actuation
- Turbine trip due to operator error
- STAR/NCR not evaluating past operability
- Temporary modification not shown on CR drawing
- Loss of SDC, operator closed suction isolation valve
- Weak annunciator response on loss of SDC
- Operator failed to identify level out of sight on EDG cooling water tank
- Spent fuel pool housekeeping
- Failure of SGWL Rosemount transmitter (maintenance)
- 2B LPSI pump found airbound
- Failure to sample SIT within TS required time frame following volume addition (second occurrence in 2 years)
- Failure to identify and analyze hot leg flow stratification

Strengths

Strengths have been identified in operator response to trips, transients, and power maneuvers. Post job or evaluation briefings have overall been timely and thorough with the exception of several recent events.

Weaknesses

See paragraph 1 above.

Conclusion

Operations performance has declined in the past six months. Operators respond well to events but do not always have a questioning attitude, appear to have lapses in procedural use and compliance, and are not identifying and forcing

plant support organizations to correct plant deficiencies. Logkeeping and attention to detail have led to an increase in errors.

Maintenance/Surveillance

Summary of Previous Assessment

Maintenance was assessed as category 1 in the previous SALP. The previous assessments made during the current SALP cycle indicated that the performance level of maintenance activities had not abated. Strong performance had been noted in the support of the Unit 2 outage, and housekeeping and plant preservation activities were deemed good.

Last Six Months

During the past six months, 24 maintenance activities were observed in varying levels of depth. One violation involving the installation of incorrect size motor overload heater was identified. Three potential violations involving, inadequate surveillance, inadequate post maintenance tests, and two inoperable PORV are currently being evaluated. Two non-cited violations involving inadequate control on jumpers were identified. Workers were found to be well skilled and trained overall. Problems have been noted in procedural adequacy and use.

Twenty-two surveillance activities were observed. Two non-cited violations involving the failure to perform surveillance within specified time limits were identified. These were the result of a weak surveillance tracking system.

Strengths

The licensee continues to use and improve on their online maintenance procedure and implementation. Overall maintenance activities performed under this program were well planned and executed. The craft work force is motivated and overall skill level is high. The licensee has completed the development of their maintenance rule and it will be in place and operating by the end of September. The predictive maintenance organization continues to provide early indication of pending failure and assists in root cause evaluation of equipment failures.

Weaknesses

Weaknesses have been identified in the following areas:

- Procedural adequacy
- Procedural compliance
- Installation of incorrect parts
- Personnel errors in work performance
- Surveillance tracking system
- Equipment failures
- Control of lifted leads/jumpers
- Communications

Conclusion

Maintenance performance has declined. Equipment failures have impacted plant operation. Craft personnel are not identifying and correcting procedural deficiencies and are using procedures as general guidance. Individual and group performance is generally excellent for high visibility jobs, but attention to detail appears to lapse on routine work.

Engineering

Summary of Previous Assessment

The previous assessments for this SALP cycle concluded that engineering was generally strong. Good support of the Unit 2 outage was noted, as was good QA with respect to fuel fabrication and receipt inspection.

Last Six Months

Engineering at St. Lucie remains strong. The trend in the number of issues over the past two quarters has gone down. However, a number of issues in the area organizational/program communication have arisen. Recently, there was an upward turn in the number of engineering issues during the recent shut down. Those issues are being inspected. In the aggregate, St. Lucie Engineering has provided valuable support to Operations and Maintenance throughout the past year.

Conclusion

Engineering continues to perform well. No weaknesses have been identified in this functional area.

Plant Support

Radiological Controls

The radiation protection program continues to adequately maintain external and internal radiation exposures ALARA and within

regulatory limits. The licensee adequately incorporated revisions to the radiation protection program for implementation of the new requirements of 10 CFR Part 20. The licensee continued efforts to reduce respirator usage while maintaining personnel exposures TEDE ALARA. The licensee maintained adequate control of radioactive material to include adequate radiological survey activities.

Emergency Preparedness

The licensee continues to maintain an effective EP program.

Security

Security upgrades made prior to the last SALP were notable. The licensee continues to maintain a very effective security program.

Fire Protection

The licensee continues to maintain an effective fire protection program.

Housekeeping

Housekeeping has been generally very good.

PART 3 - SIGNIFICANT EVENTS

3.1 SIGNIFICANT EVENTS BRIEFINGS (Past 12 Months)

Unit 1: None this period

Unit 2: Failure of a GE AK-25 Trip Circuit Breaker

3.2 ENFORCEMENT STATUS/HISTORY (Past 12 Months)

Currently, there is one escalated enforcement actions pending at St. Lucie.

PART 4 - STAFFING AND TRAINING

4.1 OPERATIONS STAFF - OVERALL (9/95)

Average performance of the operations staff has been noted. Control room demeanor of personnel is above average.

Number of Shifts:	(RCO, SRO) Five shift rotation, 8-hour shifts; (NPO, ANPO, SNPO) Five shift rotation, 8-hour shifts.
Number of SROs:	38 active/13 inactive/ 51 total
Number of ROs:	23 active/1 inactive/ 24 total

Total Licensed Operators: 61 active/14 inactive/ 75 total

4.2 WORK FORCE (8/94)

	FPL	Contractor
Plant personnel (excluding disciplines below)	699	122
Training	64	0
Quality Assurance/ISEG/SPEAKOUT	39	0
Materials Management	47	0
Security	11	122
Site Engineering	48	0

4.3 OPERATOR QUALIFICATION/REQUALIFICATION PROGRAM (Past Two Years)

4.3.1 REQUALIFICATION PROGRAM

Last Inspection - 9/26/94, Inspection Report 50-335,389/94-19

Next Inspection - 10/96

4.3.2 INITIAL EXAMS

Last Exams 10/17/94 -	2 RO	2 passed for 100%
	9 SRO	9 passed for 100%
Next Exam 3/25/96 -	6 RO	

4.3.3 GENERIC FUNDAMENTAL EXAM

Next Exam 10/3/95 - 1 applicant

4.4 PLANT SIMULATOR

The simulator is on site and fully certified to meet ANSI/ANS 3.5, 1985.

4.5 INPO ACCREDITATION

All training programs are maintaining INPO accreditation. The site specific simulator has been used for training since 1988 and has been fully certified for approximately 5 years. NRC inspections in the form of operator examinations at the simulator have found no serious problems.

PART 5 - INSPECTION ACTIVITIES

5.1 INSPECTION FOLLOWUP OPEN ITEMS SUMMARY (UNITS 1 AND 2 COMBINED) (10/6/94)

Division	Pre 93	Total	Change from Last Report
DRP	0	18	-12
DRS	0	14	7
DRSS	0	1	-1
Totals	0	33	-6

Note: Each item that applies to both units is counted as one item.

5.2 MAJOR INSPECTIONS

IR-No.	Date	Type
89-02	1/89	RG-1.97
89-03	3/89	NDE
89-07	3/89	EQ
89-09	3/89	Design Control
89-24	10/89	Maintenance Team Inspection
89-27	11/89	EOP Followup
90-09	4-5/90	OSTI
91-03	2-3/91	EDSFI
91-18	9/91	MOV (no negative findings)
91-201	9-10/91	Service Water Inspection
92-14	7/92	Emergency Preparedness Program
92-17	7/92	EDSFI Followup
93-01	1/93	Check Valves
94-11	5/94	MOV Followup
95-05	6/95	Engineering

5.3 PLANNED TEAM INSPECTIONS

None

5.4 INFREQUENT INSPECTION PROCEDURE STATUS

No core modules are overdue at this time.

5.5 SIMS STATUS - OPEN TMI ITEMS

There are no open TMI items.

NRR ASSESSMENT FOR ST. LUCIE

September 1995

CURRENT ISSUES

-Seismic qualification of electrical and mechanical equipment (GL 87-02, USI A-46) issue on Unit 1 is still not resolved. The staff anticipates closure after an audit to be conducted later this year.

-Unit 1 will be replacing steam generators in 1997. The licensee is well into planning for the event.

-Recent events at the plant may be an idication of deteriorating performance.

Contact:

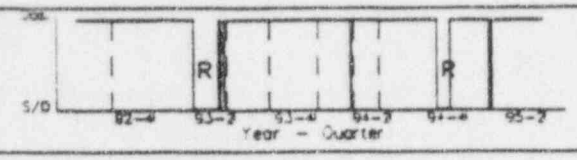
Jan A. Norris
415-1483

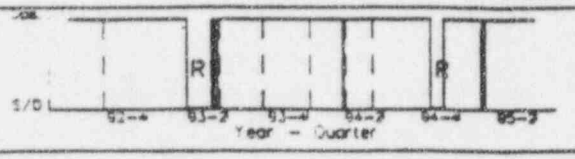
ST. LUCIE 1

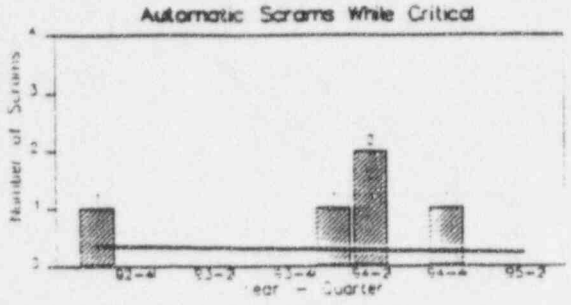
92-3 to 95-2

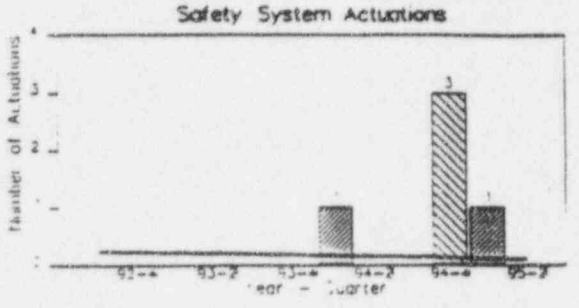
Quarterly Data

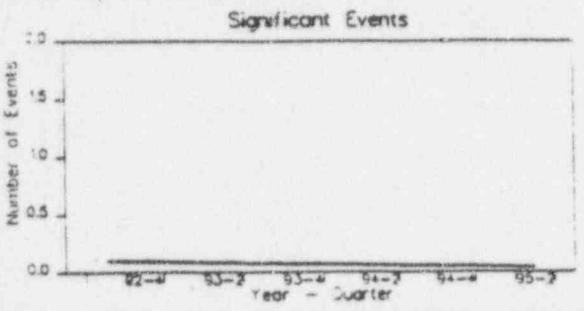
Legend
Shutdown < approx 72 hrs.
Refueling
Industry Avg. Trend
StartUp
Operation
ShutDown
Not Shown using Op. Cycle

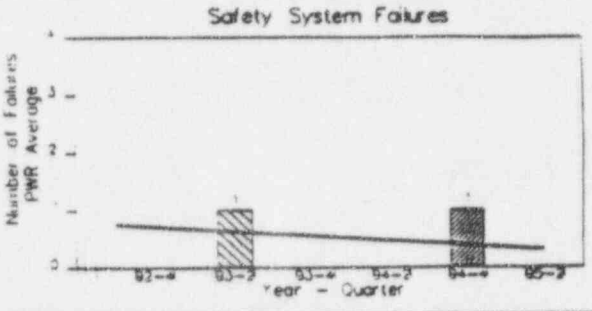


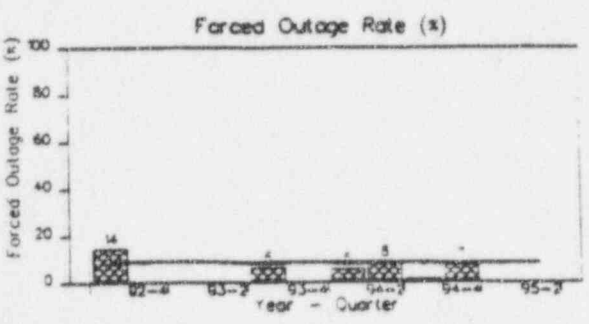


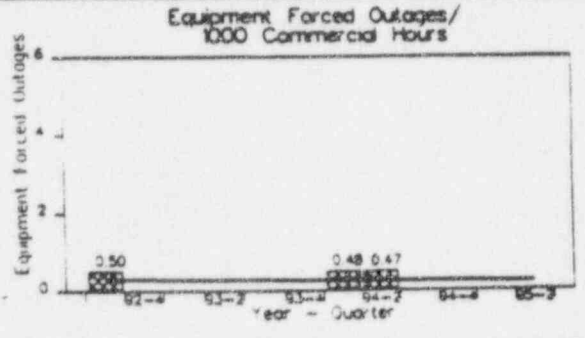


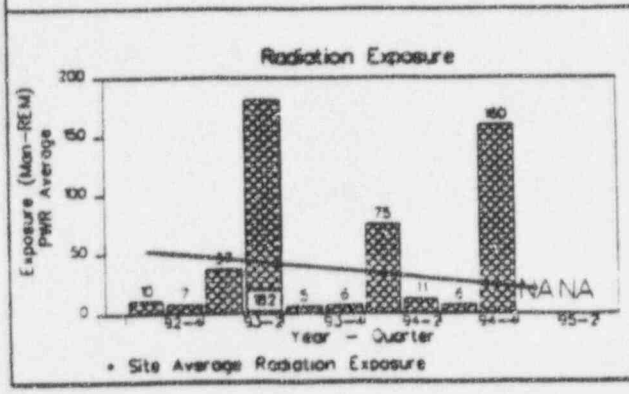


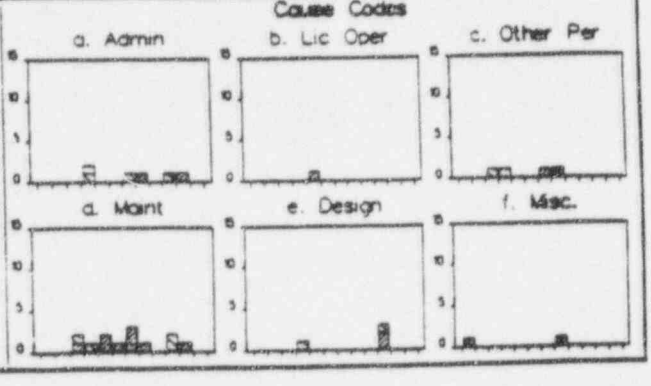










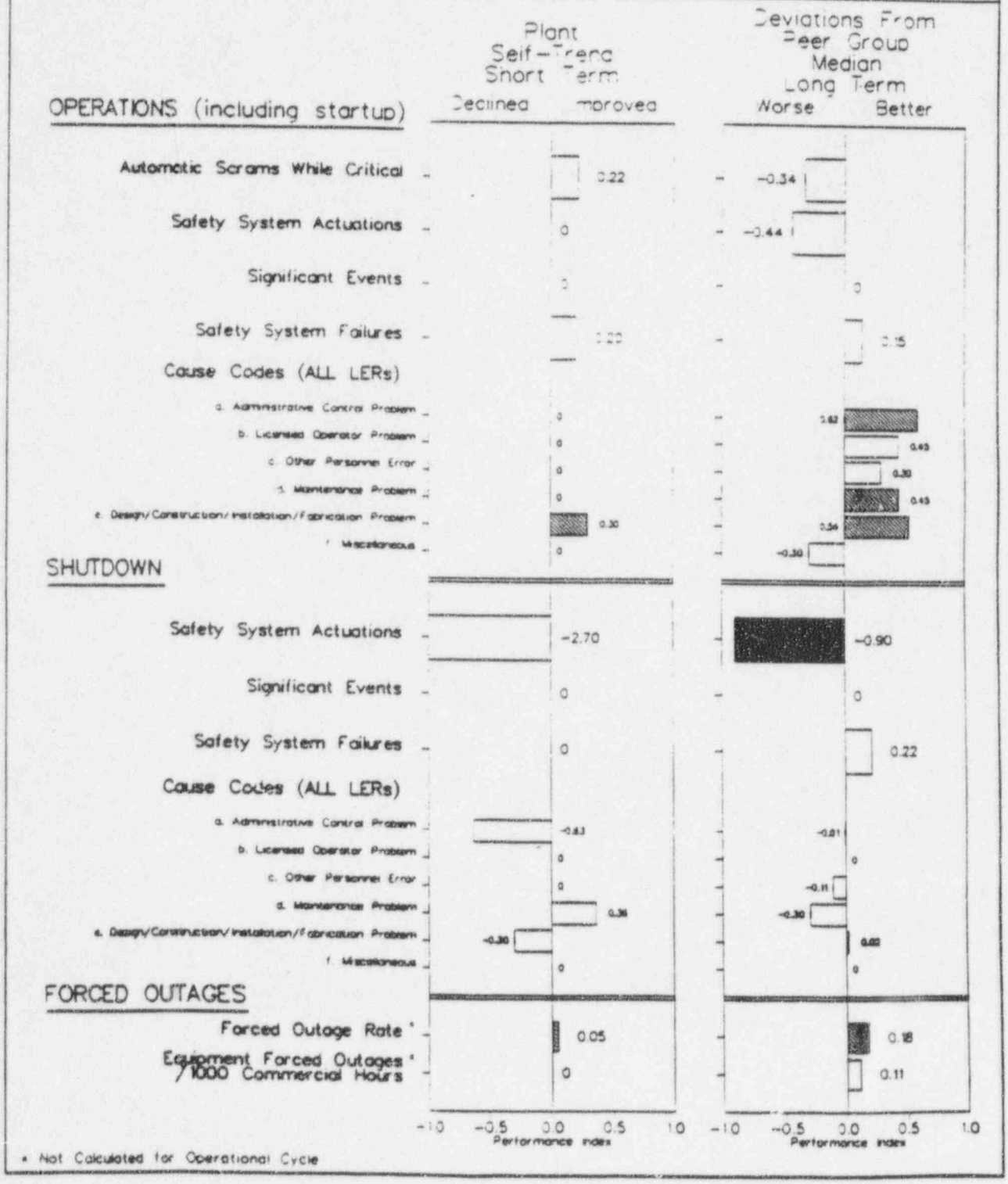


ST. LUCIE 1

Peer Group: Combustion Engineering w/o CPC|

92-3 to 95-2 Trends and Deviations

Legend: Statistical Significance — High, Medium, Low

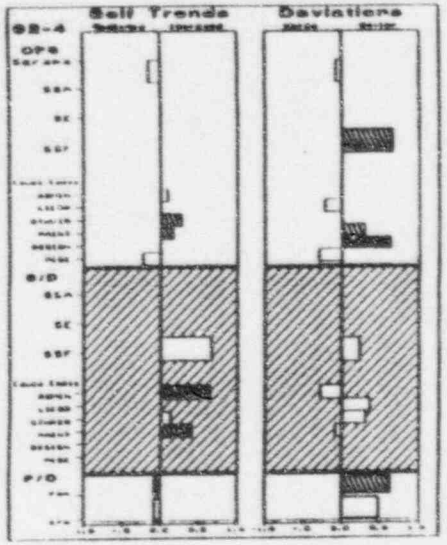


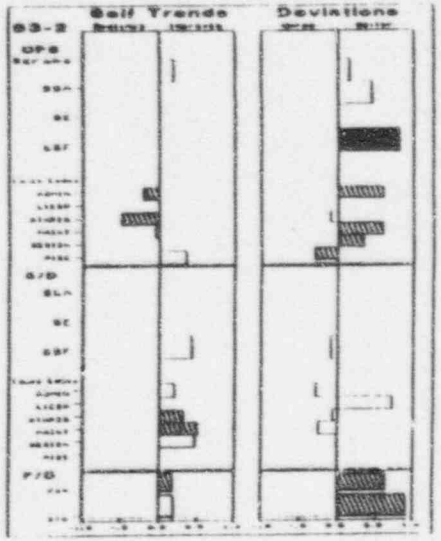
* Not Calculated for Operational Cycle

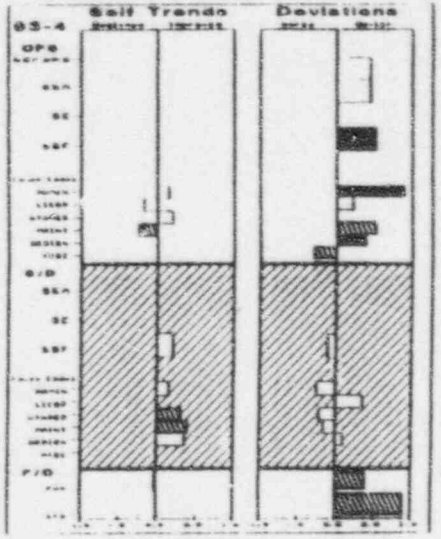
ST. LUCIE 1

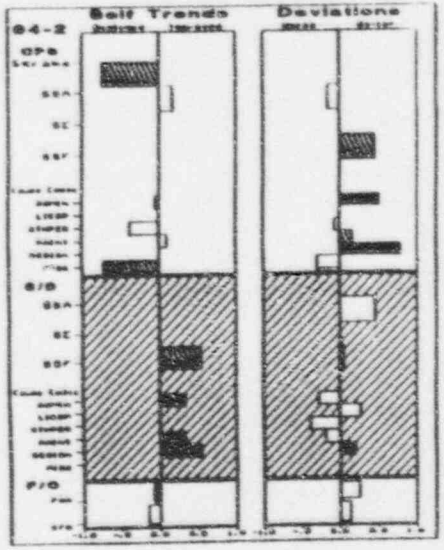
Trends & Deviations

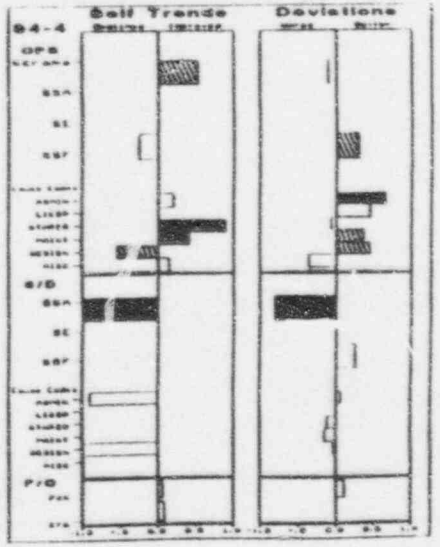
PREDECISIONAL

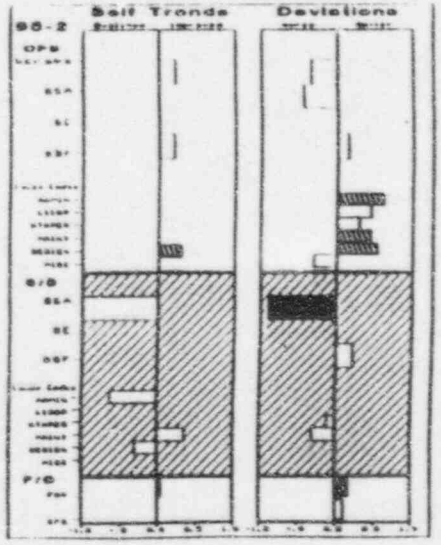












Shaded Regions: Inadequate phase time in last 2 quarters to update calculations

ST. LUCIE 2

PI EVENTS FOR 94-3

NONE

PI EVENTS FOR 94-4

NONE

PI EVENTS FOR 95-1

SCRAM 02/21/95 LER# 38995002 50.72#: 28416 PWR HIST: POWER OPERATIONS AT 100%
DESC : A REACTOR TRIP RESULTED FROM A LOW STEAM GENERATOR LEVEL AFTER A STEAM GENERATOR LEVEL INSTRUMENT FAILED HIGH, CAUSING THE FEED REGULATING VALVE TO CLOSE.

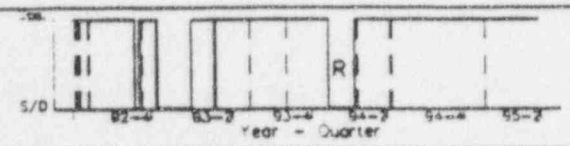
PI EVENTS FOR 95-2

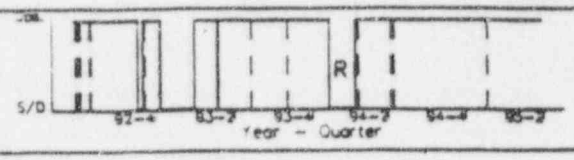
NONE

ST. LUCIE 2

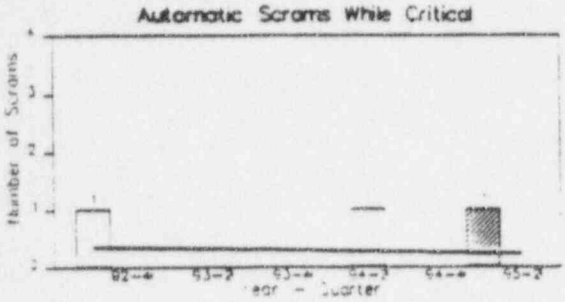
92-3 to 95-2 Quarterly Data

Legend:
Shutdown < approx. 72 hrs. | Refueling | Industry Avg. Trend
StartUp | Operation | ShutDown | Not Shown Using Op. Cycle

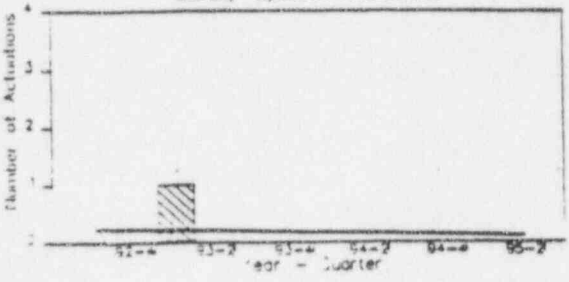




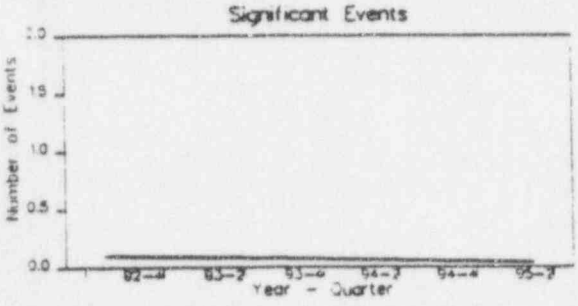
Automatic Scrams While Critical



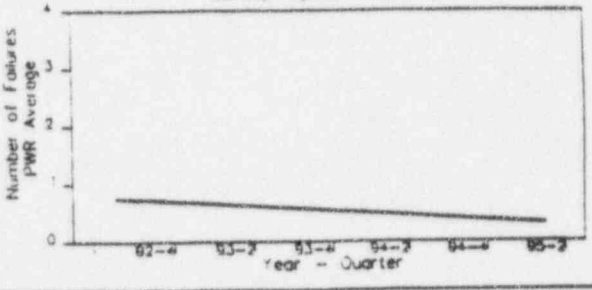
Safety System Actuations



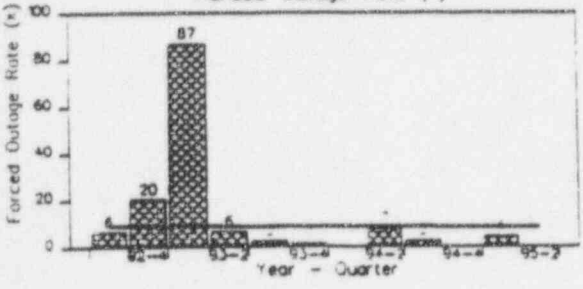
Significant Events



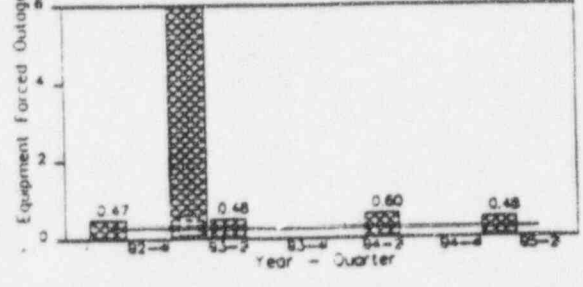
Safety System Failures



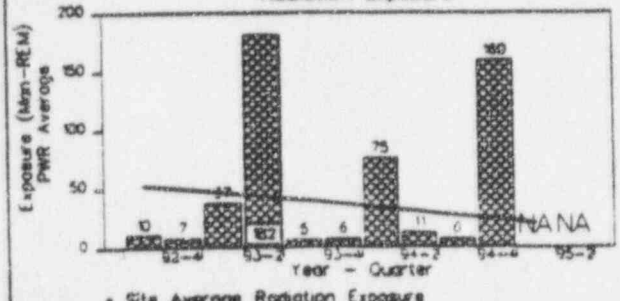
Forced Outage Rate (%)



Equipment Forced Outages/ 1000 Commercial Hours

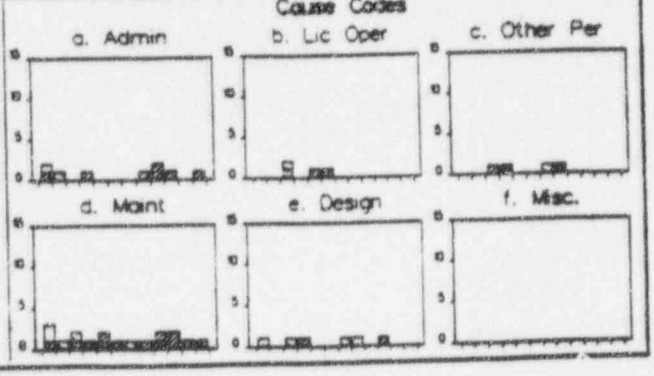


Radiation Exposure



• Site Average Radiation Exposure

Cause Codes

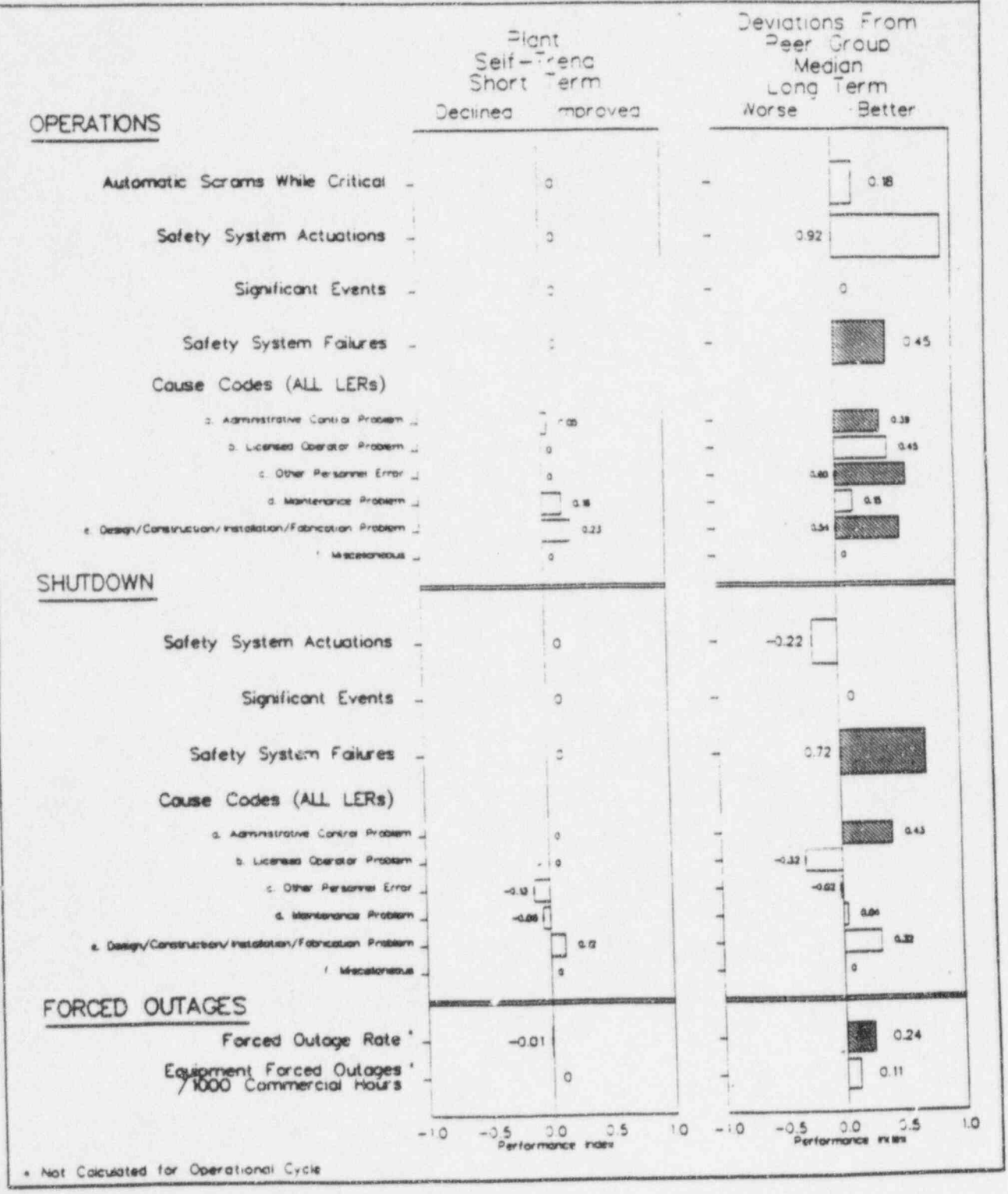


ST. LUCIE 2

Peer Group: Combustion Engineering w/o CPC

92-3 to 95-2 Trends and Deviations

Legend: Statistical Significance — High, Medium, Low

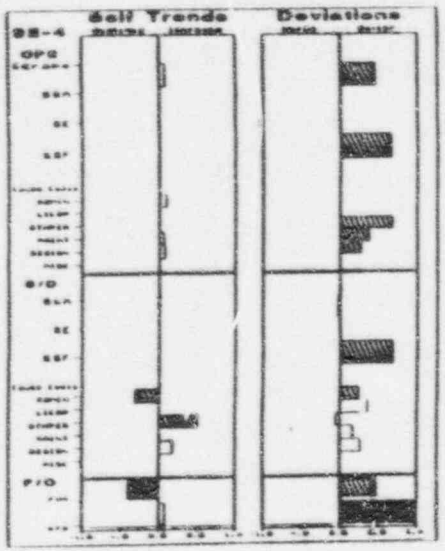


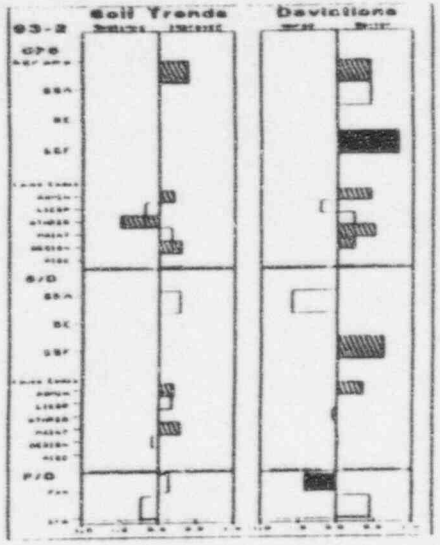
* Not Calculated for Operational Cycle

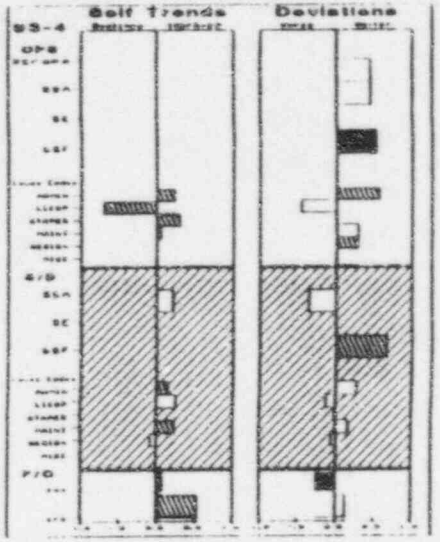
ST. LUCIE 2

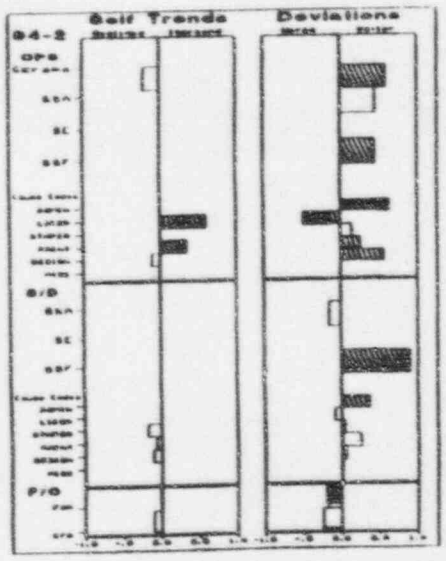
Trends & Deviations

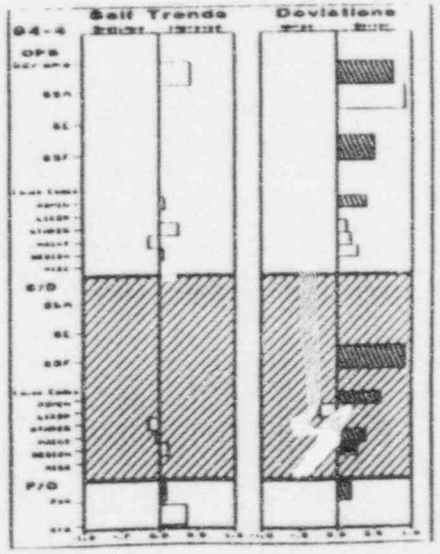
PREDECICIONAL

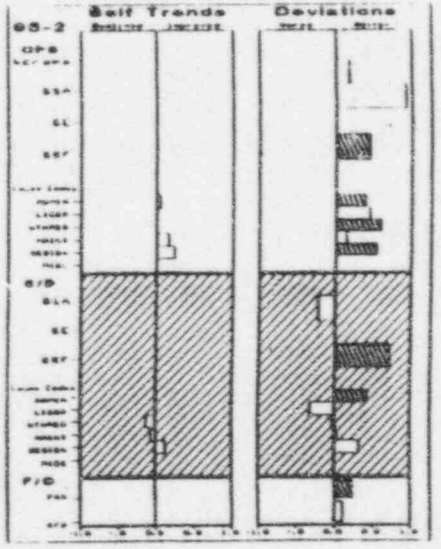












Shaded Regions: Inadequate phase time in last 2 quarters to update calculations

ST. LUCIE 1

PI EVENTS FOR 94-3

NONE

PI EVENTS FOR 94-4

SSF 10/23/94 LER# 33594006 50.72#: 27940
PWR HIST: CONDITION EXISTED IN ALL MODES UP TO 100% POWER SINCE 1978.
GROUP : CONTAINMENT AND CONTAINMENT ISOLATION GROUP
SYSTEM : REACTOR CONTAINMENT BUILDING
DESC : A DESIGN ERROR COULD HAVE ALLOWED CROSS-TRAIN PRESSURIZATION OF AN IDLE CONTAINMENT SPRAY PUMP RESULTING IN A RELIEF VALVE LIFTING AND CONTAINMENT SUMP INVENTORY LOSS TO THE AUXILIARY BUILDING IN EXCESS OF DESIGN LIMITS.

SCRAM 10/26/94 LER# 33594007 50.72#: 27954 PWR HIST: POWER OPERATIONS AT 100%
DESC : A REACTOR TRIP FOLLOWED A MAIN TURBINE TRIP. THIS OCCURRED WHEN A FAULTED SWITCH YARD POTENTIAL TRANSFORMER CAUSED A MAIN GENERATOR LOCKOUT.

SSA 11/22/94 LER# 33594009 50.72#: 28060 PWR HIST: COLD SHUTDOWN
DESC : BOTH EDGS STARTED ON AN SIAS. ONE EDG LOADED ITS BUS WHEN AN ABNORMAL ELECTRICAL LINEUP COMBINED WITH ADDITIONAL LOADING FROM THE SIAS CAUSED THE BUS BREAKER TO OPEN.

SSA 11/22/94 LER# 33594009 50.72#: 28060 PWR HIST: COLD SHUTDOWN
DESC : AN SIAS OCCURRED WHEN TWO OF FOUR PRESSURIZER PRESSURE TRANSMITTERS DRIFTED HIGH DUE TO ENTRAPPED HYDROGEN. THE CHARGING PUMPS WERE SECURED BEFORE THE LTOP SETPOINT WAS REACHED.

SSA 11/24/94 LER# 33594010 50.72#: 28068 PWR HIST: COLD SHUTDOWN
DESC : AN INADVERTENT SAFETY INJECTION ACTUATION SIGNAL OCCURRED DURING REACTOR PROTECTIVE AND ENGINEERED SAFEGUARDS SYSTEM TESTING WHEN A PRESSURIZER PRESSURE BISTABLE TRIPPED. THE CAUSE WAS A DEFICIENT PROCEDURE.

PI EVENTS FOR 95-1

SSA 02/16/95 LER# 33595001 50.72#: 28400 PWR HIST: POWER OPERATIONS AT 100%
DESC : WHILE RESTORING A SAFETY BUS TO A NORMAL LINEUP FOLLOWING RELAY REPLACEMENT, THE BUS WAS DEENERGIZED. THE EMERGENCY DIESEL STARTED AND LOADED ONTO THE BUS.

PI EVENTS FOR 95-2

NONE

NRR ASSESSMENT FOR ST. LUCIE

September 1995

CURRENT ISSUES

-Seismic qualification of electrical and mechanical equipment (GL 87-02, USI A-46) issue on Unit 1 is still not resolved. The staff anticipates closure after an audit to be conducted later this year.

-Unit 1 will be replacing steam generators in 1997. The licensee is well into planning for the event.

-Recent events at the plant may be an idication of deteriorating performance.

Contact:

Jan A. Norris
415-1483

October 19, 1994

ST LUCIE

Briefing Paper for November 1, 1994, Senior Management Briefing

A. Current Plant Status

Unit 1 has been operating at power since June 11, 1994. The next refueling outage is scheduled for October 31, 1994.

Unit 2 has been operating at power since July 15, 1994. The next fefueling outage is scheduled for October 1, 1995.

B. Management

The licensee made the following organization changes effective September 1, 1994:

C. A. Pell, former site Services Manager, became Outage Manager, reporting directly to the Site Vice President. C. Scott, former Outage Manager, reports to Mr. Pell.

J. West, former Operations Supervisor, became Services Manager.

C. Wood, former Assistant Operations Supervisor, became Operations Supervisor.

L. McLaughlin, former Licensing Manager, was added to the Steam Generator Replacement Project Team, reporting to R. Sipos.

R. Dawson, former Maintenance Manager, became Licensing Manager.

J. Marchese, former Turkey Point Construction Services Manager, became St. Lucie Maintenance Manager.

Also, D. Denver, from FPL nuclear technical support in Juno, became site Engineering Manager. He replaced J. Hosmer, who left FPL.

C. Plant Performance

The units have experienced more reactor trips than usual. During the last year, there were six reactor trips plus one unit shutdown and one unit offline for maintenance:

November 2, 1993 - Manual trip of Unit 2 in response to increasing main generator hydrogen temperature.
January 9, 1994 - Manual trip of Unit 1 in response to a main feed pump trip due to a control circuit failure.
March 28, 1994 - Automatic trip of Unit 1 caused by a maintenance foreman opening the generator exciter field breaker (on the wrong unit).

April 3, 1994 - Automatic trip of Unit 1 due to procedural error in powering the A emergency bus from the A EDG for maintenance.

April 23, 1994 - Automatic trip of Unit 2 during maintenance due to pre-existing RPS cabinet wiring error.

June 6, 1994 - Automatic trip of Unit 1 caused by storm blowing aluminum flashing across two phases of main transformer output.

July 8, 1994 - Unit 2 was taken off line (Mode 2) because the 2B1 RCP lower oil level indication failed, incorrectly showing a leak.

July 14, 1994 - Unit 2 was shut down to allow repair of a stuck-closed reactor trip circuit breaker.

The six reactor trips had no identified common causes. Operator performance on the trips was good.

D. Performance Indicators

St. Lucie performance indicators indicate above average performance of both units, but with a recent increase in automatic scrams.

E. Enforcement History

There have been no escalated enforcement actions in 1993 or 1994.

An enforcement panel was held on September 12, 1994, regarding operation of the 1A EDG in an electrical lineup for which the TS-required surveillance testing had not been performed (with the safety-related swing bus powered from the 1A EDG bus). This issue is currently unresolved pending further NRC assessment of safety significance based upon results of licensee completion of TS-required testing for this lineup in November, 1994, during the scheduled Unit 1 outage.

F. SALP

	Period ended May 1992	Period ended January 1994
Operations	1	1
Maintenance	1	1
Engineering	1	1
Rad. Con.	1	
Security	1	
Emerg. Prep.	1	
SA/QV	1	
Plant Support		1

G. INPO

INPO assessment March 1992 - Category 1
INPO/WANO assessment April 1994
Next INPO assessment scheduled for August-October 1995

H. 1994 Precursor Events

On March 18, Unit 2 operators stopped shutdown cooling for six minutes, as a precautionary protection for pumps, until conditions were assessed after a misanalyzed clearance resulted in automatic valve realignments that secured flow to one of two operating trains of shutdown cooling. The NRC assessed the temporary cessation of shutdown cooling as a conservative action by operators.

On July 8, 1994, Unit 2 operators entered TS 3.0.3 (with both trains of ECCS inoperable) due to improperly placing the 2A1 LPSI pump and the 2B charging pump OOS at the same time. The NRC assessed this event as not constituting a violation of TS or a loss of ECCS capability since the time in 3.0.3 was less than one hour and all other ECCS equipment in both trains was operable. Also, the licensee took prompt action to enhance the process for placing equipment out of service to avoid any future similar occurrence.

I. Allegations and DOL Cases

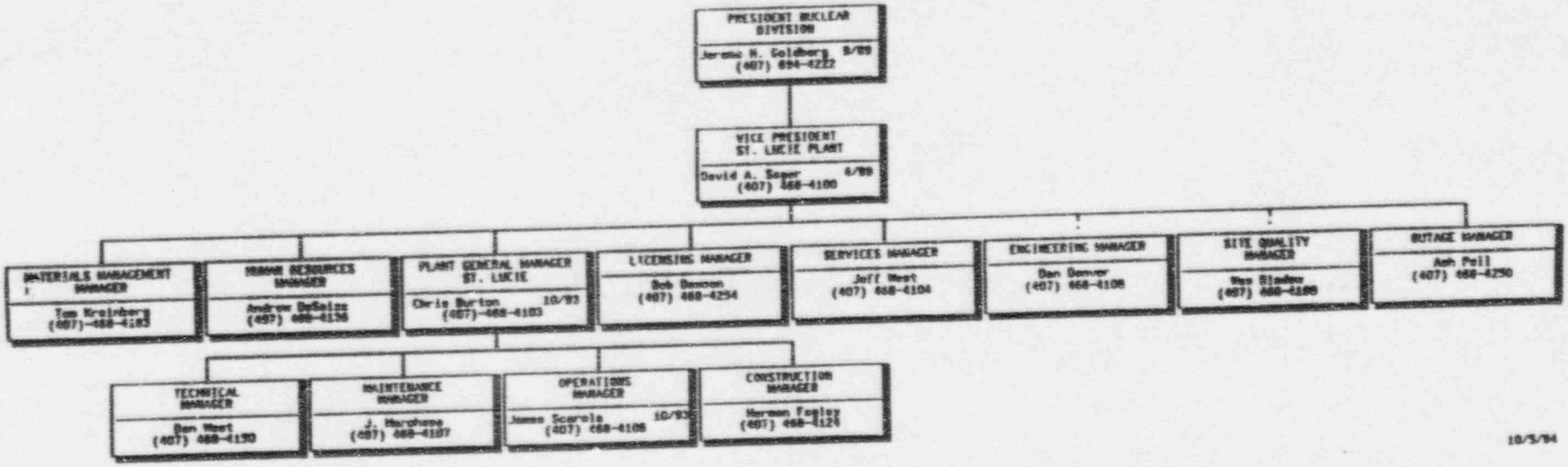
Five allegations are open: one drawing control; one area evacuation ability (bridge maintenance); one negative performance appraisal; one security guards - insufficient number, drinking, etc.; and one licensed operator overtime policy.

No DOL cases are open.

J. Attachments

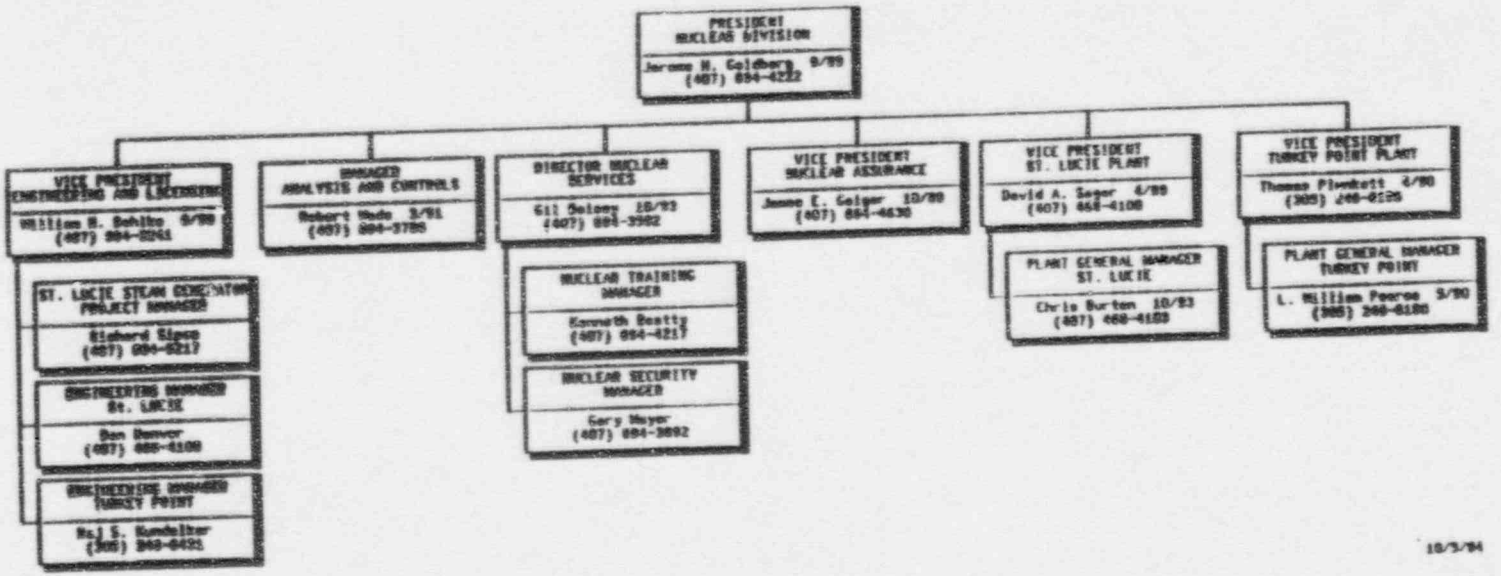
1. Organization Charts
2. Power Profiles
3. Plant Status Report
4. Events Matrix

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE UNITS 1 & 2



10/5/94

FLORIDA POWER AND LIGHT
MANAGEMENT OVERVIEW



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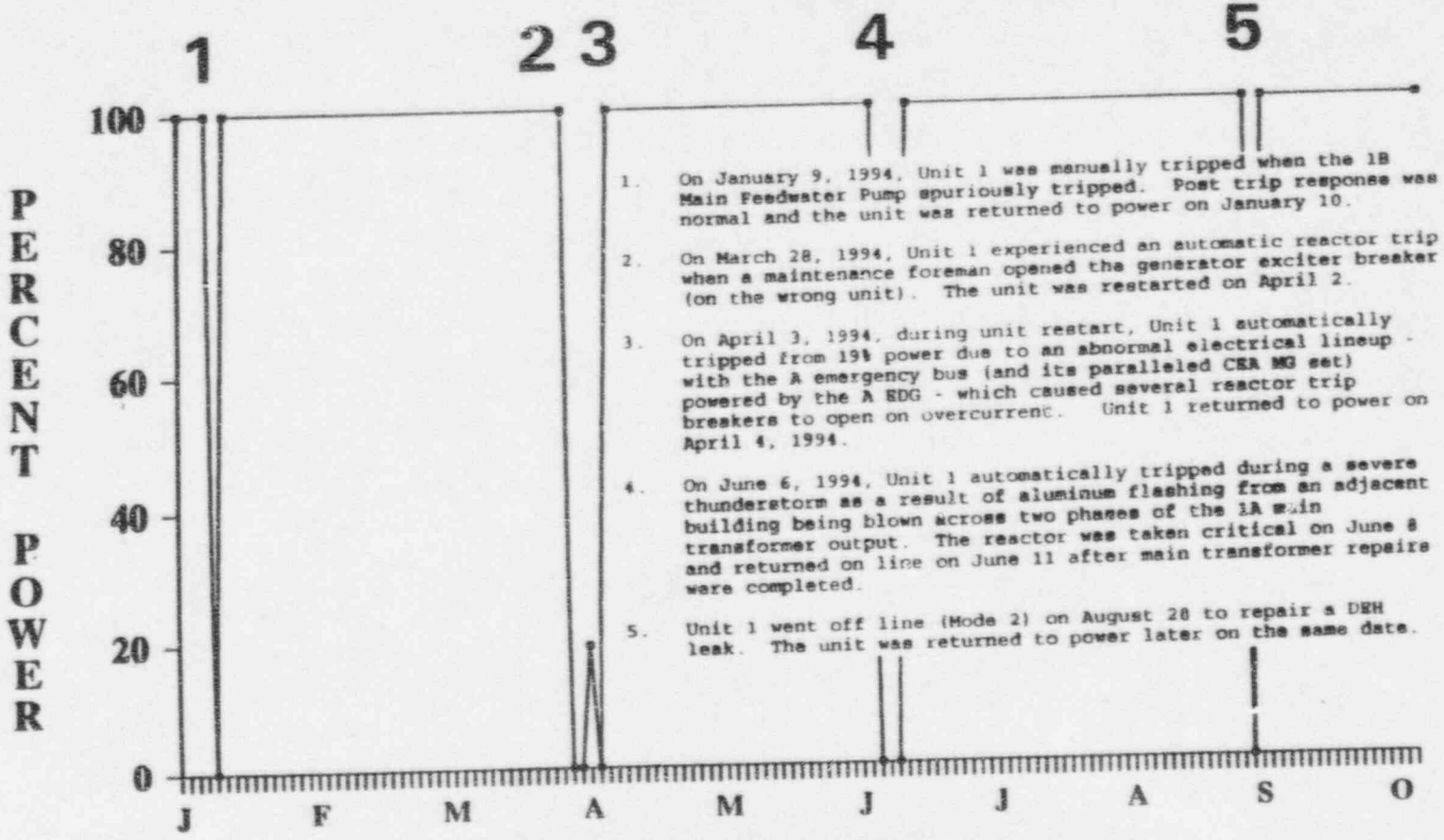
ST. LUCIE 1

Operational Period January 1, 1994 through October 5, 1994



1. On January 9, 1994, Unit 1 was manually tripped when the 1B Main Feedwater Pump spuriously tripped. Post trip response was normal and the unit was returned to power on January 10.

2. On March 28, 1994, Unit 1 experienced an automatic reactor trip when a maintenance foreman opened the generator exciter breaker (on the wrong unit). The unit was restarted on April 2.

3. On April 3, 1994, during unit restart, Unit 1 automatically tripped from 19% power due to an abnormal electrical lineup - with the A emergency bus (and its paralleled CEA MG set) powered by the A EDG - which caused several reactor trip breakers to open on overcurrent. Unit 1 returned to power on April 4, 1994.

4. On June 6, 1994, Unit 1 automatically tripped during a severe thunderstorm as a result of aluminum flashing from an adjacent building being blown across two phases of the 1A main transformer output. The reactor was taken critical on June 8 and returned on line on June 11 after main transformer repairs were completed.

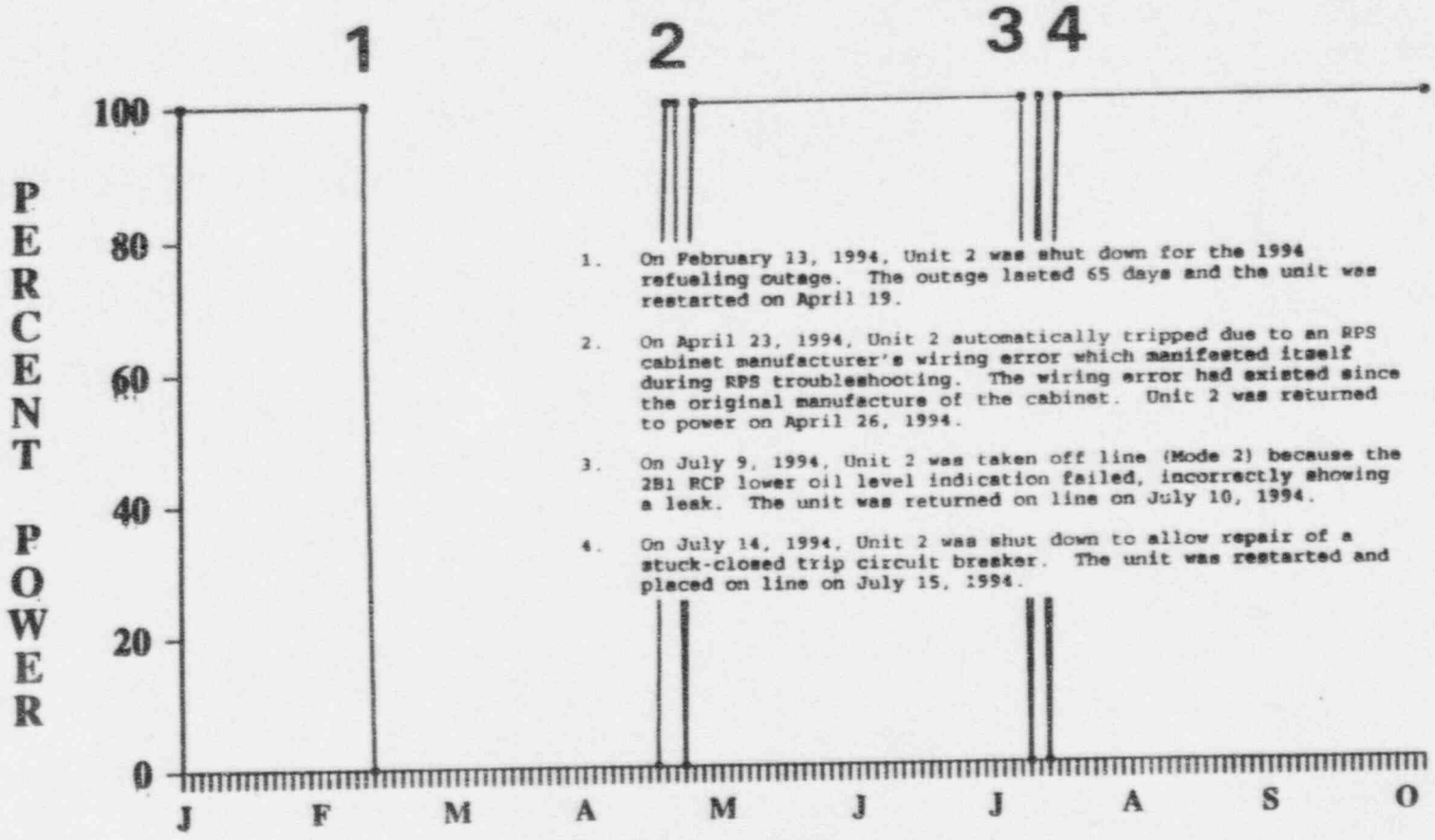
5. Unit 1 went off line (Mode 2) on August 28 to repair a DRH leak. The unit was returned to power later on the same date.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

ST. LUCIE 2

Operational Period January 1, 1994 through October 5, 1994



1. On February 13, 1994, Unit 2 was shut down for the 1994 refueling outage. The outage lasted 65 days and the unit was restarted on April 19.

2. On April 23, 1994, Unit 2 automatically tripped due to an RPS cabinet manufacturer's wiring error which manifested itself during RPS troubleshooting. The wiring error had existed since the original manufacture of the cabinet. Unit 2 was returned to power on April 26, 1994.

3. On July 9, 1994, Unit 2 was taken off line (Mode 2) because the 2B1 RCP lower oil level indication failed, incorrectly showing a leak. The unit was returned on line on July 10, 1994.

4. On July 14, 1994, Unit 2 was shut down to allow repair of a stuck-closed trip circuit breaker. The unit was restarted and placed on line on July 15, 1994.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

OPERATING REACTORS EVENTS BRIEFING 94-25

LOCATION: O-10B11, WHITE FLINT
WEDNESDAY, JULY 20, 1994 11:00 A.M.

COOPER	EMERGENCY DIESEL GENERATOR LOAD SHEDDING TEST DEFICIENCIES
SAINT LUCIE, UNIT 2	REACTOR TRIP BREAKER FAILURE DURING SURVEILLANCE TEST

PRESENTED BY: EVENTS ASSESSMENT BRANCH
DIVISION OF OPERATING REACTOR SUPPORT, NRR

SAINT LUCIE, UNIT 2
REACTOR TRIP BREAKER FAILURE DURING SURVEILLANCE TEST
JULY 14, 1994

PROBLEM:
FAILURE OF A REACTOR TRIP BREAKER TO OPEN DURING A SURVEILLANCE TEST.

CAUSE:
BREAKER OPENING PREVENTED BY FOREIGN MATERIAL LODGED IN TRIP LATCH AREA.

ETY SIGNIFICANCE:
APPEARS TO BE A GENERIC PROBLEM WHICH MAY BE MORE SIGNIFICANT FOR BREAKERS USED IN OTHER APPLICATIONS OR INSTALLED IN NSSS OF OTHER VENDORS.

DISCUSSION:

- ON JULY 14, 1994, DURING MONTHLY REACTOR PROTECTION SYSTEM (RPS) LOGIC MATRIX TESTING, TRIP CIRCUIT BREAKER 5 (TCB5), GE MODEL AK 2-25, FAILED TO OPEN (FIGURE 1).

- OPERATORS UNSUCCESSFULLY ATTEMPTED TO OPEN THE BREAKER FROM THE CONTROL ROOM. THE BREAKER ALSO COULD NOT BE OPENED LOCALLY BY EITHER ELECTRICAL OR MECHANICAL MEANS.

CONTACT: N. FIELDS, NRR/DORS/EAB
REFERENCES: MORNING REPORTS 2-94-0058 AND 2-94-0059

AIT: NO
SIGEVENT: TBD

- TCB5 HAD SUCCESSFULLY OPENED ON THREE PREVIOUS OCCASIONS IN RESPONSE TO PREVIOUS STEPS IN THE SURVEILLANCE PROCEDURE.

- TCB5 IS ONE OF EIGHT REACTOR TRIP BREAKERS IN THE ONE-OUT-OF-TWO TAKEN TWICE REACTOR TRIP COINCIDENCE LOGIC.

- THE LICENSEE ISOLATED TCB5 AS REQUIRED BY TECHNICAL SPECIFICATIONS AND BEGAN A REACTOR SHUTDOWN.

- INSPECTION OF TCB5 FOUND THAT A SMALL PIECE OF PHENOLIC MATERIAL FROM THE CUTOFF SWITCH BODY (PART OF THE ANTI-PUMP CIRCUIT) HAD BROKEN OFF AND BECOME LODGED IN THE TRIP LATCH MECHANISM, PREVENTING ITS MOTION (SEE FIGURES 2 AND 3).

- THE LICENSEE SPECULATES THAT A LOOSE MACHINE SCREW, ATTACHING THE CUTOFF SWITCH TO ITS MOUNT, ALLOWED THE SWITCH TO MIGRATE SUCH THAT ITS BODY WAS DAMAGED AS A RESULT OF COMPONENT MOVEMENT DURING PREVIOUS BREAKER OPERATIONS.

- GE SERVICE ADVISORY LETTER 303.0, ISSUED APRIL 6, 1989, INFORMED OWNERS OF AK 25-1 BREAKERS OF THE POTENTIAL FOR CUTOFF SWITCH BODY CRACKING. THE LICENSEE STATED NO KNOWLEDGE OF SAL.

FOLLOWUP:

- LICENSEE AND A VENDOR REPRESENTATIVE INSPECTED ALL SIMILAR UNIT 2 BREAKERS. SWITCHES WERE FOUND WITH SCRATCHES OR "MOLD MARKS" BUT NONE OF THE CONDITIONS WERE JUDGED SIGNIFICANT. NONE OF THE CUTOFF SWITCHES WERE FOUND WITH LOOSE SCREWS.

- REGION II IS REVIEWING THE EVENT AND LICENSEE ACTIONS IN LIGHT OF THE TECHNICAL SPECIFICATION ACTION STATEMENT.

- THE REGION WILL DRAFT AN INFORMATION NOTICE DESCRIBING THIS EVENT.

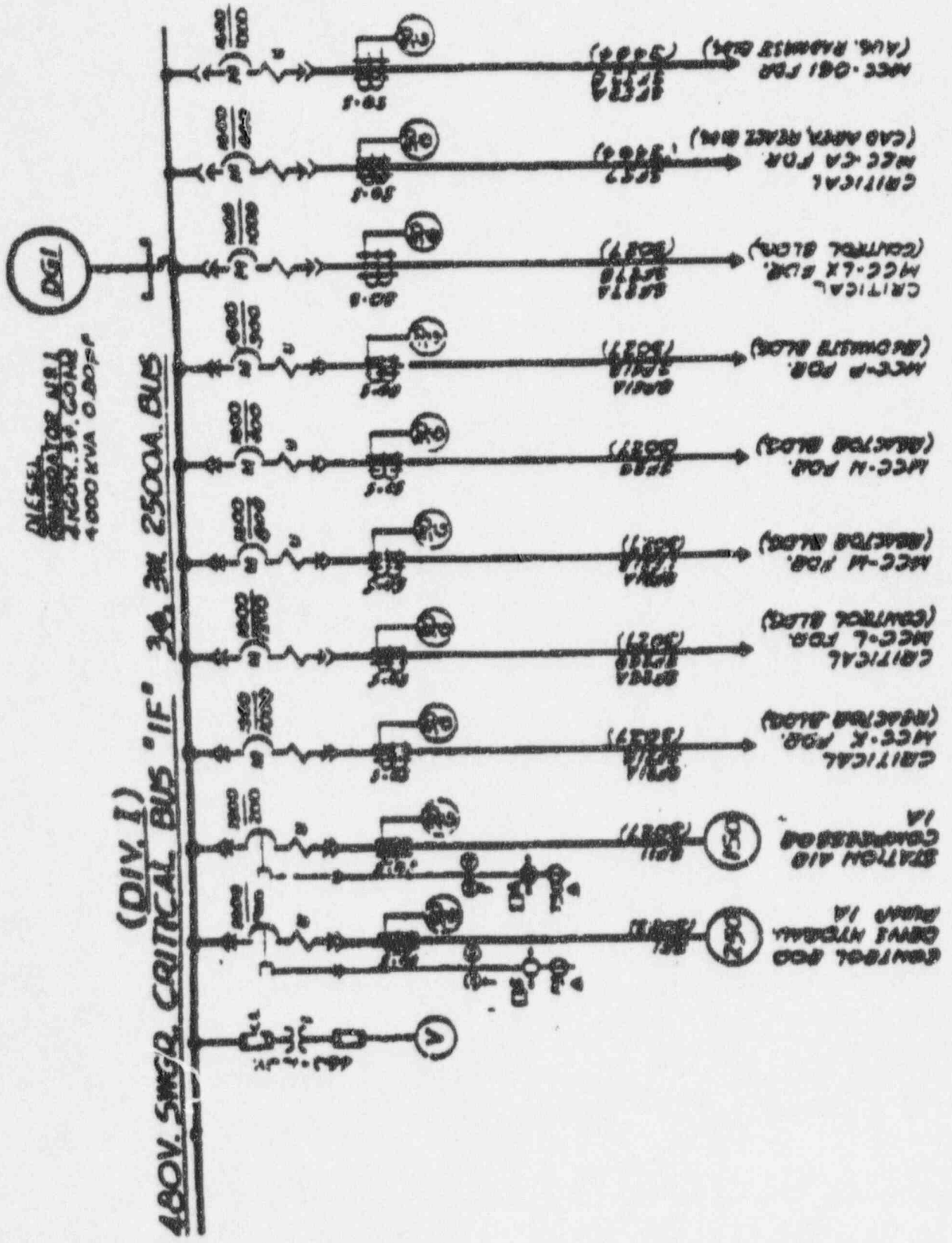


FIGURE 1 - COOPER DIVISION I 480 VAC CRITICAL BUS "1F"

BRIEFING 94-25
ST. LUCIE
FIGURE 1

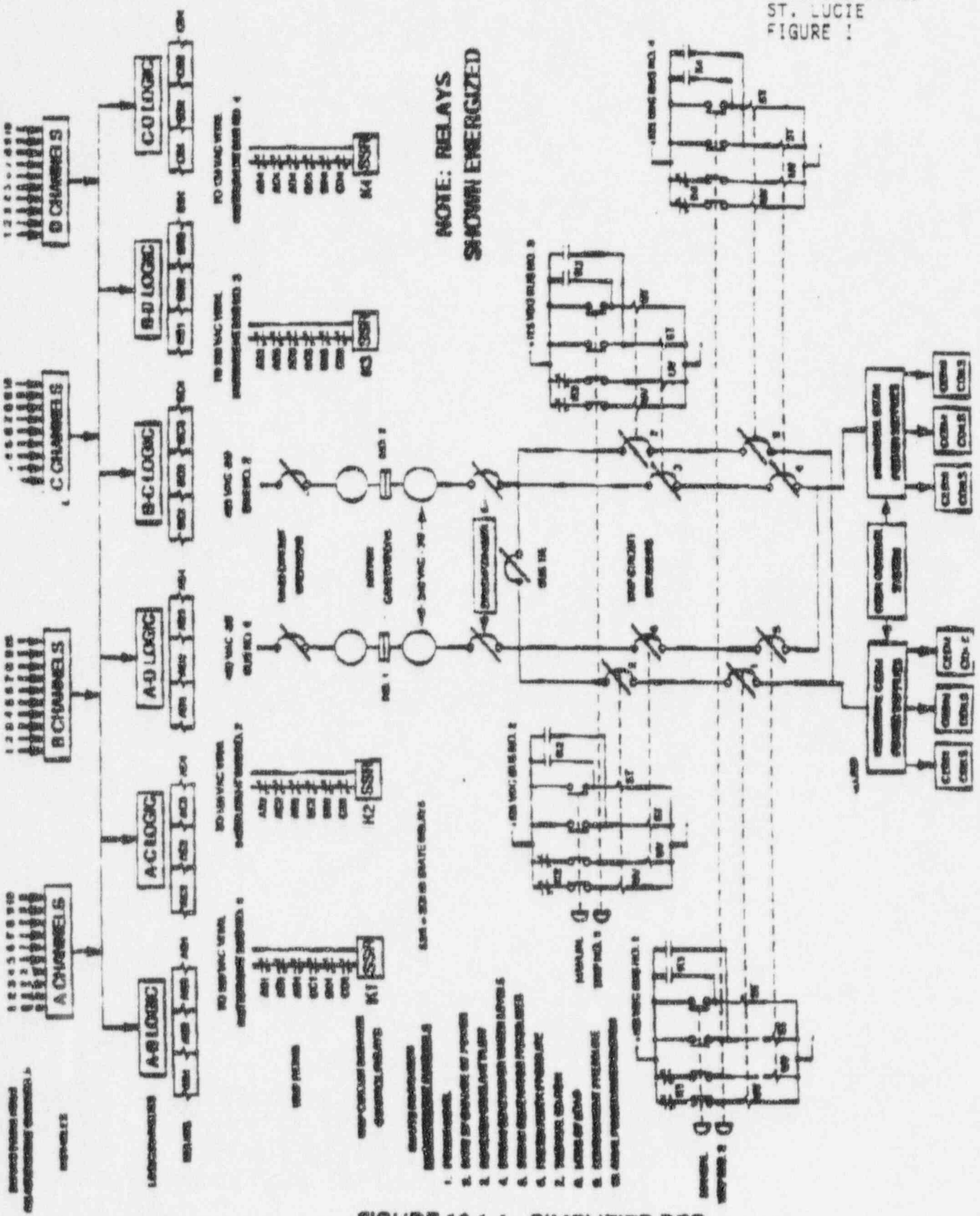
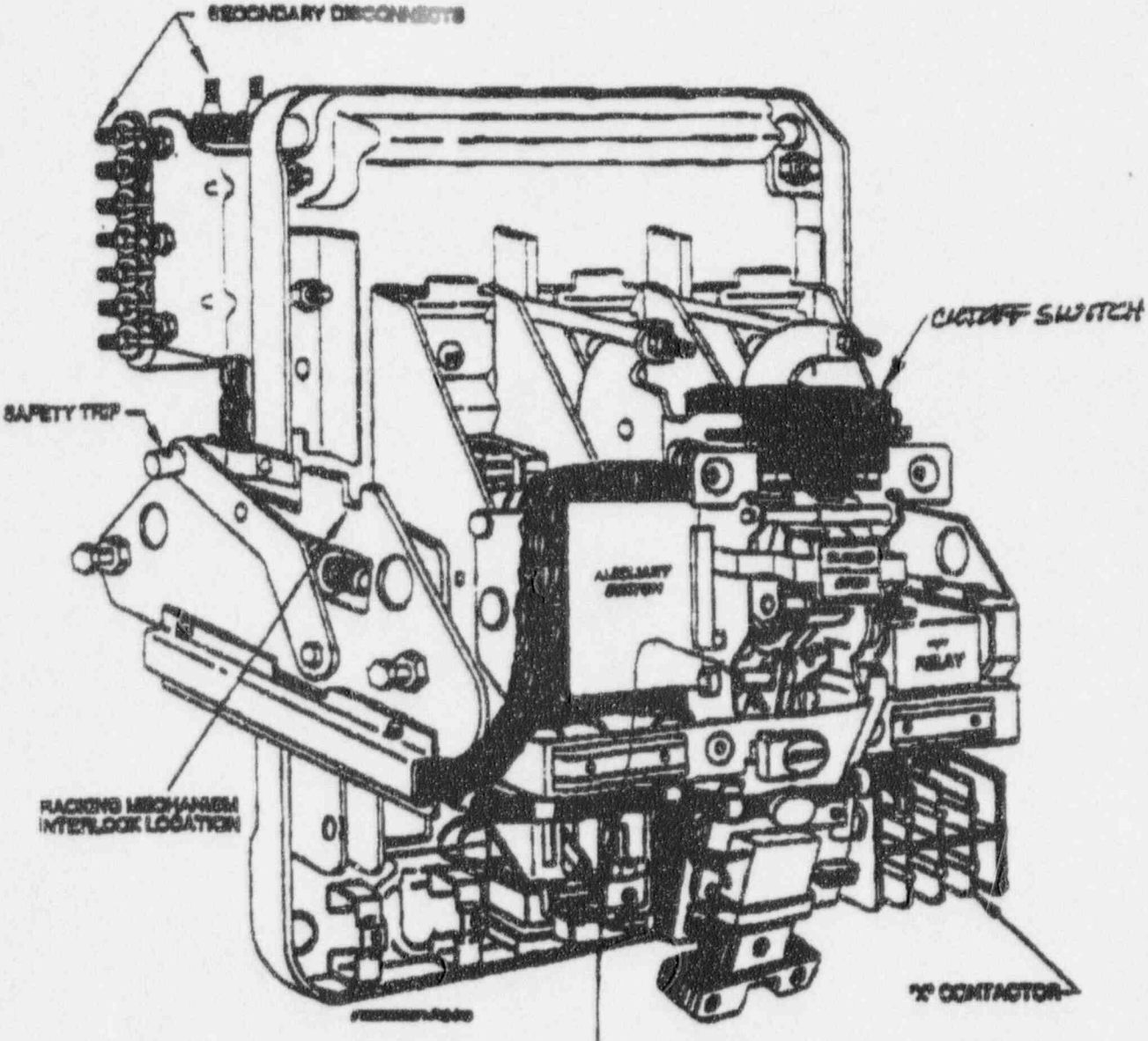


FIGURE 10.1-1 SIMPLIFIED RPS

BRIEFING 94-25
ST. LUCIE
FIGURE 2

ST. LUCIE PLANT
MAINTENANCE PROCEDURE NO. 0920071, REVISION 2
PERIODIC MAINTENANCE OF REACTOR TRIP SWITCHGEAR AND BREAKERS

FIGURE 12
AK-2 BREAKER FRONT VIEW



(CUTOFF SWITCH P/N 622C505G1)

PIECE OF CUTOFF SWITCH BROKEN DOWN TO BOTTOM OF THE TRIP LATCH PREVENTING THE LATCH FROM ACTUATING

March 23, 1994

ST LUCIE

Briefing Paper for April 7, 1994, Senior Management Briefing

A. Current Plant Status

Unit 1 has been operating at power for 71 days, with the next refueling outage scheduled to begin in October 1994.

Unit 2 is in day 38 of a scheduled 45 day refueling outage.

B. Management and Organizational Changes

There have been no major management changes since the last Senior Management Meeting.

C. Plant Performance

The units ran fairly well in 1993. There were five reactor trips and one TS required shutdown in 1993:

May 21, 1993 - Manual trip of Unit 2 in response to seven CEAs dropping into the core due to electrical faults.
May 30, 1993 - TS-required shutdown of Unit 1 due to unlatched CEA discovered during post-refueling startup physics testing.
September 18, 20, and 22, 1993 - Three manual trips of Unit 1 in response to jellyfish clogging intake screens.
November 2, 1993 - Manual trip of Unit 2 in response to increasing main generator hydrogen temperature.

There has also been one reactor trip in 1994:

January 9, 1994 - Manual trip of Unit 1 in response to a main feed pump trip due to a control circuit failure.

On February 17, 1994, the St. Lucie SALP presentation was conducted at the site. St. Lucie received Category 1 ratings in all functional areas for the second consecutive time.

On March 16, 1994, during the Unit 2 refueling outage, the licensee discovered boron residue under one of the four pressurizer steam space instrument nozzles. Dye penetrant and eddy current testing revealed indications in three of the four nozzle-to-pressurizer welds inside the pressurizer. These are the same 3/4-inch pipe diameter instrument nozzles that were repaired one year ago. At that time, the Inconel 600 nozzle pipe was replaced with Inconel 690.

The licensee is planning to install a one-cycle Code repair to all four nozzles. That repair includes: analysis of the existing indications with fracture mechanics and leaving the existing nozzle-to-pressurizer

welds in place; cutting off all but a stub of the existing nozzles; and rewelding the existing nozzles to the outside of the pressurizer. These repairs will extend the refueling outage by about two weeks. The licensee needs NRC approval to use Inconel 690 equivalent weld material and to leave the existing nozzle-to-pressurizer welds in place.

D. Performance Indicators

St. Lucie performance indicators indicate above average performance of both units.

E. Enforcement History

There have been no escalated enforcement actions in 1993 or 1994.

F. SALP

	Period ended May 1992	Period ended January 1994
Operations	1	1
Maintenance	1	1
Engineering	1	1
Rad. Con.	1	
Security	1	
Emerg. Prep.	1	
SA/QV	1	
Plant Support		1

G. INPO

INPO assessment March 1992 - Category 1
Next INPO assessment scheduled for April 1994

H. 1993 Precursor Events

None

I. Allegations and DOL Cases

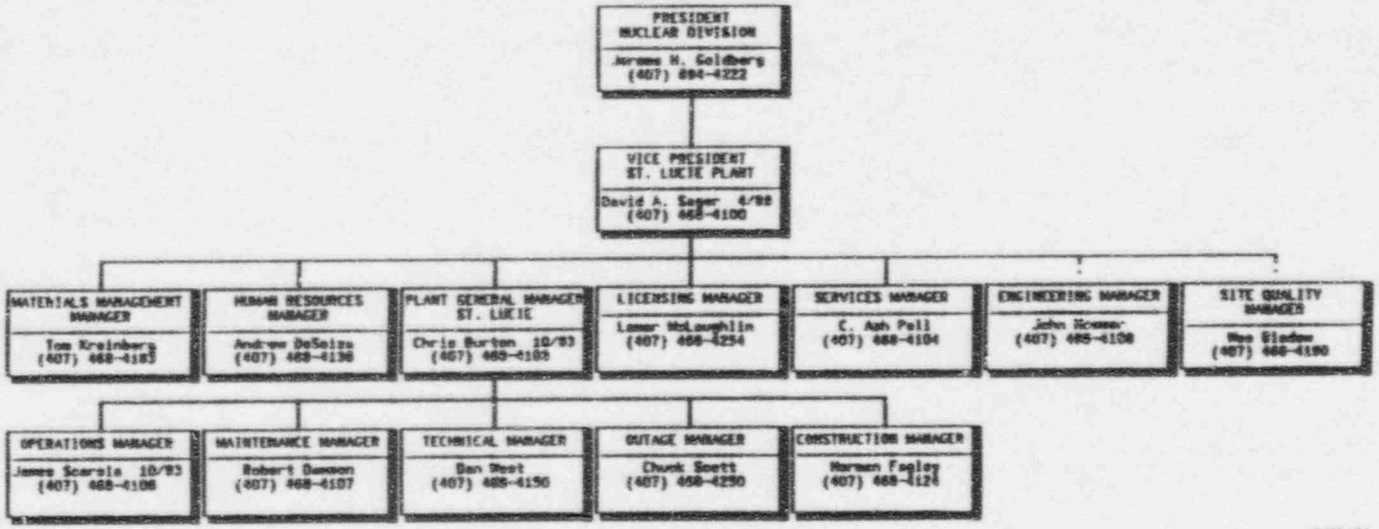
Two allegations are open: one drawing control and one alleged drug use by a contract employee.

One DOL case is open: discrimination - DOL action pending

J. Attachments

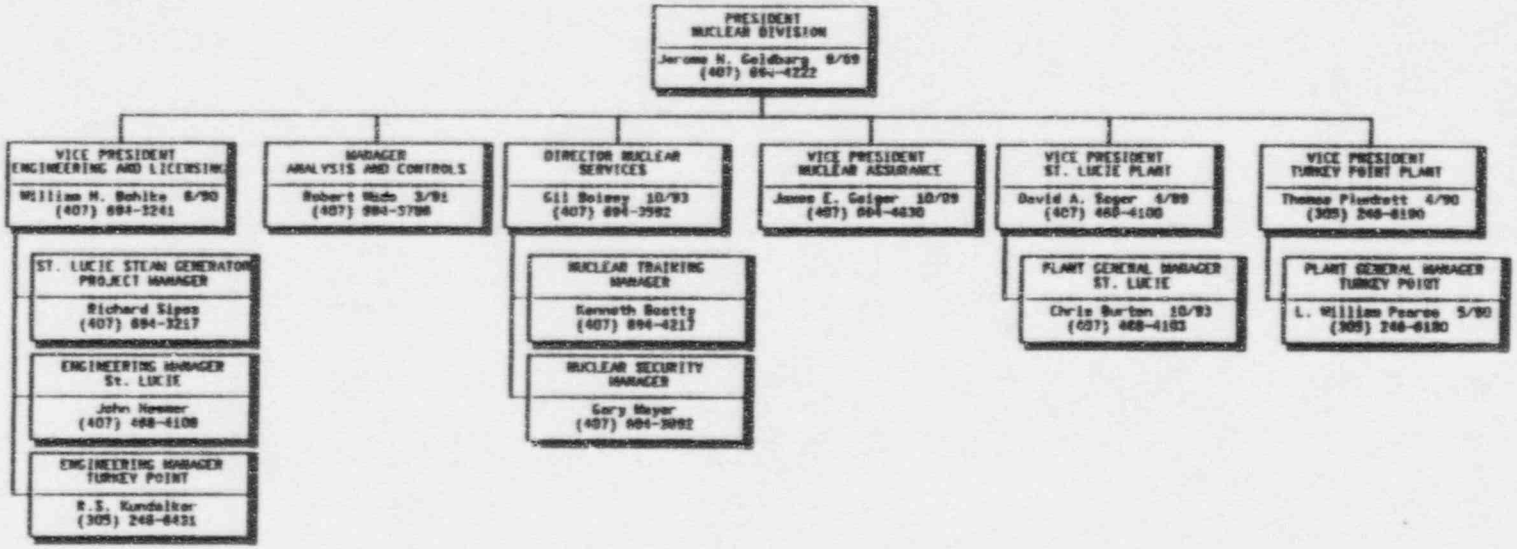
1. Organization Charts
2. Power profiles
3. Plant Status Report

FLORIDA POWER AND LIGHT COMPANY
ST. LUCIE UNITS 1 & 2



3/29/84

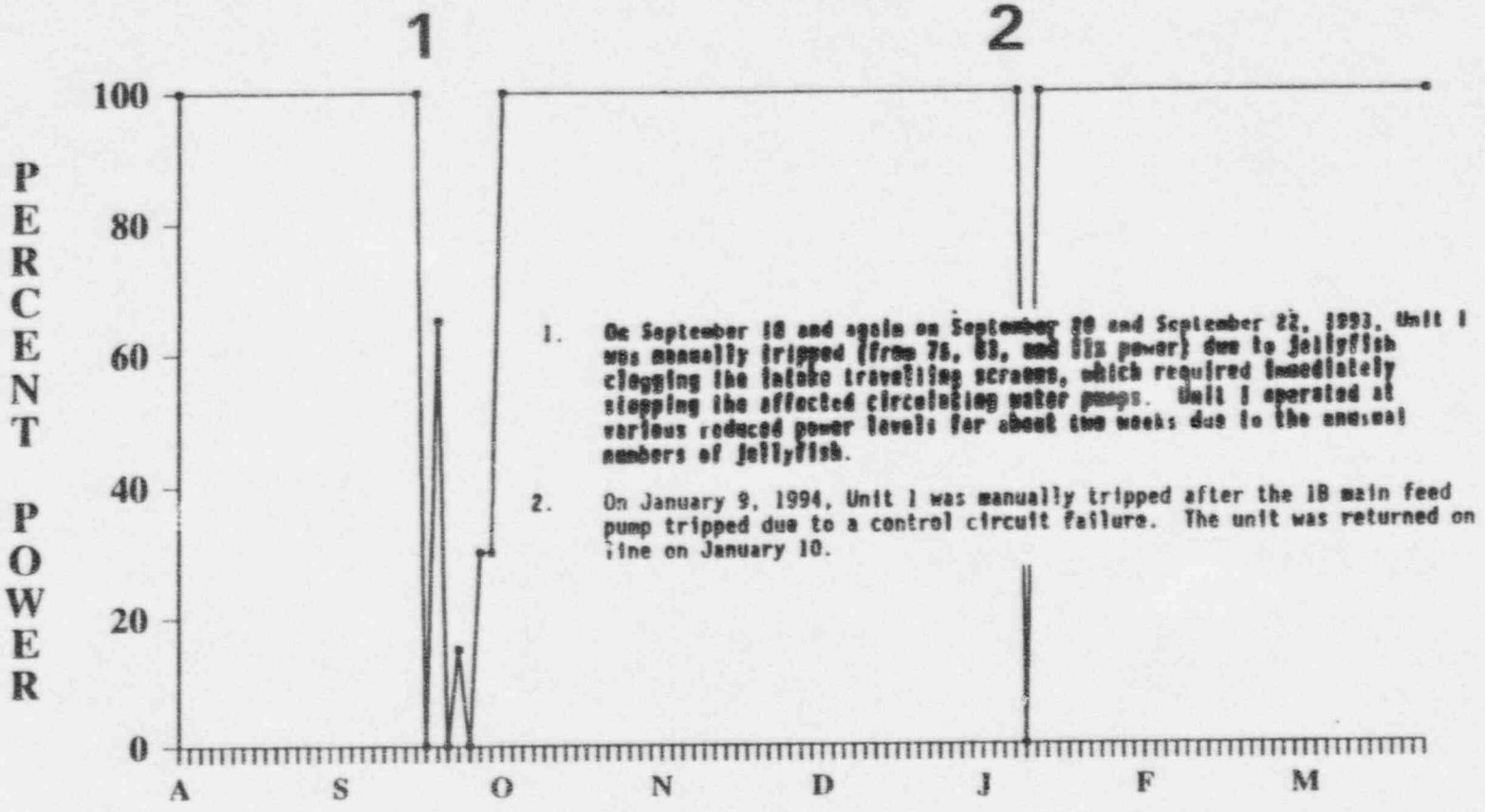
FLORIDA POWER AND LIGHT
MANAGEMENT OVERVIEW



3/24/94

ST. LUCIE 1

Operational Period August 1, 1993 through March 23, 1994



1. On September 18 and again on September 20 and September 22, 1993, Unit 1 was manually tripped (from 75, 83, and 81% power) due to jellyfish clogging the intake travelling screens, which required immediately stopping the affected circulating water pumps. Unit 1 operated at various reduced power levels for about two weeks due to the unusual numbers of jellyfish.

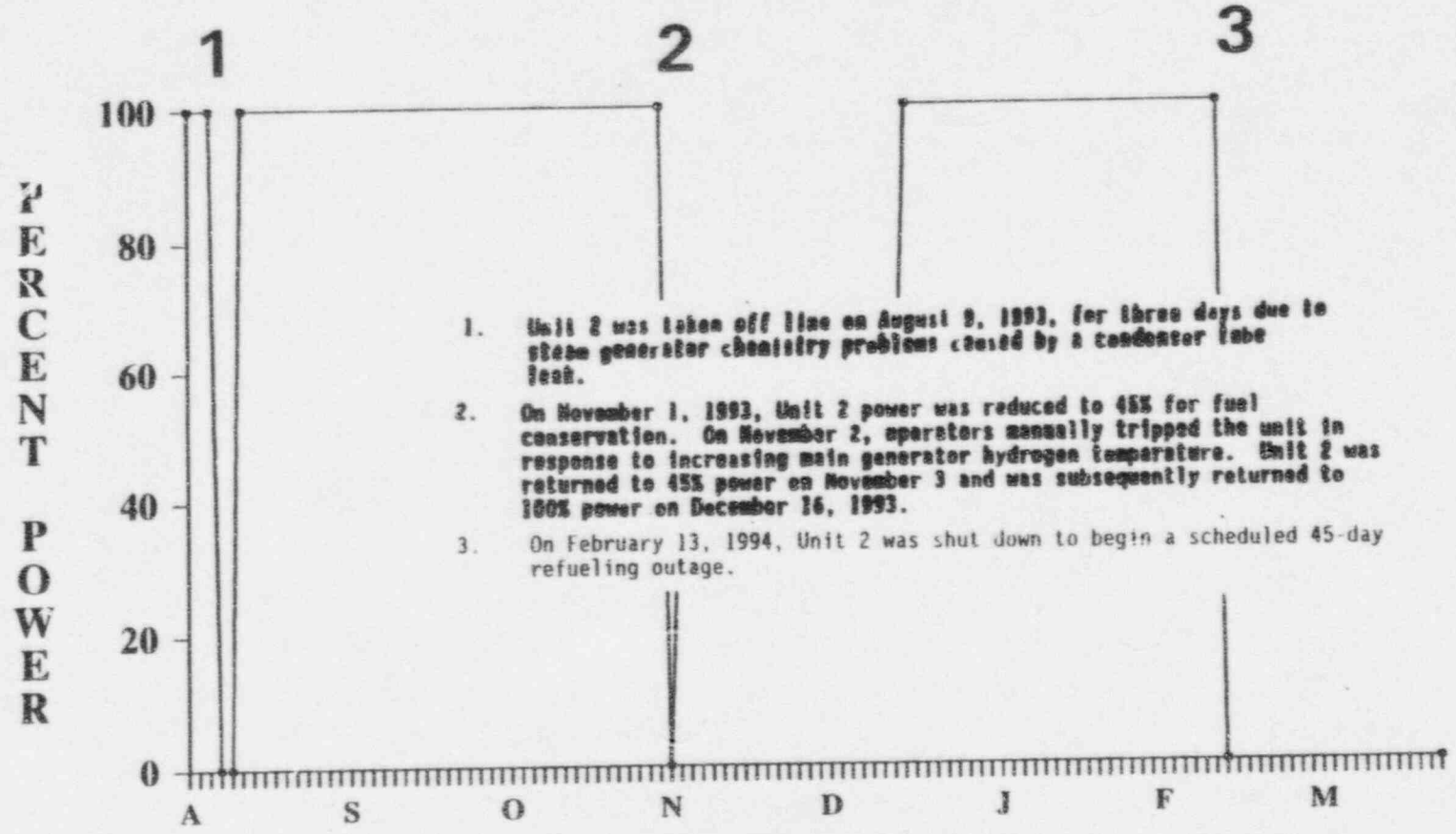
2. On January 9, 1994, Unit 1 was manually tripped after the 1B main feed pump tripped due to a control circuit failure. The unit was returned on line on January 10.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

ST. LUCIE 2

Operational Period August 1, 1993 through March 23, 1994



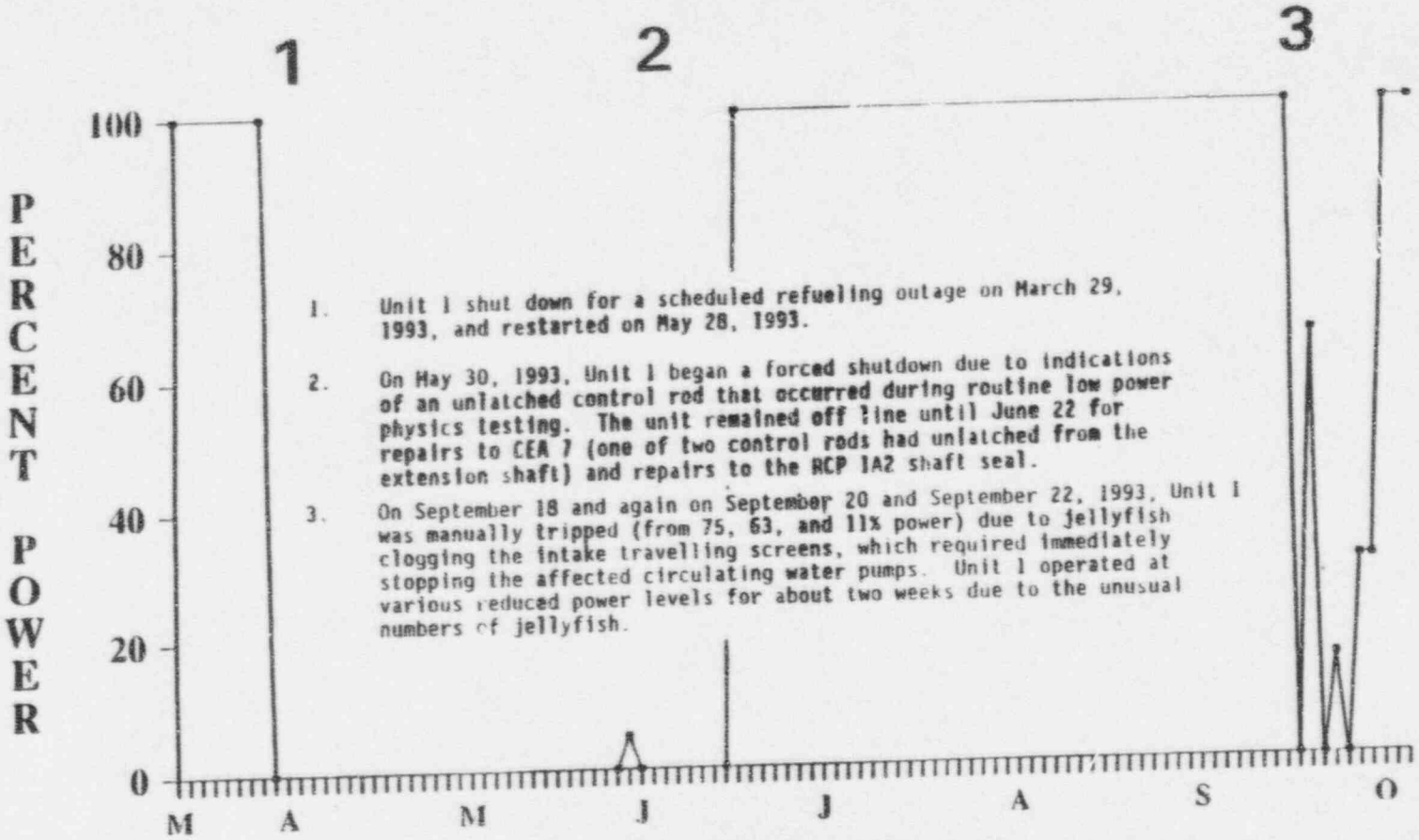
1. Unit 2 was taken off line on August 9, 1993, for three days due to steam generator chemistry problems caused by a condenser tube leak.
2. On November 1, 1993, Unit 2 power was reduced to 45% for fuel conservation. On November 2, operators manually tripped the unit in response to increasing main generator hydrogen temperature. Unit 2 was returned to 45% power on November 3 and was subsequently returned to 100% power on December 16, 1993.
3. On February 13, 1994, Unit 2 was shut down to begin a scheduled 45-day refueling outage.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

ST. LUCIE 1

Operational Period March 13, 1993 through October 6, 1993



1. Unit 1 shut down for a scheduled refueling outage on March 29, 1993, and restarted on May 28, 1993.

2. On May 30, 1993, Unit 1 began a forced shutdown due to indications of an unlatched control rod that occurred during routine low power physics testing. The unit remained off line until June 22 for repairs to CEA 7 (one of two control rods had unlatched from the extension shaft) and repairs to the RCP 1A2 shaft seal.

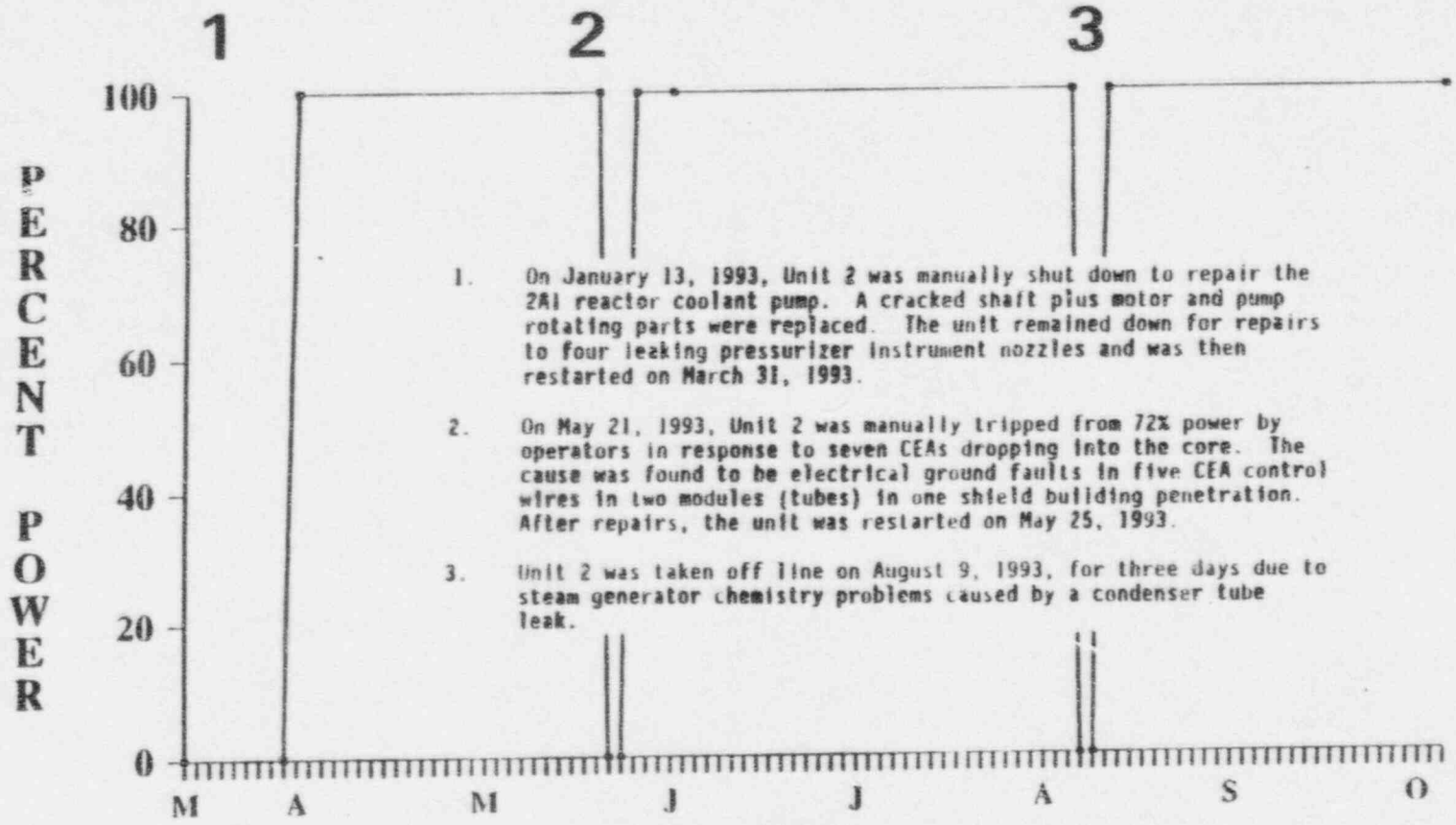
3. On September 18 and again on September 20 and September 22, 1993, Unit 1 was manually tripped (from 75, 63, and 11% power) due to jellyfish clogging the intake travelling screens, which required immediately stopping the affected circulating water pumps. Unit 1 operated at various reduced power levels for about two weeks due to the unusual numbers of jellyfish.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.

ST. LUCIE 2

Operational Period March 13, 1993 through October 6, 1993



1. On January 13, 1993, Unit 2 was manually shut down to repair the 2A1 reactor coolant pump. A cracked shaft plus motor and pump rotating parts were replaced. The unit remained down for repairs to four leaking pressurizer instrument nozzles and was then restarted on March 31, 1993.

2. On May 21, 1993, Unit 2 was manually tripped from 72% power by operators in response to seven CEAs dropping into the core. The cause was found to be electrical ground faults in five CEA control wires in two modules (tubes) in one shield building penetration. After repairs, the unit was restarted on May 25, 1993.

3. Unit 2 was taken off line on August 9, 1993, for three days due to steam generator chemistry problems caused by a condenser tube leak.

PERIOD OF OPERATION

Graph does not include power reductions for routine repairs, waterbox cleaning, or required repairs.