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Rick J. King Director Nuclear Safety & Regulatory Affairs

June 17, 1997

U.S. Nuclear Regulatory Commission Document Control Desk, OP1-17 Washington, DC 20555

Subject:

River Bend Station - Unit I

Docket No. 50-458 License No. NPF-47

Response Supplement 1 to NRC Generic Letter 96-06

File Nos.:

G9.5, G9.33.4

RBF1-97-0189 RBG-44001

Ladies and Gentlemen:

As requested by your staff, River Bend Station (RBS) herein provides the attached information which supplements our original Generic Letter (GL) 96-06 response dated January 28, 1997. This supplement provides a summary of our intended corrective actions for the conditions described in the original response.

GL 96-06 notified licensees of conditions that could affect containment integrity and equipment operability during certain accident conditions. These conditions included, in part, overpressurization of piping systems that penetrate the containment due to thermal expansion of process fluid. In addition to the generic letter requirements associated with containment penetrations, RBS evaluated drywell penetrations for similar conditions.

As discussed in our original response to the generic letter, one containment and eight drywell piping penetrations were susceptible to potential overpressurization. The containment penetration condition will be resolved by the installation of an ASME code qualified pressure relieving device. This installation will be completed during Refueling Outage 7 which is scheduled to begin September 12, 1997.

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PDR AUGCK 05000458



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The drywell penetration condition was evaluated further and it was concluded that the current plant configuration meets the drywell design and safety function requirements. Potential hardware modifications associated with the penetration piping were evaluated but due to the drywell penetration configuration, any hardware modification would either adversely impact system function during normal operation or create an additional drywell leakage path during post accident conditions. However, associated plant design and testing documents will be updated to include any postulated drywell leakage for the eight penetrations (nominal piping sizes 0.5 to 1.0 inches). This approach is consistent with the RBS design specifications and the contribution to overall drywell leakage is well within the current Technical Specification and analytical limits established for allowable drywell bypass leakage.

If you have any questions or require additional information, please contact Rick McAdams at (504) 336-6224.

Sincerely,

RJK\RMM\kvm

attachment

cc: Mr. David L. Wigginton

U. S. Nuclear Regulatory Commission

M/S OWFN 13-H-3

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