

May 15, 1997

Mr. H. L. Sumner, Jr.
Vice President
Southern Nuclear Operating
Company, Inc.
P. O. Box 1295
Birmingham, Alabama 35201-1295

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION ON NRC BULLETIN 96-03 "POTENTIAL PLUGGING OF EMERGENCY CORE COOLING SUCTION STRAINERS BY DEBRIS IN BOILING-WATER REACTORS" - EDWIN I. HATCH NUCLEAR PLANT, UNITS 1 AND 2 (TAC NOS. M96148 AND M96149)

Dear Mr. Sumner:

By letters dated August 30 and October 1, 1996, and March 25, 1997, you responded to NRC Bulletin 96-03 regarding the above subject. Based on our review, we find that additional information, in response to the comments outlined in the enclosure, is needed to enable the NRC staff to complete its review of your responses for Hatch Units 1 and 2. These comments were discussed with your staff via a telephone conference on May 13, 1997.

Please provide the requested information as soon as possible in order to allow sufficient time for the staff to review the requested information on a schedule compatible with your requested date. If you have any questions, please contact me at (301) 415-1496.

Sincerely,

ORIGINAL SIGNED BY J. ZIMMERMAN FOR:

Kahtan N. Jabbour, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

Enclosure: Request For Additional Information

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

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Sincerely,

Kathan M. Jabbour, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket Nos. 50-321 and 50-366

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cc w/encl: See next page

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Units 1 and 2

cc:

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REQUEST FOR ADDITIONAL INFORMATION

ON NRC BULLETIN 96-03 "POTENTIAL PLUGGING OF EMERGENCY CORE
COOLING SUCTION STRAINERS BY DEBRIS IN BOILING-WATER REACTORS"

DOCKET NOS. 50-321 AND 50-366

1. Your estimates of debris generated/transported do not include either calcium silicate or urethane. Please provide your justification for not including them.
2. Have you conducted parametric evaluations to demonstrate that the head loss is maximized for the selected break? Your March 25, 1997, submittal appears to focus entirely on maximizing the quantity of fibrous insulation.
3. Please describe the transportable materials, and discuss how they were incorporated into the head loss estimate?
4. Please discuss whether or not you intend to have a dedicated strainer for each pump such that one residual heat removal strainer and two core spray strainers can handle 100 percent of all debris generated/transported? If not, do you intend to use a common header? Please provide sufficient details of your design.
5. Using the generic methodology provided in the General Electric report as applied to Hatch, describe the characteristics, including the sizes, of the strainers proposed for use at Hatch. What is the actual size of the strainer used in the GE testing? In particular, please provide details regarding the crevices (troughs), etc.
6. What is the fraction of the insulation contained below the grating in the "worst-case" break?
7. During a call with the licensee on May 13, 1997, the licensee stated that it will maintain the margin with regards to minimum containment pressure available and low-pressure coolant injection (LPCI) net positive suction head required. This is described in the Hatch Unit 1 Final Safety Analysis Report (FSAR) on page 14.4-15. However, this statement is not consistent to the statements with regard to the use of containment overpressure on page E-2 of the March 25, 1997, submittal. In particular, the staff's licensing safety evaluation and the FSAR for Hatch, Unit 1, indicate that credit was taken for 1 psig of containment overpressure for the LPCI pumps, and that no overpressure was taken for the core spray pumps. Please reconcile the docketed information with the information provided in the phone conference.

Enclosure