



Northern States Power Company

Prairie Island Nuclear Generating Plant

1717 Wakonade Dr. East
Welch, Minnesota 55089

May 6, 1997

10 CFR Part 50
Section 50.55a

U S Nuclear Regulatory Commission
Attn: Document Control Desk
Washington, DC 20555

PRAIRIE ISLAND NUCLEAR GENERATING PLANT

Docket Nos. 50-282 License Nos. DPR-42
50-306 DPR-60

Steam Generator Weld Indication Evaluation

During the Unit 2 refueling outage in Winter 1997, ultrasonic examinations of the steam generators were performed in accordance with ASME Boiler and Pressure Vessel Code Section XI. Examinations were done to satisfy the first period requirements of the third ten-year interval. The third ten-year interval plan for Prairie Island was written to conform to the 1989 edition of ASME Section XI.

During the examinations, indications were identified in the tubesheet to channel head weld region for #22 steam generator (see attached cover sheets to examination report #97-0136). These indications exceed the allowable flaw size when evaluated against the standards provided in ASME Section XI, IWB-3500. Accordingly, we performed an analytical evaluation of each of these flaws per ASME Section XI, IWB-3610. These flaws were found acceptable per these analyses. Attached for your review are the results of these evaluations. The procedure used for these evaluations is contained in WCAP-14166 which we submitted for review in January 1995.

In this letter we have made no new Nuclear Regulatory Commission commitments.

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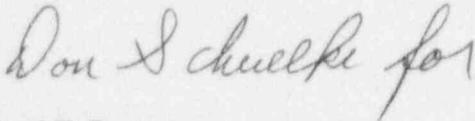
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Please contact Jack Leveille (612-388-1121, Ext. 4662) if you have any questions related to this letter.



Joel P Sorensen
Plant Manager
Prairie Island Nuclear Generating Plant

c: Regional Administrator - Region III, NRC
Senior Resident Inspector, NRC
NRR Project Manager, NRC
J E Silberg

Attachment: Ultrasonic Examination Reports #97-0136 and #97-0136 R1