



**Wisconsin
Electric**
POWER COMPANY

Point Beach Nuclear Plant
6610 Nuclear Rd., Two Rivers, WI 54241

(414) 755-2321

NPL 97-0318

June 5, 1997

Document Control Desk
U. S. NUCLEAR REGULATORY COMMISSION
Mail Station P1-137
Washington, DC 20555

Ladies/Gentlemen:

DOCKETS 50-266 AND 50-301
MONTHLY OPERATING REPORTS
POINT BEACH NUCLEAR PLANT, UNITS 1 AND 2

Attached are monthly operating reports for Units 1 and 2 of the Point Beach Nuclear Plant for the calendar month of May 1997.

Sincerely,

D. F. Johnson
Manager -
Regulatory Services & Licensing

FE04/1

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Attachments

cc: L. L. Smith, PSCW
NRC Regional Administrator, Region III
NRC Resident Inspector



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PDR ADOCK 05000266
R PDR

OPERATING DATA REPORT

DOCKET NO. 50-266

DATE: 06/02/97

COMPLETED BY: M. B. Koudelka

TELEPHONE: 414-755-6480

OPERATING STATUS

- | | | |
|--|---|-------|
| 1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 1 | . | NOTES |
| 2. REPORTING PERIOD: May - 1997 | . | |
| 3. LICENSED THERMAL POWER (MWT): 1518.5 | . | |
| 4. NAMEPLATE RATING (GROSS MWE): 523.8 | . | |
| 5. DESIGN ELECTRICAL RATING (NET MWE): 497.0 | . | |
| 6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0 | . | |
| 7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0 | . | |
| 8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS: | | |
| NA | | |
| 9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA | | |
| 10. REASONS FOR RESTRICTIONS, (IF ANY): | | |
| NA | | |

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	3,623.0	232,895.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	1,173.6	194,047.5
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	667.3
14. HOURS GENERATOR ON LINE	0.0	1,151.0	190,806.0
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	846.9
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	1,558,331.0	271,052,716.0
17. GROSS ELECTRICAL ENERGY GENERATED	0.0	534,720.0	91,625,810.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.0	494,009.0	87,350,428.5
19. UNIT SERVICE FACTOR	0.0%	31.8%	81.9%
20. UNIT AVAILABILITY FACTOR	0.0%	31.8%	82.3%
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0%	28.1%	77.0%
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0%	27.4%	75.5%
23. UNIT FORCED OUTAGE RATE	100.0%	68.2%	2.7%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):			
Refueling, 09/13/97, 49 days			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:			
07/18/97			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POINT BEACH NUCLEAR PLANT

DOCKET NO. 50-266

AVERAGE DAILY UNIT POWER LEVEL

UNIT NAME Point Beach, Unit 1

MONTH MAY - 1997

DATE 6/2/97

COMPLETED BY M. B. Koudelka

TELEPHONE 414-755-6480

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-2</u>	11	<u>-2</u>	21	<u>-2</u>
2	<u>-2</u>	12	<u>-2</u>	22	<u>-2</u>
3	<u>-2</u>	13	<u>-2</u>	23	<u>-2</u>
4	<u>-2</u>	14	<u>-2</u>	24	<u>-2</u>
5	<u>-2</u>	15	<u>-2</u>	25	<u>-2</u>
6	<u>-2</u>	16	<u>-2</u>	26	<u>-2</u>
7	<u>-2</u>	17	<u>-2</u>	27	<u>-2</u>
8	<u>-2</u>	18	<u>-2</u>	28	<u>-2</u>
9	<u>-2</u>	19	<u>-2</u>	29	<u>-2</u>
10	<u>-2</u>	20	<u>-2</u>	30	<u>-2</u>
				31	<u>-2</u>

POINT BEACH NUCLEAR PLANT
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY - 1997

Docket No. 50-266
 Unit Name Point Beach, Unit 1
 Date 6/2/97
 Completed by M. B. Koudelka
 Telephone No. 414/755-6480

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Reactor Shut Down ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
2	970218	F	744	H	1	NA	NA	NA	The unit was in hot shutdown and taken to cold shutdown on March 15, 1997.

¹F: Forced
 S: Scheduled

²Reason:
 A - Equipment Failure (explain)
 B - Maintenance or Testing
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Exam
 F - Administrative
 G - Operational Error (explain)
 H - Other (explain)

³Method:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Continuation of Previous Shutdown
 5 - Reduced Load
 6 - Other (explain)

⁴Exhibit G - Instructions for preparation of data entry sheets LER file (NUREG-0161)

⁵Exhibit I - Same Source

DOCKET NO. 50-266
UNIT NAME Point Beach Unit 1
DATE 06/04/97
COMPLETED BY M. B. Koudelka
TELEPHONE 414/755-6480

Unit 1 was in cold shutdown during the May, 1997 report period.

The following Licensee Event Reports were submitted to the NRC during May, 1997:

Licensee Event Report 50-266/97-013-01, Component Cooling Water System not in Accordance with Plant Design Basis.

Licensee Event Report 50-266/97-017-00, Containment Hatch Temporary Third Door Blocked Open During Refueling Operations.

Licensee Event Report 50-266/97-018-00, Potential Residual Heat Removal System Overpressure During Accident Conditions.

Licensee Event Report 50-266/97-019-00, Residual Heat Removal not Aligned in Accordance with Technical Specifications Operability Requirements.

Licensee Event Report 50-266/97-020-00, Conditions Outside 10 CFR 50 Appendix R Safe Shutdown Analysis.

Licensee Event Report 50-266/97-021-00, Spent Fuel Pool Cooling System not in Accordance with Plant Design.

Safety-related maintenance included:

1. AF-4007 and AF-4014 P-38A&B auxiliary feedwater pump mini-recirculation control valve relay calibration check.
2. 1AF-4002 P-29 auxiliary feedwater pump mini-recirculation control valve relay calibration check.
3. 1P-1B reactor coolant pump had oil added to the upper oil pot.
4. 1SW-2907 HX-15A-D containment recirculation heat exchanger emergency flow control valve replaced.
5. CC-722B 1P-11A&B component cooling pump discharge cross-connect valve stem inspected.
6. SW-2832B-S K-3B service air compressor inlet solenoid valve replaced.

OPERATING DATA REPORT

DOCKET NO. 50-301

DATE: 06/02/97

COMPLETED BY: M. B. Koudelka

TELEPHONE: 414-755-6480

OPERATING STATUS

1. UNIT NAME: POINT BEACH NUCLEAR PLANT - UNIT 2
2. REPORTING PERIOD: May - 1997
3. LICENSED THERMAL POWER (MWT): 1518.5
4. NAMEPLATE RATING (GROSS MWE): 523.8
5. DESIGN ELECTRICAL RATING (NET MWE): 497.0
6. MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 509.0
7. MAXIMUM DEPENDABLE CAPACITY (NET MWE): 485.0
8. IF CHANGES OCCUR IN CAPACITY RATINGS (ITEMS 3 THROUGH 7) SINCE LAST REPORT, GIVE REASONS:
NA
9. POWER LEVEL TO WHICH RESTRICTED, IF ANY (NET MWE): NA
10. REASONS FOR RESTRICTIONS, (IF ANY):
NA

NOTES

	THIS MONTH	YEAR TO DATE	CUMULATIVE
11. HOURS IN REPORTING PERIOD	744.0	3,623.0	217,680.0
12. NUMBER OF HOURS REACTOR WAS CRITICAL	0.0	0.0	185,987.6
13. REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	233.9
14. HOURS GENERATOR ON LINE	0.0	0.0	183,508.3
15. UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	302.2
16. GROSS THERMAL ENERGY GENERATED (MWH)	0.0	0.0	263,605,007.0
17. GROSS ELECTRICAL ENERGY GENERATED	0.0	0.0	89,603,310.0
18. NET ELECTRICAL ENERGY GENERATED (MWH)	0.0	(8,429.0)	85,403,154.5
19. UNIT SERVICE FACTOR	0.0%	0.0%	84.3%
20. UNIT AVAILABILITY FACTOR	0.0%	0.0%	84.4%
21. UNIT CAPACITY FACTOR (USING MDC NET)	0.0%	-0.5%	80.4%
22. UNIT CAPACITY FACTOR (USING DER NET)	0.0%	-0.5%	78.9%
23. UNIT FORCED OUTAGE RATE	0.0%	0.0%	0.9%
24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): None			
25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: 06/25/97			

DATA REPORTED AND FACTORS CALCULATED AS REQUESTED IN NRC LETTER DATED SEPTEMBER 22, 1977

POINT BEACH NUCLEAR PLANT

DOCKET NO. 50-301

AVERAGE DAILY UNIT POWER LEVEL

UNIT NAME Point Beach, Unit 2

MONTH MAY - 1997

DATE 6/2/97

COMPLETED BY M. B. Koudelka

TELEPHONE 414-755-6480

<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>	<u>DAY</u>	<u>AVERAGE DAILY POWER LEVEL MWe NET</u>
1	<u>-3</u>	11	<u>-3</u>	21	<u>-3</u>
2	<u>-3</u>	12	<u>-3</u>	22	<u>-3</u>
3	<u>-3</u>	13	<u>-3</u>	23	<u>-3</u>
4	<u>-3</u>	14	<u>-3</u>	24	<u>-3</u>
5	<u>-3</u>	15	<u>-3</u>	25	<u>-3</u>
6	<u>-3</u>	16	<u>-3</u>	26	<u>-3</u>
7	<u>-3</u>	17	<u>-3</u>	27	<u>-3</u>
8	<u>-3</u>	18	<u>-3</u>	28	<u>-3</u>
9	<u>-3</u>	19	<u>-3</u>	29	<u>-3</u>
10	<u>-3</u>	20	<u>-3</u>	30	<u>-3</u>
				31	<u>-3</u>

POINT BEACH NUCLEAR PLANT
UNIT SHUTDOWNS AND POWER REDUCTIONS

REPORT MONTH MAY - 1997

Docket No. 50-301
 Unit Name Point Beach, Unit 2
 Date 6/2/97
 Completed By M. B. Koudelka
 Telephone No. 414/755-6480

No.	Date	Type ¹	Duration (Hours)	Reason ²	Method of Reactor Shut Down ³	Licensee Event Report No.	System Code ⁴	Component Code ⁵	Cause and Corrective Action To Prevent Recurrence
4	961005	S	744	C	1	NA	NA	NA	Scheduled refueling and maintenance outage (U2R22). Major work items include: Replacing both Unit 2 steam generators; main control board wire separation resolution; replace 8 main steam condenser valves; HP turbine crossunder piping; repair service water west header; replace boric acid and reactor makeup water totalizers; replace safety injection accumulator level transmitters; modify X-02, X-03, and X-04 transformer metering and relay circuits; and exciter modification.

¹F: Forced
 S: Scheduled

²Reason:
 A - Equipment Failure (explain)
 B - Maintenance or Testing
 C - Refueling
 D - Regulatory Restriction
 E - Operator Training & Licensing Exam
 F - Administrative
 G - Operational Error (explain)
 H - Other (explain)

³Method:
 1 - Manual
 2 - Manual Scram
 3 - Automatic Scram
 4 - Continuation of Previous Shutdown
 5 - Reduced Load
 6 - Other (explain)

⁴Exhibit G - Instructions for preparation of data entry sheets LER file (NUREG-0161)

⁵Exhibit I - Same Source

DOCKET NO. 50-301
UNIT NAME Point Beach Unit 2
DATE 06/04/97
COMPLETED BY M. B. Koudelka
TELEPHONE 414/755-6480

Unit 2 was in a refueling outage during the entire May, 1997 report period.

The following Licensee Event Report was submitted to the NRC during May, 1997:

Licensee Event Report 50-301/97-002-00, Potential Reactor Coolant System Branch Connection Stresses Beyond Design Basis.

Safety-related maintenance included:

1. 2CV-200C-S letdown orifice "C" outlet control solenoid valve replaced.
2. 2AF-4002 P-29 auxiliary feedwater pump mini-recirculation control valve relay calibration check.
3. 2B52-325J power to 2CV-112C volume control tank outlet breaker overload reset replaced.
4. 2P-1A&B-M reactor coolant pump motor megged.
5. 2RH-709A P-10A residual heat removal pump discharge body gasket replaced.
6. 2CC-D-15 HX-8B boric acid evaporator component coolant supply drain valve replaced.