

# UNITED STATES NUCLEAR REGULATORY COMMISSION WASHINGTON, D. C. 20555

DUQUESNE LIGHT COMPANY

OHIO EDISON COMPANY

PENNSYLVANIA POWER COMPANY

DOCKET NO. 50-334

BEAVER VALLEY POWER STATION, UNIT NO. 1

AMENDMENT TO FACILITY OPERATING LICENSE

Amendment No.100 License No. DPR-66

- 1. The Nuclear Regulatory Commission (the Commission) has found that:
  - A. The application for amendment by Duquesne Light Company, Ohio Edison Company, and Pennsylvania Power Company (the licensees) dated January 2, 1986, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act) and the Commission's rules and regulations set forth in 10 CFR Chapter I;
  - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
  - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
  - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
  - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied.
- Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C.(2) of Facility Operating License No. DPR-66 is hereby amended to read as follows:

# (2) Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No.100 , are hereby incorporated in the license. The licensee shall operate the facility in accordance with the Technical Specifications.

 This amendment is effective on issuance, to be implemented no later than 30 days after issuance.

FOR THE NUCLEAR REGULATORY COMMISSION

Lester S. Rubenstein, Director PWR Project Directorate #2 Division of PWR Licensing-A

Attachment: Changes to the Technical Specifications

Date of Issuance: March 18, 1986

## ATTACHMENT TO LICENSE AMENDMENT

## AMENDMENT NO. 100 TO FACILITY OPERATING LICENSE NO. DPR-66

DOCKET NO. 50-334

Revise Appendix A as follows:

Remove Pages

Insert Pages

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3.7.12 All snubbers shall be OPERABLE. The only snubbers excluded from this requirement are those installed on non-safety-related systems and then only if their failure or failure of the system on which they are installed, would have no adverse effect on any safety-related system.

APPLICABILITY: MODES 1, 2, 3 and 4. (MODES 5 and 6 for snubbers located on systems \*\* required OPERABLE in those MODES).

#### ACTION:

With one or more snubbers inoperable, within 72 hours replace or restore the inoperable snubber(s) to OPERABLE status and perform an engineering evaluation per Specification 4.7.12.c on the supported component or declare the supported system inoperable and follow the appropriate ACTION statement for that system.

#### SURVEILLANCE REQUIREMENTS

4.7.12 Each snubber shall be demonstrated OPERABLE by performance of the following augmented inservice inspection program and the requirements of Specification 4.0.5.

a. Visual Inspections

The first inservice visual inspection of snubbers shall be performed after four months but within 10 months of commencing POWER OPERATION and shall include all snubbers. If less than two (2) snubbers are found inoperable during the first inservice visual inspection, the second inservice visual inspection shall be performed 12 months + 25% from the date of the first inspection. Otherwise, subsequent visual inspections shall be performed in accordance with the following schedule:

No.	Inop	erabl	e Snubbers
per	Ins	pecti	on Period

Ö 3.4 5,6,7 8 or more

#### Subsequent Visual Inspection Period\* #

18 months + 25% 12 months + 25% \*\* 6 months + 25% 124 days ± 25% 62 days ± 25% 31 days ± 25%

The snubbers may be categorized into two groups: those accessible and those inaccessible during reactor operation. Each group may be inspected independently in accordance with the above schedule.

<sup>\*</sup> The inspection interval shall not be lengthened more than one step at a time.

<sup>\*</sup> The provisions of Specification 4.0.2 are not applicable.

<sup>\*\*</sup> These systems are defined as those portions or subsystems required to prevent releases in excess of 10 CFR 100 limits.

<sup>\*\*</sup> A one-time extension is granted to the above 12 month + 25% schedule which resulted from the fourth refueling inspection activities. The visual inspections required following the fourth refueling outage will be performed during the fifth refueling outage. This extension expires upon startup from the fifth refueling outage.