

SNUPPS

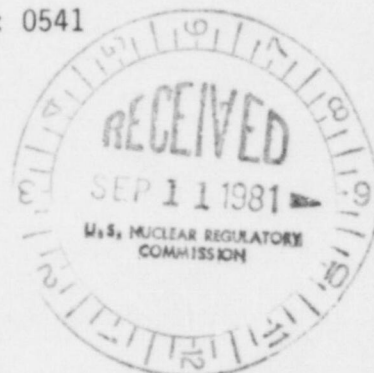
Standardized Nuclear Unit  
Power Plant System

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Nicholas A. Petrick  
Executive Director

September 4, 1981

SLNRC 81-86 FILE: 0541  
SUBJ: ETSB Review



Mr. Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
U.S. Nuclear Regulatory Commission  
Washington, D.C. 20555

Docket Nos.: STN 50-482, STN 50-483, and STN 50-486

Dear Mr. Denton:

The referenced letter provided FSAR changes that were needed by the Effluent Treatment Systems Branch in order to complete their review of the SNUPPS FSAR. In discussions with Dr. Gordon Edison, NRC project manager for the SNUPPS applications, it was learned that additional information is required. The enclosure to this letter is a change to the information provided in the reference. The enclosure and the other changes provided in the reference will be incorporated in the next revision of the SNUPPS FSAR.

Very truly yours,

*N.A. Petrick*

Nicholas A. Petrick

RLS/dck/3a24

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## SNUPPS

In view of the fact that these systems are designed to preclude operation during an accident, upgrading the testing and surveillance of these systems is not justified.

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### 18.3.4.3 Conclusion

The SNUPPS design includes provisions to insure the integrity of fluids systems which are postulated to contain highly contaminated fluids following a design basis accident. The provision will be based on the preservice and inservice tests required by the ASME Code. These provisions provide assurance that these systems will perform their intended functions, including leaktightness, following a design basis accident. This commitment satisfies Item III.D.1.1 of NUREG-0737.

### 18.3.5 IMPROVED IN-PLANT IODINE INSTRUMENTATION UNDER ACCIDENT CONDITIONS (II.D.3.3)

Refer to each Site Addendum.

### 18.3.6 CONTROL ROOM HABITABILITY (III.D.3.4)

#### 18.3.6.1 NRC Guidance per NUREG-0737

##### Position

In accordance with Task Action Plan Item III.D.3.4 and control room habitability, licensees shall ensure that control room operators will be adequately protected against the effects of accidental release of toxic and radioactive gases and that the nuclear power plant can be safely operated or shut down under design basis accident conditions (Criterion 19, "Control Room," of Appendix A, "General Design Criteria for Nuclear Power Plants," to 10 CFR Part 50).

##### Clarification

- (1) All licensees must make a submittal to the NRC regardless of whether or not they met the criteria of the referenced Standard Review Plans (SRP) sections. The new clarification specifies that licensees that meet the criteria of the SRPs should provide the basis for their conclusion that SRP 6.4 requirements are met. Licensees may establish this basis by referencing past submittals to the NRC and/or providing new or additional information to supplement past submittals.
- (2) All licensees with control rooms that meet the criteria of the following sections of the Standard Review Plan:
  - 2.2.1-2.2.2 Identification of Potential Hazards in Site Vicinity,
  - 2.2.3 Evaluation of Potential Accidents, and
  - 6.4 Habitability Systems

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The SNUPPS Utilities will have a program to reduce leakage from systems outside containment that would or could contain highly radioactive fluids in a post-accident situation. The programs will meet Item III.D.1.1 of NUREG-0737 and will include the following:

1. The implementation of all practical leak reduction measures for all systems that could carry radioactive fluid outside of containment.
2. The measurement of actual leakrates with the systems in operation and the reporting of the results to the NRC.
3. A continuing leak reduction program of preventive maintenance to reduce leakage to as-low-as-practical levels. This program will include periodic integrated leak tests at intervals not to exceed each refueling cycle.

Detailed information on the program and a list of any excluded systems and their justification for exclusion will be provided at least four months prior to fuel load of each SNUPPS unit.