NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Waterford Steam Electric Station Unit 3	0 5 0 0 0 3 8 2	1 OF 0 3

TITLE (4)

Reactor Trip Resulting from Condenser Level Perturbation

EVENT DATE (5)			LER NUMBER (6)					REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR		SEQUENTIAL NUMBER		REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
											N/A	0 5 0 0 0
1 2	1 9	8 5	8 5	—	0 5 6	—	0 0	0 1	1 7	8 6	N/A	0 5 0 0 0

OPERATING MODE (9)	1	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
		20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	1 0 0	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
		20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
		20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
		20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
		20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
L. Myers, Operations Superintendent	5 0 4 4 6 4 - 3 1 1 8

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 1443 hours on December 19, 1985, Waterford Steam Electric Station Unit 3 was in mode 1 (power operation) at 100% reactor power when a reactor trip occurred as a result of low water level in both steam generators. The water level in the steam generators decreased when Main Feedwater Pump A (EIIS System Code SJ) tripped due to high water level in the Feedwater Pump Seal Leakoff Tank. The entire event was initiated when plant personnel were performing maintenance on the C Main Condenser (EIIS System Code SG) and inadvertently isolated its Air Evacuation Pump (EIIS System Code SH) suction valve instead of the Circulating Water (EIIS System Code KE) discharge valve. Upon receiving the reactor trip, operations personnel immediately entered procedures OP-902-000, "Emergency Entry Procedure" and OP-902-001, "Uncomplicated Reactor Trip Recovery Procedure." Plant conditions were stabilized in mode 3 (hot standby).

Operations Personnel will be reinstructed in procedure OI-10-000, "Waterford 3 Operations Department Good Operating Practices", particularly those sections dealing with operator errors.

IE22
1/1

☆U.S.GPO:1984-0-454-481/16759

FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
Waterford Steam Electric Station Unit 3	0 5 0 0 0 3 8 2	8 5	— 0 5 6	— 0 0	0 2 OF 0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 1443 hours on December 19, 1985 Waterford Steam Electric Station Unit 3 was at 100% reactor power when Main Feedwater Pump A (EIIS System Code SJ) tripped due to a high water level in the Feedwater Pump Seal Leakoff Tank. Due to the decrease in feedwater flow, the water level in both Steam Generators began to decrease until the low level reactor trip and Emergency Feedwater Actuation setpoint was reached. Upon receiving the reactor trip, operations personnel immediately entered procedures OP-902-000, "Emergency Entry Procedure" and OP-902-001, "Uncomplicated Reactor Trip Recovery Procedure". Plant conditions were stabilized in mode 3 (hot standby).

The event was initiated when operations personnel were attempting to isolate a portion of the waterbox for the C Main Condenser. After isolating the Circulating Water (EIIS System Code KE) intake valve to the waterbox, the operator inadvertently isolated the Air Evacuation Pump (EIIS System Code SH) suction valve to the adjacent portion of the C Condenser hotwell rather than the Circulating Water discharge valve. This caused a decrease in vacuum in the C Main Condenser. Since the A, B, and C Main Condensers are interconnected, a recurring increase and decrease in hotwell water level (or wave motion), as measured in the A Main Condenser, was initiated due to the pressure differential in the three Condenser hotwells. As the water level in the A Condenser decreased, the Condenser hotwell makeup valve opened to compensate for the level decrease. Since the water level fluctuated from high to low (due to the wave motion), the makeup valve was constantly cycling. This recurred until the water level in the hotwell increased enough to overflow into the Feedwater Pump Seal

Leakoff Tank. The water level in the tank increased until the level reached the feedwater pump trip setpoint (this feature prevents water from backing up into the feedwater pump turbine).

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an actuation of the Reactor Protective System in which no primary system parameters were exceeded. Since the Control Element Assemblies (EIIS System Code AA) and the Reactor Protective System (EIIS System Code JC) functioned as designed, the event in no way placed Waterford 3 in a degraded safety condition.

CORRECTIVE ACTION

The Operations Superintendent has instructed all Shift Supervisors to review with their shift personnel all sections to procedure OI-10-000, "Waterford 3 Operations Department Good Operating Practices," particularly those sections dealing with operator errors.

SIMILAR EVENTS

None

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118.

LOUISIANA POWER & LIGHT / 142 DELARONDE STREET
P. O. BOX 6008 • NEW ORLEANS, LOUISIANA 70174 • (504) 366-2345
MIDDLE SOUTH UTILITIES SYSTEM

January 17, 1986

W3P86-0013
A4.05
QA

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-056-00 for Waterford 3. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, J.H. Wilson, NRC Resident Inspectors Office, INPO Records Center (J.T. Wheelock), B.W. Churchill, W.M. Stevenson

IE22
1/1