

From: William Miller, NRC R2
To: ATP1.PEF (P. Fredrickson, NRC R2)
Date: 12/16/96 5:15pm
Subject: St. Lucie -Forwarded

Paul,

During my assignment at St Lucie this summer, I worked on two issues which were turned over to John York for additional follow-up. These issues involved the use of a temporary fire pump without an evaluation and the construction of an enclosure for the control rod drive electronic equipment within the cable spreading room without an appropriate evaluation.

The temporary fire pump was 750 gpm and replaced a 2,500 gpm fire pump. No evaluation had been made to determine if this pump met the required fire flow. The control rod drive enclosure had not been properly evaluated resulting in the requirement for the installation of additional seismic restraints.

Today, I learned that both of these items which were initially identified as apparent violations were dropped and classified as not being a violation to the NRC requirements.

I was not consulted on this matter. I agree that these issues may not warrant escalated enforcement but they do appear to be violations of NRC requirements.

Please provide me with guidance as to how I should address this issue.

Thanks for your help.

Bill

00/25

ST. LUCIE SECURITY EVALUATION

A. Assessment: Overall, site security has been adequate. Licensee management aggressively pursued a tampering event, which occurred in August, to determine the cause. However, prior events leading to this tampering event should have had more indepth review by the licensee to determine if tampering was the cause.

B. Basis:

- The licensee failed to report the confirmed tampering event within one hour, which resulted in a violation.
- Prior to the tampering event in August, two events in May and June related to valves, were documented by the licensee as tampered or unauthorized work. Licensee management failed to notify security of these events.
- The FFD program was transferred to the site. Numerous problems discovered by a QA audit determined the program to be weak.
- Training and Qualification noted as a strength.

C. PLANS:

INSPECTION

One core
One reactive

REASON

81700
Followup on tampering event/FFD implementation

SALP INPUT FEEDER FORM

ISSUE	EXAMPLES	DATE	REFERENCE	NRC CONTACT	ASSESSMENT
1. SECURITY Training & Qualification	Document reviews, interviews and observations of security force performance of duty indicated training was per the Trng. & Qual. Plan. New firing range with aggressive tactical response training aids and a new training bldg. at the range were completed. Trainers were knowledgeable and dedicated, and equipped with an extensive array of training material and equipment.	5/10-14/93 6/20-24/94	93-14: 2.f 94-14: 2.f	TILLMAN STANSBERRY	Security training function was excellent.
2. SECURITY Security Plans, Procedures Adherence and Changes	Two Physical Security Plan changes submitted and approved without complication.	5/10-14/93 6/20-24/94	93-14: 2.a 94-14: 2.c	TILLMAN STANSBERRY	Security, Contingency, Training and Qualification Plan, and Implementing Procedures were current and adequate.
3. SECURITY Access Controls	One violation was due to problem with Corporate HQ's processing access information; access granted before B/I was completed. Hand geometry access control equipment has been installed. One of the first licensees to install this within the US.	5/10-14/93 6/20-24/94	93-14: 2.c 94-14: 2.b	TILLMAN STANSBERRY	Access controls were proactive and exceptional.
4. SECURITY Access Authorization and Safeguards Information Controls	NOT INSPECTED DURING SALP PERIOD				
5. SECURITY Fitness For Duty	(Report pending)	6/26- 7/1/94	94-15:	TOBIN	

DDI/26

6. SECURITY Effectiveness of Management Controls	No items on the Open Items List, downward trends in the Security Incident Tracking and Trending Program and low number of items logged in the Safeguards Nonreportable Incident Log. Testing and maintenance of security equipment support also assisted in low log entries and influencing the downward trending of hardware errors.	6/20-24/94	94-14: 2.b	STANSBERRY	Onsite management's effectiveness was good.
7. SECURITY Assessment Aids	Licensee has improved assessment capabilities by installing the latest model of Video Capture. This reduces the nuisance and false alarm responses by the security force.	6/20-24/94	94-14: 2.a	STANSBERRY	Assessment Aids were outstanding.
8. SECURITY Audits & Corrective Action	QA Audit personnel were appropriately trained and motivated. Annual QA Audit was thorough, producing one Finding; appropriate corrective action was taken to resolve the Finding.	5/10-14/93	93-14: 2.a	TILLMAN	Spirit and innovative perspective of the audit team was commendable.
9. SECURITY Other- Operational Safeguards Response Evaluation (OSRE)	NRC Headquarters evaluation resulted in four concerns, no regulatory violations. Concerns involved responsiveness and equipment allocation.	6/20-24/94	94-14: 2.b	STANSBERRY	Licensee's response to OSRE report was responsive and adequate.

Licensee/Facility:

Florida Power & Light Co.
Saint Lucie 1
Fort Pierce, Florida
Dockets: 50-335
PWR/CE

Notification:

MR Number: 2-96-XXXX
Date: 01/25/96
SRI

Subject: EXCESSIVE BORON DILUTION

Discussion:

At approximately 0220 on January 22, 1996, the Unit 1 control board Reactor Controls Operator (RCO) observed that T_c had decreased from 549°F (normal 100% power value) to 548.7°F. At 0225, he began a manual dilution to the RCS by aligning the Primary Makeup Water (PMW) system directly to the suction of the 1B Charging Pump (approximately 44 gpm capacity). He intended to dilute by adding between 25 and 40 gallons of PMW. Moments after beginning the dilution, the board RCO saw the desk RCO return from the kitchen. He requested that the desk RCO relieve him so that he could prepare his lunch. During the relief, there was no discussion of the dilution in progress. The Nuclear Plant Supervisor (NPS), who was at the desk RCO station, was not aware that a dilution was in progress. The Board RCO returned between 5-10 minutes later and immediately recognized his error. He was securing the dilution when the control room annunciator M-16 "RCP CONT BLD OFF PRESS HIGH" alarmed due to a higher than normal VCT pressure caused by high VCT level (a result of the charging pump taking suction from PMW instead of the VCT). The NPS directed the ANPS take charge and begin a manual boration. Unit 1 briefly entered TS LCO Action Statement 3.2.5 for T_c greater than 549°F. The maximum T_c obtained was 549.9°F at 100.2% reactor power.

The licensee has the root cause under investigation.

Regional Action:

A three member inspection team consisting of two resident and one region-based inspector is onsite investigating this event and evaluating the licensee's corrective actions.

Contact: K. Landis (404)331-5509
M. Miller (407)464-7822
R. Schin (404)331-5588

00/27

NRC QUESTION/RESPONSE FORM

QUESTION #:

DATE:

1/28/96

INSPECTOR:

(CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER

BOB SCHINN

UTILITY REPRESENTATIVE:

DIETZ

INSPECTION CATEGORY:

(CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST

DOCUMENTATION

WALKDOWN

PROCEDURES

NRC REQUEST OR CONCERN:

ASSUMING THE DIVERT VALVE HAD BEEN IN THE "DIVERT" POSITION, e.g., SUCH THAT THE "RCP BLEEDOFF PRESSURE" ANNUNCIATOR HAD NOT ACTUATED;

- ① WHAT IS/ARE THE ALARM(S) THAT WOULD WARN THE OPERATOR THAT R_x POWER $> 100\%$ PRIOR TO THE VAR HI POWER PRETRIP/TRIP?
- ② DOES DDPS HAVE AN ALARM ASSOCIATED WITH "CALORIMETRIC POWER"?
- ③ AT APPROX. WHAT R_x POWER (TOTAL POWER) WOULD INCOLE ALARMS CORRELATE TO?

RESPONDING INDIVIDUAL/DEPT:

DIETZ - 1 LIC
(VERIFIED W. MEAD R/E)

UTILITY RESPONSE:

- ① Point I.D. #742 ON PLANT COMPUTER (DDPS) PROVIDES AN ALARM FOR "AT POWER" THAT IS EQUIVALENT TO 101% R_x THERMAL POWER. THIS ALARM DID NOT ACTIVATE DURING THE EVENT.
- ② SPECIFICALLY FROM "CALORIMETRIC POWER", ANSWER IS NO.
- ③ INCOLE ALARMS BASED ON "LOCAL POWER LEVELS (PLHR)" AT 20% , 40% , 60% , 80% OF CORE ELEVATION. DO NOT DIRECTLY CORRELATE TO "TOTAL CORE POWER" - (NO INCOLE ALARMS OCCURRED DURING THE EVENT)

DD/30

Joyce on her way in 1/23

NRC QUESTION/RESPONSE FORM

QUESTION #: _____

DATE: 1/27

INSPECTOR: (CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER Schinn

UTILITY REPRESENTATIVE: GRM

INSPECTION CATEGORY: (CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST DOCUMENTATION WALKDOWN PROCEDURES

NRC REQUEST OR CONCERN: Need copies of
OP 1-3200020 & OP 1-1200051 for
the 1/22 mid shift.
Need Strip charts for JR011 & JR010
for period 2-3:30 AM 1/22

RESPONDING INDIVIDUAL/DEPT: Joyce Mann ^{Duct} Vault

UTILITY RESPONSE: (Procedures Attached) Steve Sandin has
the strip charts JR011 & JR010.

FLORIDA POWER & LIGHT COMPANY
ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051
REVISION 17

1.0 TITLE:

NUCLEAR AND DELTA T POWER CALIBRATION

2.0 REVIEW AND APPROVAL:

Reviewed by Plant Nuclear Safety Committee _____ 5/1 1975

Approved by K. N. Harris Plant General Manager _____ 5/19 1975

Revision 17 Reviewed by Facility Review Group _____ 8/29 1995

Approved by J. Scarola Plant General Manager _____ 8/29 1995

3.0 PURPOSE:

To provide detailed instructions for the adjustment of Nuclear and Delta T Power to agree with the thermal energy balance calculation.

4.0 PRECAUTIONS AND LIMITS:

4.1 Perform calibration on only one channel at a time.

4.2 If a pre-trip alarm is received on a power range safety channel being adjusted, stop adjustment. Verify proper test switch lineup and if test line up is correct, request I & C recheck channel pre-trip and trip setpoints.

4.3 If steady state equilibrium power level conditions change from time of completing a plant calorimetric and completion of this procedure, a new calorimetric must be performed prior to performing channel adjustment, unless a monitor of reactor power deemed acceptable by the ANPS/NPS is available. In no case is a change in reactor power of $\geq 2\%$ acceptable.

4.4 Notify Reactor Engineering of a deviation in excess of $\pm 5\%$ between Calorimetric and NI indicated power prior to performing channel calibration.

4.5 T_c should be constant and consistent with the T_c present during the calorimetric calculation.

4.6 Power level should remain constant during calibration.

FOR INFORMATION ONLY	
This document is not controlled. Before using, verify information with a controlled document.	
DATE VERIFIED	INITIAL
1/21/96	AKB

S 1 OPS	
DATE	960122
DOCT PROCEDURE	
DOCN	1-1200051
SYS	CPS
COMP	COMPLETED
ITM	17

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

5.0 RELATED SYSTEM STATUS:

5.1 Power level is being maintained constant through the completion of this procedure.

6.0 REFERENCES:

6.1 Guideline NT-PY-4, Appendix H, Nuclear and Delta T Calibration

6.2 Technical Manual 8770-7120, Reactor Protective System, Volume I

6.3 St. Lucie Unit 1 Technical Specifications, Table 4.3.1

7.0 RECORDS REQUIRED:

7.1 A signed and dated copy of this procedure shall be maintained in the plant files in accordance with QI 17-PR/PSL-1, "Quality Assurance Records."

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

8.0 INSTRUCTIONS:NOTE

At times during core cycle a known hot leg swirl effect may be encountered indicating that flow stratification is occurring. This effect may cause a slight rise or fall in one or both hot leg indicated temperatures. When this occurs a similar change in Delta T power may be seen in any of the four RPS channels. When this phenomenon is recognized as occurring, Appendix A should be performed.

NOTE

Perform calibration on only one channel at a time.

8.1 Calculation of Calorimetric Power

1. Demand a DDPS calorimetric and record the value of reactor power.

100.0 %

2. Request T cold from ERDADS Cold Leg Temperature display (screen number CLT) and record T cold.

548.9 °F

3. Calculate primary system power in accordance with OP 1-3200020 and record the value of reactor power.

99.65 %

NOTE

For this procedure, use DDPS power as the reference power unless there is a 2% deviation between the DDPS and manual calorimetric values, in which case use the higher value as reference power.

4. If the power values recorded in Step 8.1.1 and 8.1.3 differ by more than 2%, notify I & C as soon as possible.

8.2 Proceed with the following instructions for Channel A of the Reactor Protection System.

- 8.3 If adjusting Delta T Power calibration and/or Nuclear Pwr Calibrate potentiometer, Then use keys 81, 87, 88 and 90 from the ANPS key locker to bypass the Hi Power, TM/Lo Press, Loss Load and Loc Pwr Den Trips (Bypass Switch 1, 7, 8 and 10, located on the Trip Inhibit Switch Panel) and verify that the respective indicator lights illuminate.

/R17

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

8.0 INSTRUCTIONS: (continued)

8.4 Record the Delta T Pwr Calibration potentiometer dial setting.

POT SETTING

A 374

B 401

C 389

D 432

8.5 Position the meter input switch to the Delta T PWR position and record the DVM reading.

DVM DELTA T PWR (%)

A 100

B 100

C 100

D 100

8.6 While monitoring the DVM indication, disengage the locking device on the Delta T Power Calibration potentiometer and slowly begin adjusting for a DVM indication equal to that of the plant calorimetric. Engage the locking device on the Delta T Power Calibration potentiometer and verify the DVM indication equals the plant calorimetric value.

8.7 Record the new Delta T Power Calibration potentiometer setting.

POT SETTING

A 374

B 401

C 389

D 432

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

8.0 INSTRUCTIONS: (continued)

8.8 Record the DVM Delta T Power value.

DVM DELTA T PWR (%)

A 100

B 100

C 100

D 100

8.9 Record the Nuclear Pwr Calibrate potentiometer dial setting.

POT SETTING

A 598

B 492

C 494

D 598

8.10 Position meter input switch to Nuclear Pwr position and record the DVM reading.

DVM NUCLEAR PWR (%)

A 100

B 100

C 100

D 100

- * 8.11 While monitoring the RPSCIP Nuclear Pwr - Delta T Pwr deviation meter, disengage the locking device on the Nuclear Pwr Calibrate potentiometer and slowly begin adjusting for a meter indication equal to zero. Engage the locking device on the Nuclear Power Calibrate potentiometer and verify the Nuclear Pwr - Delta T Pwr meter indicates zero.

ST. LUCIE UNIT 1
 OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

8.0 INSTRUCTIONS: (continued)

8.12 Record the new Nuclear Pwr Calibrate potentiometer setting.

POT SETTING

A 598

B 492

C 494

D 598

8.13 Record the RPSCIP Nuclear Pwr - Delta T Pwr deviation meter reading.

DEVIATION (%)

A 0

B 0

C 0

D 0

8.14 Record the Q-Power meter reading located on RTGB-104.

Q-POWER (%)

JI-003A 101

JI-003B 100

JI-003C 101

JI-003D 100

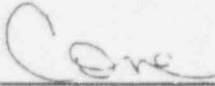
8.15 Clear all trips and alarms. If used, remove trip bypass keys 81, 87, 88 and 90 and verify that the respective indicator lights extinguish.


ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-1200051, REVISION 17
NUCLEAR AND DELTA T POWER CALIBRATION

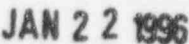
8.0 INSTRUCTIONS: (continued)

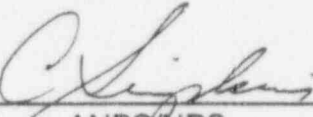
- 8.16 Complete steps 8.3 through 8.15 for Channel B, C and D of the Reactor Protection System.
- 8.17 Return keys 81, 87, 88 and 90 to ANPS key locker.

*Satisfactory performance of all the above asterisked steps assures conformance with applicable Technical Specifications.


_____ RCO


_____ Time


_____ Date

Reviewed: 
_____ ANPS/NPS

NRC QUESTION/RESPONSE FORM

QUESTION #:

DATE:

1/28/96

INSPECTOR:

(CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER

BOB SCHINN

UTILITY REPRESENTATIVE:

DIETZ

INSPECTION CATEGORY:

(CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST

DOCUMENTATION

WALKDOWN

PROCEDURES

NRC REQUEST OR CONCERN:

PLEASE PROVIDE A GRAPH SHOWING Temp vs. TIME FOR THE PERIOD COVERING "BEFORE, DURING, AND AFTER" THE TRANSIENT.

RESPONDING INDIVIDUAL/DEPT:

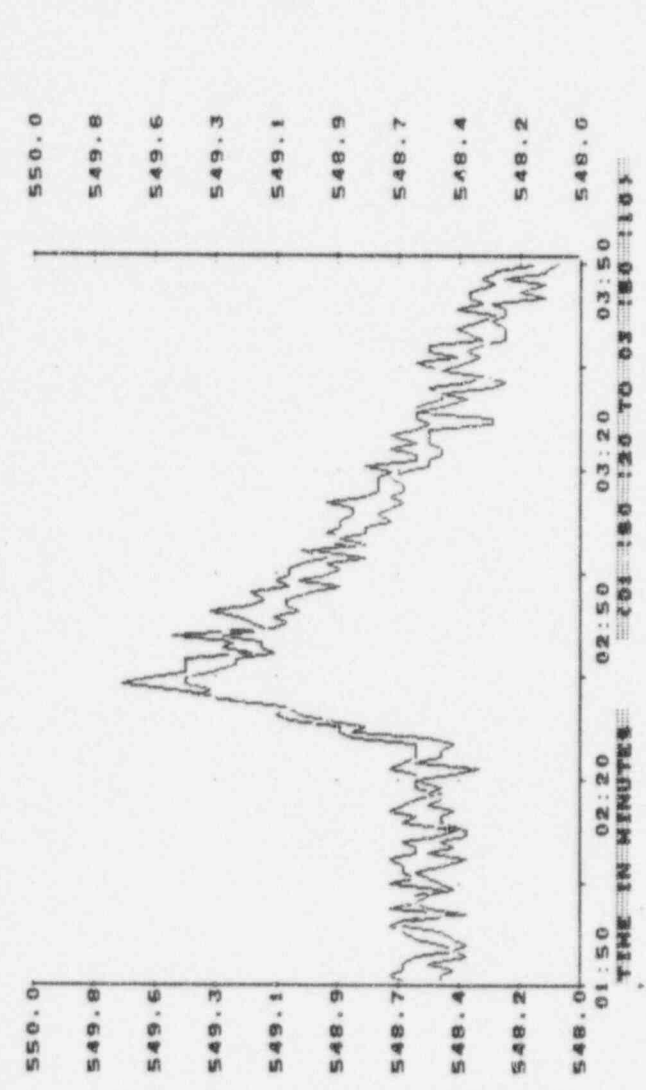
Diety : LIC

UTILITY RESPONSE:

GRAPHS Attached for IA1, IA2, IB1, IB2 NARROW BANDS COLD LEG TEMPERATURES ALONG WITH TABULATED VALUES FROM ERDADS

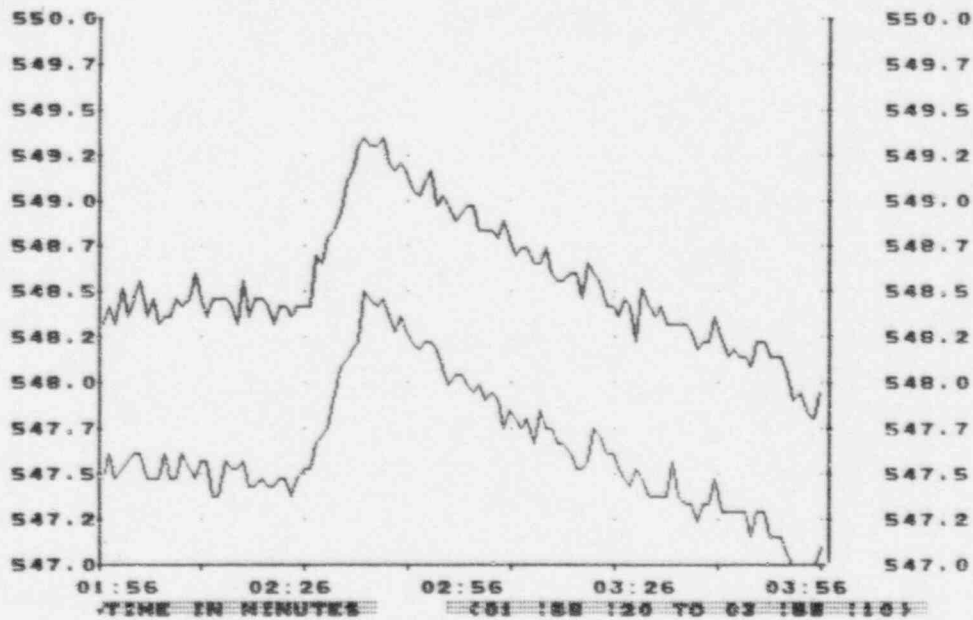
GRAPHS FOR HOT LEG TEMPS A & B ALSO ATTACHED

1. 10. 20. 30. 40. 50. 60. 70. 80. 90. 100. 110. 120. 130. 140. 150. 160. 170. 180. 190. 200. 210. 220. 230. 240. 250. 260. 270. 280. 290. 300. 310. 320. 330. 340. 350. 360. 370. 380. 390. 400. 410. 420. 430. 440. 450. 460. 470. 480. 490. 500. 510. 520. 530. 540. 550. 560. 570. 580. 590. 600. 610. 620. 630. 640. 650. 660. 670. 680. 690. 700. 710. 720. 730. 740. 750. 760. 770. 780. 790. 800. 810. 820. 830. 840. 850. 860. 870. 880. 890. 900. 910. 920. 930. 940. 950. 960. 970. 980. 990. 1000.



UNIT 1 TELE BIT LEVEL
 COLD LEG TEMP LOOP 1A1
 COLD LEG TEMP LOOP 1A2
 02:50:37
 548.84 DEG-F
 548.88 DEG-F

| | | | |
|----------------|----------------------------------|----------|-----------|
| UNIT 2 22 | UNIT BATTERY (AIR GROUND STATUS) | U | 230 |
| ST LUCIE PLANT | BIT LEVEL | 03:56:12 | 22 JAN 96 |
| 0A0023-1 | COLD LEG TEMP LOOP 1B1 | 547.98 | DEG-F |
| 0A0036-1 | COLD LEG TEMP LOOP 1B2 | 547.18 | DEG-F |



USE PG-UP AND PG-DN TO PAGE, PREV OR CLEAR TO END

TABULAR TREND REPORT

TREND # 1 PAGE 25

| DATE / TIME | 0A0033-1 | 0R0035-1 |
|-------------------|----------|----------|
| 22 JAN 96 2:23:30 | 548.50 | 548.55 |
| 22 JAN 96 2:24:30 | 548.60 | 548.51 |
| 22 JAN 96 2:25:30 | 548.60 | 548.60 |
| 22 JAN 96 2:26:30 | 548.60 | 548.46 |
| 22 JAN 96 2:27:30 | 548.69 | 548.37 |
| 22 JAN 96 2:28:30 | 548.65 | 548.60 |
| 22 JAN 96 2:29:30 | 548.60 | 548.51 |
| 22 JAN 96 2:30:30 | 548.60 | 548.51 |
| 22 JAN 96 2:31:30 | 548.60 | 548.46 |
| 22 JAN 96 2:32:30 | 548.83 | 548.55 |
| 22 JAN 96 2:33:30 | 548.88 | 548.84 |
| 22 JAN 96 2:34:30 | 548.88 | 548.79 |
| 22 JAN 96 2:35:30 | 549.07 | 548.93 |
| 22 JAN 96 2:36:30 | 549.11 | 549.02 |
| 22 JAN 96 2:37:30 | 549.07 | 549.07 |
| 22 JAN 96 2:38:30 | 549.25 | 549.26 |
| 22 JAN 96 2:39:30 | 549.35 | 549.40 |
| 22 JAN 96 2:40:30 | 549.49 | 549.35 |
| 22 JAN 96 2:41:30 | 549.68 | 549.44 |
| 22 JAN 96 2:42:30 | 549.58 | 549.44 |
| 22 JAN 96 2:43:30 | 549.44 | 549.35 |
| 22 JAN 96 2:44:30 | 549.44 | 549.26 |
| 22 JAN 96 2:45:30 | 549.44 | 549.26 |
| 22 JAN 96 2:46:30 | 549.21 | 549.12 |
| 22 JAN 96 2:47:30 | 549.30 | 549.16 |
| 22 JAN 96 2:48:30 | 549.25 | 549.16 |
| 22 JAN 96 2:49:30 | 549.49 | 549.30 |
| 22 JAN 96 2:50:30 | 549.10 | 549.12 |
| 22 JAN 96 2:51:30 | 549.21 | 549.07 |
| 22 JAN 96 2:52:30 | 549.25 | 549.12 |

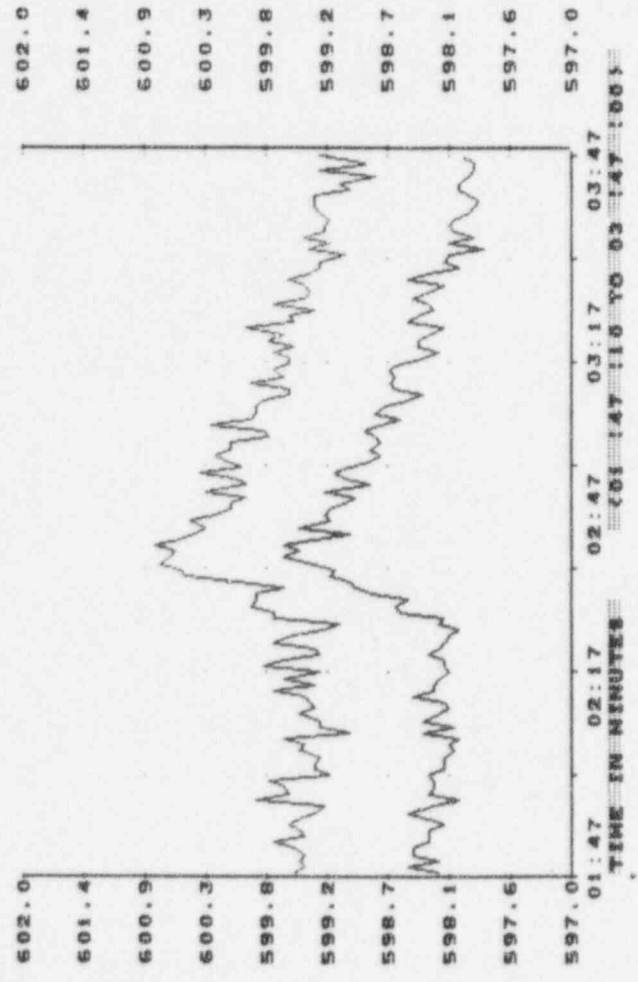
USE PG-UP AND PG-DN TO PAGE, PREV OR CLEAR TO END

TABULAR TREND REPORT

TREND # 1 PAGE 25

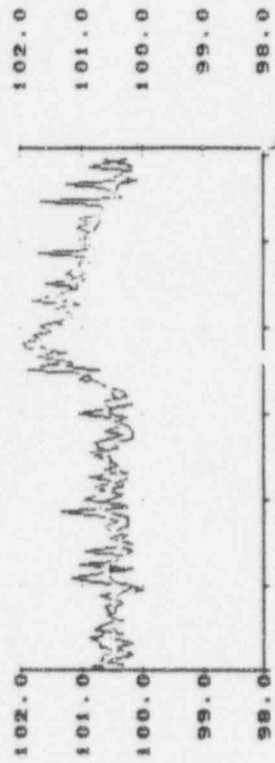
| DATE / TIME | QA0023-1 | QA0035-1 |
|-------------------|----------|----------|
| 22 JAN 96 2:27:40 | 548.46 | 547.47 |
| 22 JAN 96 2:28:40 | 548.37 | 547.38 |
| 22 JAN 96 2:29:40 | 548.41 | 547.47 |
| 22 JAN 96 2:30:40 | 548.41 | 547.47 |
| 22 JAN 96 2:31:40 | 548.37 | 547.47 |
| 22 JAN 96 2:32:40 | 548.65 | 547.66 |
| 22 JAN 96 2:33:40 | 548.69 | 547.71 |
| 22 JAN 96 2:34:40 | 548.74 | 547.75 |
| 22 JAN 96 2:35:40 | 548.84 | 548.04 |
| 22 JAN 96 2:36:40 | 548.98 | 547.99 |
| 22 JAN 96 2:37:40 | 549.02 | 548.18 |
| 22 JAN 96 2:38:40 | 549.16 | 548.18 |
| 22 JAN 96 2:39:40 | 549.30 | 548.36 |
| 22 JAN 96 2:40:40 | 549.35 | 548.41 |
| 22 JAN 96 2:41:40 | 549.35 | 548.46 |
| 22 JAN 96 2:42:40 | 549.35 | 548.50 |
| 22 JAN 96 2:43:40 | 549.44 | 548.36 |
| 22 JAN 96 2:44:40 | 549.30 | 548.27 |
| 22 JAN 96 2:45:40 | 549.12 | 548.22 |
| 22 JAN 96 2:46:40 | 549.21 | 548.27 |
| 22 JAN 96 2:47:40 | 549.12 | 548.27 |
| 22 JAN 96 2:48:40 | 549.16 | 548.22 |
| 22 JAN 96 2:49:40 | 549.07 | 548.27 |
| 22 JAN 96 2:50:40 | 549.16 | 548.13 |
| 22 JAN 96 2:51:40 | 548.93 | 548.13 |
| 22 JAN 96 2:52:40 | 549.07 | 548.22 |
| 22 JAN 96 2:53:40 | 548.98 | 548.13 |
| 22 JAN 96 2:54:40 | 548.98 | 548.08 |
| 22 JAN 96 2:55:40 | 549.02 | 548.04 |
| 22 JAN 96 2:56:40 | 548.84 | 548.04 |

STATION: BOSTON 1
 UNIT: 1
 000029-1
 000030-1
 HL TEMP A
 HL TEMP B
 02:47:22
 22 JAN 96
 597.97 DEG-F
 598.13 DEG-F



01:47 02:17 02:47 03:17 03:47
 TIME IN MINUTES
 01:47:10 TO 03:47:00

UNIT 1 BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL
 ST LUCIE SITE UNIT 1 BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL BIT LEVEL
 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1
 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1 J1005-1



01:25 01:55 02:25 02:55 03:25
 TIME IN MINUTES 01:25 TO 03:25
 POWER RANGE REACTOR POWER CH 100.33 X
 POWER RANGE REACTOR POWER CH 100.33 X

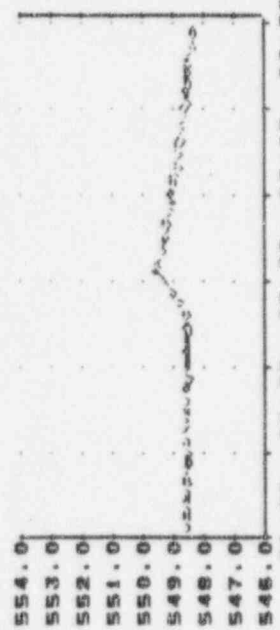
J1007-1
 J1008-1



01:25 01:55 02:25 02:55 03:25
 TIME IN MINUTES 01:25 TO 03:25
 POWER RANGE REACTOR POWER CH 100.33 X
 POWER RANGE REACTOR POWER CH 100.33 X

ST LUCIE ISLAND
 UNIT 1 & 2
 000033-1
 000035-1

STATION: 10001
 DATE: 22 JAN 98
 TIME: 02:39
 COLD LEG TEMP LOOP 1A1
 COLD LEG TEMP LOOP 1A2



TIME IN MINUTES: 01:39 TO 03:39
 COLD LEG TEMP LOOP 1B1
 COLD LEG TEMP LOOP 1B2

000023-1
 000036-1

STATION: 10001
 DATE: 22 JAN 98
 TIME: 02:39
 COLD LEG TEMP LOOP 1A1
 COLD LEG TEMP LOOP 1A2



TIME IN MINUTES: 01:39 TO 03:39

NRC QUESTION/RESPONSE FORM

QUESTION #:

DATE:

1/28/96

INSPECTOR:

(CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER

BOB SCHIND

UTILITY REPRESENTATIVE:

DIETZ

INSPECTION CATEGORY:

(CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST

DOCUMENTATION

WALKDOWN

PROCEDURES

NRC REQUEST OR CONCERN:

① ERDADS PRINTOUTS SHOW JI-005-1 THROUGH JI-008-1 LABELLED AS "POWER RANGE REACTOR POWER". IS THIS CALORIMETRIC (PPS) POWER, NI POWER, DT POWER, OR Q POWER.
② ERDADS PRINTOUT ALSO LABELS QA0058-1 AS "POWER RANGE POWER LEVEL"; WHAT IS THIS??

RESPONDING INDIVIDUAL/DEPT:

DIETZ - 1 LIC

UTILITY RESPONSE:

JI 005 thru JI 008 ARE INDICATIONS OF "Q POWER", WHERE Q-POWER IS THE AUCTIONEERED MAXIMUM OF NI POWER AND "DT POWER".

ST LUCIE PLANT
UNIT 1 SITLV

SIT LEVEL

15:31:30
27 JAN 96

RE

QA0058-1

POWER RANGE POWER LEVEL

1 %

J1005-1

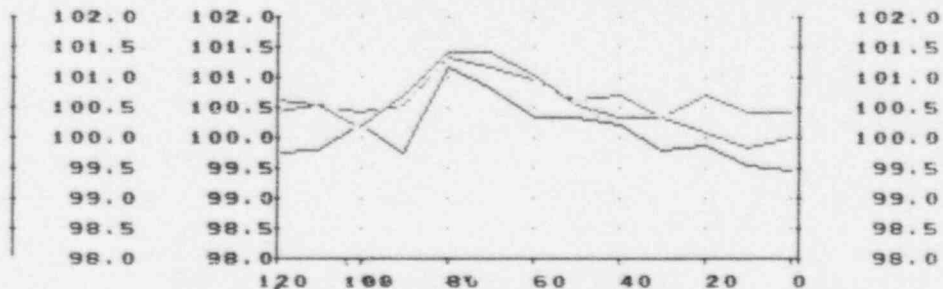
POWER RANGE REACTOR POWER CH

1 %

J1006-1

POWER RANGE REACTOR POWER CH

1 %



J1007-1

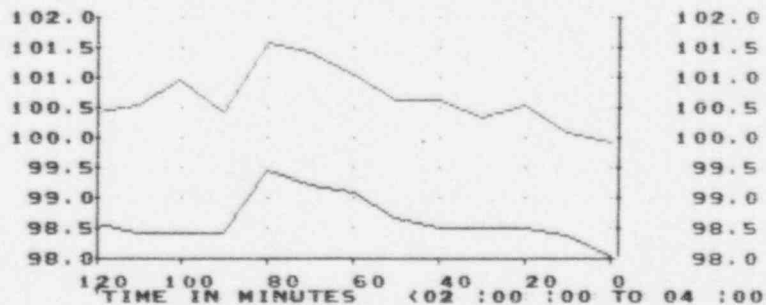
TIME IN MINUTES (02:00:00 TO 04:00:00) → 1/22/96

1 %

J1008-1

POWER RANGE REACTOR POWER CH

1 %



0200 0210 0240 0300 0330 0340 0400

USE PG-UP AND PG-DN TO PAGE, PREV OR CLEAR TO END

TREND # 1 PAGE 65

TABULAR TREND REPORT

| DATE | / | TIME | |
|------|--------|--------|---|
| 21 | JAN 96 | 23:10: | 0 |
| 21 | JAN 96 | 23:20: | 0 |
| 21 | JAN 96 | 23:30: | 0 |
| 21 | JAN 96 | 23:40: | 0 |
| 21 | JAN 96 | 23:50: | 0 |
| 22 | JAN 96 | 0:00: | 0 |
| 22 | JAN 96 | 0:10: | 0 |
| 22 | JAN 96 | 0:20: | 0 |
| 22 | JAN 96 | 0:30: | 0 |
| 22 | JAN 96 | 0:40: | 0 |
| 22 | JAN 96 | 0:50: | 0 |
| 22 | JAN 96 | 1:00: | 0 |
| 22 | JAN 96 | 1:10: | 0 |
| 22 | JAN 96 | 1:20: | 0 |
| 22 | JAN 96 | 1:30: | 0 |
| 22 | JAN 96 | 1:40: | 0 |
| 22 | JAN 96 | 1:50: | 0 |
| 22 | JAN 96 | 2:00: | 0 |
| 22 | JAN 96 | 2:10: | 0 |
| 22 | JAN 96 | 2:20: | 0 |
| 22 | JAN 96 | 2:30: | 0 |
| 22 | JAN 96 | 2:40: | 0 |
| 22 | JAN 96 | 2:50: | 0 |
| 22 | JAN 96 | 3:00: | 0 |
| 22 | JAN 96 | 3:10: | 0 |
| 22 | JAN 96 | 3:20: | 0 |
| 22 | JAN 96 | 3:30: | 0 |
| 22 | JAN 96 | 3:40: | 0 |
| 22 | JAN 96 | 3:50: | 0 |
| 22 | JAN 96 | 4:00: | 0 |

0A0058-1
99.94
100.00
100.05
100.00
99.92
100.25
99.94
99.94
100.13
100.04
100.07
99.86
99.86
100.11
99.88
100.00
100.07
99.76
99.80
100.19
99.76
101.15
100.82
100.33
100.33
100.19
99.78
99.86
99.55
99.47

USE PG-LP AND FG-DH TO PAGE; PREV OR CLEAR TO END

TREND # 1 PAGE 66

TABULAR TREND REPORT

| DATE | / | TIME | |
|------|-----|------|---------|
| 22 | JAN | 96 | 4:10: 0 |
| 22 | JAN | 96 | 4:20: 0 |
| 22 | JAN | 96 | 4:30: 0 |
| 22 | JAN | 96 | 4:40: 0 |
| 22 | JAN | 96 | 4:50: 0 |
| 22 | JAN | 96 | 5: 0: 0 |
| 22 | JAN | 96 | 5:10: 0 |
| 22 | JAN | 96 | 5:20: 0 |
| 22 | JAN | 96 | 5:30: 0 |
| 22 | JAN | 96 | 5:40: 0 |
| 22 | JAN | 96 | 5:50: 0 |
| 22 | JAN | 96 | 6: 0: 0 |
| 22 | JAN | 96 | 6:10: 0 |
| 22 | JAN | 96 | 6:20: 0 |
| 22 | JAN | 96 | 6:30: 0 |
| 22 | JAN | 96 | 6:40: 0 |
| 22 | JAN | 96 | 6:50: 0 |
| 22 | JAN | 96 | 7: 0: 0 |
| 22 | JAN | 96 | 7:10: 0 |
| 22 | JAN | 96 | 7:20: 0 |
| 22 | JAN | 96 | 7:30: 0 |
| 22 | JAN | 96 | 7:40: 0 |
| 22 | JAN | 96 | 7:50: 0 |
| 22 | JAN | 96 | 8: 0: 0 |
| 22 | JAN | 96 | 8:10: 0 |
| 22 | JAN | 96 | 8:20: 0 |
| 22 | JAN | 96 | 8:30: 0 |
| 22 | JAN | 96 | 8:40: 0 |
| 22 | JAN | 96 | 8:50: 0 |
| 22 | JAN | 96 | 9: 0: 0 |

0A0058-1
99.70
99.70
100.00
100.02
100.02
99.90
100.19
100.04
100.52
100.58
100.17
100.15
100.21
100.43
100.25
100.37
100.31
100.37
100.46
100.04
100.31
100.35
100.31
100.17
100.17
100.17
100.21
100.60
100.29
100.33

TABULAR TREND REPORT

| DATE | TIME | VALUE |
|-----------|----------|--------|
| 22 JAN 96 | 9:10:00 | 100.31 |
| 22 JAN 96 | 9:20:00 | 100.43 |
| 22 JAN 96 | 9:30:00 | 99.98 |
| 22 JAN 96 | 9:40:00 | 100.33 |
| 22 JAN 96 | 9:50:00 | 100.17 |
| 22 JAN 96 | 10:00:00 | 100.37 |
| 22 JAN 96 | 10:10:00 | 100.29 |
| 22 JAN 96 | 10:20:00 | 100.52 |
| 22 JAN 96 | 10:30:00 | 100.31 |
| 22 JAN 96 | 10:40:00 | 100.35 |
| 22 JAN 96 | 10:50:00 | 100.52 |
| 22 JAN 96 | 11:00:00 | 100.41 |
| 22 JAN 96 | 11:10:00 | 100.43 |
| 22 JAN 96 | 11:20:00 | 99.92 |
| 22 JAN 96 | 11:30:00 | 100.11 |
| 22 JAN 96 | 11:40:00 | 99.92 |
| 22 JAN 96 | 11:50:00 | 100.13 |
| 22 JAN 96 | 12:00:00 | 99.96 |
| 22 JAN 96 | 12:10:00 | 100.04 |
| 22 JAN 96 | 12:20:00 | 100.19 |
| 22 JAN 96 | 12:30:00 | 99.96 |
| 22 JAN 96 | 12:40:00 | 100.17 |
| 22 JAN 96 | 12:50:00 | 100.15 |
| 22 JAN 96 | 13:00:00 | 100.09 |
| 22 JAN 96 | 13:10:00 | 100.17 |
| 22 JAN 96 | 13:20:00 | 100.11 |
| 22 JAN 96 | 13:30:00 | 100.00 |
| 22 JAN 96 | 13:40:00 | 100.02 |
| 22 JAN 96 | 13:50:00 | 100.15 |
| 22 JAN 96 | 14:00:00 | 99.88 |

ST. LUCIE PLANT NUCLEAR PLANT ERDADS DATA BASE LISTING

UNIT #1 ANALOG POINTS

| DATA BASE
POINT NAME
(PT. ID.) | DATA BASE
POINT DESCRIPTION | POINT
ENG.
UNITS | ENGINEERING
UNITS
HIGH SCALE | ENGINEERING
UNITS
LOW SCALE |
|--------------------------------------|--------------------------------|------------------------|------------------------------------|-----------------------------------|
| MV08-1B-1 | MS HDR B BYPASS 1B POSITION | | | |
| HCV08-1B-1 | MS HDR B ISOL VLV 1B POSITION | | | |
| MSLRAD-1 | MSL HIGHEST RADIATION | MR/HR | | |
| RE26-37-1 | NEW FUEL STORAGE AREA RAD | MR/HR | 1.00000000E+04 | 1.00000000E-01 |
| QA0037-1 | PLANT EFFLUENT RADIATION | | | |
| RSC26-1H-1 | PLANT STACK (HR) NOBLE GAS RAD | UC/CC | | |
| RSC26-1L-1 | PLANT STACK (LR) NOBLE GAS RAD | UC/CC | | |
| RSC26-1M-1 | PLANT STACK (MR) NOBLE GAS RAD | UC/CC | | |
| RSC26-1F-A-1 | PLANT STACK FLOW 15MIN AVERAGE | SCFM | | |
| RSC26-1F-1 | PLANT STACK FLOW CH A RAD | SCFM | | |
| RSC26-1I-1 | PLANT STACK IODINE RAD | CPM | | |
| RSC26-1P-1 | PLANT STACK PARTICULATE RAD | CPM | | |
| RSC26-1H-A-1 | PLANT STACK(HL)NG RAD 15M AVG | UC/CC | | |
| RSC26-1L-A-1 | PLANT STACK(LR)NG RAD 15M AVG | UC/CC | | |
| RSC26-1M-A-1 | PLANT STACK(MR)NG RAD 15M AVG | UC/CC | | |
| CGAS-1 | PLANT STK. NOBLE GAS AVG | UC/CC | | |
| FT1402-O-1 | PORV V1402 FLOW ---- SQRT CONV | % | 1.00000000E+02 | 0.00000000E+00 |
| FT1404-O-1 | PORV V1404 FLOW ---- SQRT CONV | % | 1.00000000E+02 | 0.00000000E+00 |
| QA0058-1 | POWER RANGE POWER LEVEL | PRCT | | |
| J1005-1 | POWER RANGE REACTOR POWER CH A | % | 1.25000000E+02 | 0.00000000E+00 |
| J1006-1 | POWER RANGE REACTOR POWER CH B | % | 1.25000000E+02 | 0.00000000E+00 |
| J1007-1 | POWER RANGE REACTOR POWER CH C | % | 1.25000000E+02 | 0.00000000E+00 |
| J1008-1 | POWER RANGE REACTOR POWER CH D | % | 1.25000000E+02 | 0.00000000E+00 |
| QTA677-1 | POWER SELECTION | %/CPS | | |
| PROPT-1 | PRESSURIZER HEATER TOTAL POWER | KW | | |
| AM943-1 | PRESSURIZER HTR BUS 1A3 AMPS | AMPS | 1.50000000E+02 | 0.00000000E+00 |
| AM944-1 | PRESSURIZER HTR BUS 1B3 AMPS | AMPS | 1.50000000E+02 | 0.00000000E+00 |
| LT1110X-1 | PRESSURIZER LEVEL | % | 1.00000000E+02 | 0.00000000E+00 |
| LT1110Y-1 | PRESSURIZER LEVEL | % | 1.00000000E+02 | 0.00000000E+00 |
| QA0001-1 | PRESSURIZER LEVEL | PRCT | | |
| LT1103-1 | PRESSURIZER LEVEL (COLD CNDTN) | % | 1.00000000E+02 | 0.00000000E+00 |

8770-12058 REV 2 PG B-15

NRC QUESTION/RESPONSE FORM

QUESTION #:

DATE:

1/28/96

INSPECTOR:

(CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER

Bob SCHINN

UTILITY REPRESENTATIVE:

Dietz

INSPECTION CATEGORY:

(CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST

DOCUMENTATION

WALKDOWN

PROCEDURES

NRC REQUEST OR CONCERN:

STRIP CHART FOR MWe during the event, e.g. IHE states that
870 MWe started at beginning of Shift and -not indicated plant
"MWe" reached 885 and indicated average reactor power was
approximately 100.2%." This would imply a 1.2% increase.

RESPONDING INDIVIDUAL/DEPT:

Dietz 1616

UTILITY RESPONSE:

See Attached Strip chart - Max MWe
shown to be 875 MWe, not the 885 MWe
stated in the IHE.

MEGAWATTS

100 200 300 400 500 600 700 800 900 1000

Time ~ 0000 Time ~~0100~~

20 minutes
base accuracy

0100

~~0100~~

0250

0300
0350
0400

MEGAWATTS

100 200 300 400 500 600 700 800 900 1000

0400

(MVA 2)
1/22/96

ST. LUCIE UNIT 1
 OPERATING PROCEDURE NO. 1-3200020, REVISION 23
PRIMARY SYSTEM MANUAL CALORIMETRIC

DATA SHEET 1
 (Page 1 of 3)

Date: JAN 22 1996

Time: 00:01 ^(AM) PM

1. ENTER MAIN STEAM PRESSURE

PI-8013A 880
 PI-8023A 870
 PI-8013B 860
 PI-8023B 860
 PI-8013C 860
 PI-8023C 860
 PI-8013D 860
 PI-8023D 860

ENTER FEEDWATER FLOW

FR-9011 581 X 10⁶ lb/hr /R23
 FR-9021 595 X 10⁶ lb/hr /R23
 M_{FW} = 1179 X 10⁶ lb/hr (TOTAL) /R23

Average Steam Pressure = $\frac{6910}{\text{TOTAL}}$ divided by 8 = $\frac{863.75}{\text{AVERAGE}}$

ENTER FEEDWATER TEMPERATURE

Speedomax Pt. 1 434 °F
 Pt. 2 434 °F

Average Feedwater Temperature $\frac{865}{\text{TOTAL}}$ divided by 2 = $\frac{434}{\text{AVERAGE}}$

From a set of steam tables, enter the enthalpy of the steam pressure:

$h_{\text{steam}} = 1197.552$ BTU/lbm

Calculate the heat output due to steam flow (Q_{steam}):

$\frac{1197.552}{h_{\text{steam}}} \times [M_{\text{FW}} - M_{\text{BD}}] = Q_{\text{steam}}$
 $\frac{1197.552}{E6} \times [1179 - 585.9] E6 = Q_{\text{steam}}$
 $\frac{14071.71}{E6} E6$ BTU/hr

| | |
|------|---------------------|
| S | OPS |
| DATE | <u>960122</u> |
| DOCT | <u>PROCEDURE</u> |
| DOCN | <u>OP-1-3200020</u> |
| SYS | <u>OPS</u> |
| COMP | <u>125</u> |
| ITM | <u>23</u> |

ST. LUCIE UNIT 1
 OPERATING PROCEDURE NO. 1-3200020, REVISION 23
PRIMARY SYSTEM MANUAL CALORIMETRIC

DATA SHEET 1

(Page 2 of 3)

From the steam tables, enter the enthalpy of the feedwater at the average feedwater temperature:

$$h_{FW} = \frac{412.35}{1} \frac{BTU}{lbm}$$

Calculate the feedwater heat input (Q_{FW}):

$$\left(\frac{11.79}{M_{FW}} \times 10^6 \right) \times \left(\frac{412.35}{h_{FW}} \right) = \frac{4861.60}{Q_{FW}} \times 10^6 \frac{BTU}{hr}$$

Circle the total blowdown flow from the S/Gs and the corresponding heat output:
 (Interpolation is not required, circle the closest blowdown flow below)

| Total blowdown flow
(both steam generators) | Mass flow
of blowdown M_{BD} | Heat output from
blowdown Q_{BD} |
|--|-----------------------------------|---------------------------------------|
| 40 GPM | .019799 E6 lbm/hr | 9.660 E6 BTU/hr |
| 80 GPM | .039599 E6 lbm/hr | 19.320 E6 BTU/hr |
| 120 GPM | .059398 E6 lbm/hr | 28.981 E6 BTU/hr |
| 160 GPM | .079198 E6 lbm/hr | 38.641 E6 BTU/hr |
| 200 GPM | .098997 E6 lbm/hr | 48.301 E6 BTU/hr |
| 240 GPM | .118797 E6 lbm/hr | 57.961 E6 BTU/hr |

Calculate the heat output from the core:

$$\left[\frac{14071.71}{Q_{STEAM}} - \frac{4861.60}{Q_{FW}} + \frac{19.320}{Q_{BD}} - \frac{47.23}{Q_{OTHER}} \right] \times 10^6 \frac{BTU}{hr} = \frac{9182.30 \times 10^6}{Q_{CORE}} \frac{BTU}{hr}$$

Calculate percent core power:

$$\text{Core Power} = \frac{9182.30 \times 10^6}{Q_{CORE}} \text{ divided by } (92.143 \times 10^6) =$$

99.65% Manual
Calorimetric
Power

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-3200020, REVISION 23
PRIMARY SYSTEM MANUAL CALORIMETRIC

DATA SHEET 1
(Page 3 of 3)

Record DDPS Calorimetric Power (Point 32), (Point ID 31 should be used below if the reactor has not been in a stable power configuration for at least 10 minutes):

DDPS Calorimetric Power = 100.0 %

NOTE

Calculated power in percent must be within 2% of DDPS calorimetric power. If not, notify the ANPS/NPS.

2. Every Monday complete Data Sheet 2.

Cove
RCO

Aligh
ANPS/NPS

c
Initial

ST. LUCIE UNIT 1
OPERATING PROCEDURE NO. 1-3200020, REVISION 23
PRIMARY SYSTEM MANUAL CALORIMETRIC

DATA SHEET 2
(Page 1 of 1)

NOTE

This data sheet will compare feedwater temperature inputs into DDPS calorimetric with feedwater temperature indications used in manual calorimetric.

1. Record feedwater temperature inputs from DDPS.

PT 301 433.8

PT 302 431.7

PT 303 433.5

Compare PTs 301 through 303 to be within 2°F, if greater than 2°F notify I&C.

PT 304 430.7

PT 305 432.1

PT 306 433.2

Compare PTs 304 through 306 to be within 2°F, if greater than 2°F notify I&C.

2. Record feedwater temperatures from DDPS and from Speedomax.

DDPS PT 389 433.0 Speedomax PT 1 434

Ensure feedwater temperatures to be within 5°F or notify I&C and ANPS.

DDPS PT 390 431.7 Speedomax PT 2 434

Ensure feedwater temperatures to be within 5°F or notify I&C and ANPS.

RCO Come ANPS C. Higgins

Response to NRC Question Regarding 50.59 Screening of TC 1-96-017 to OP 1-0250020,
Boron Concentration Control - Normal Operation

The question raised is with respect to the makeup water flow path. The "normal" method of adjusting boron concentration involves directing the makeup water to the VCT. The subject procedure permits directing flow to the charging pump suction header, downstream of the VCT. Thus, the question is raised if this procedure constitutes a change to the facility or procedures as described in the UFSAR. (Note: The TC involved did not establish the option of directing flow directly to the charging pump suction header, this option existed in the previous revision of the procedure.)

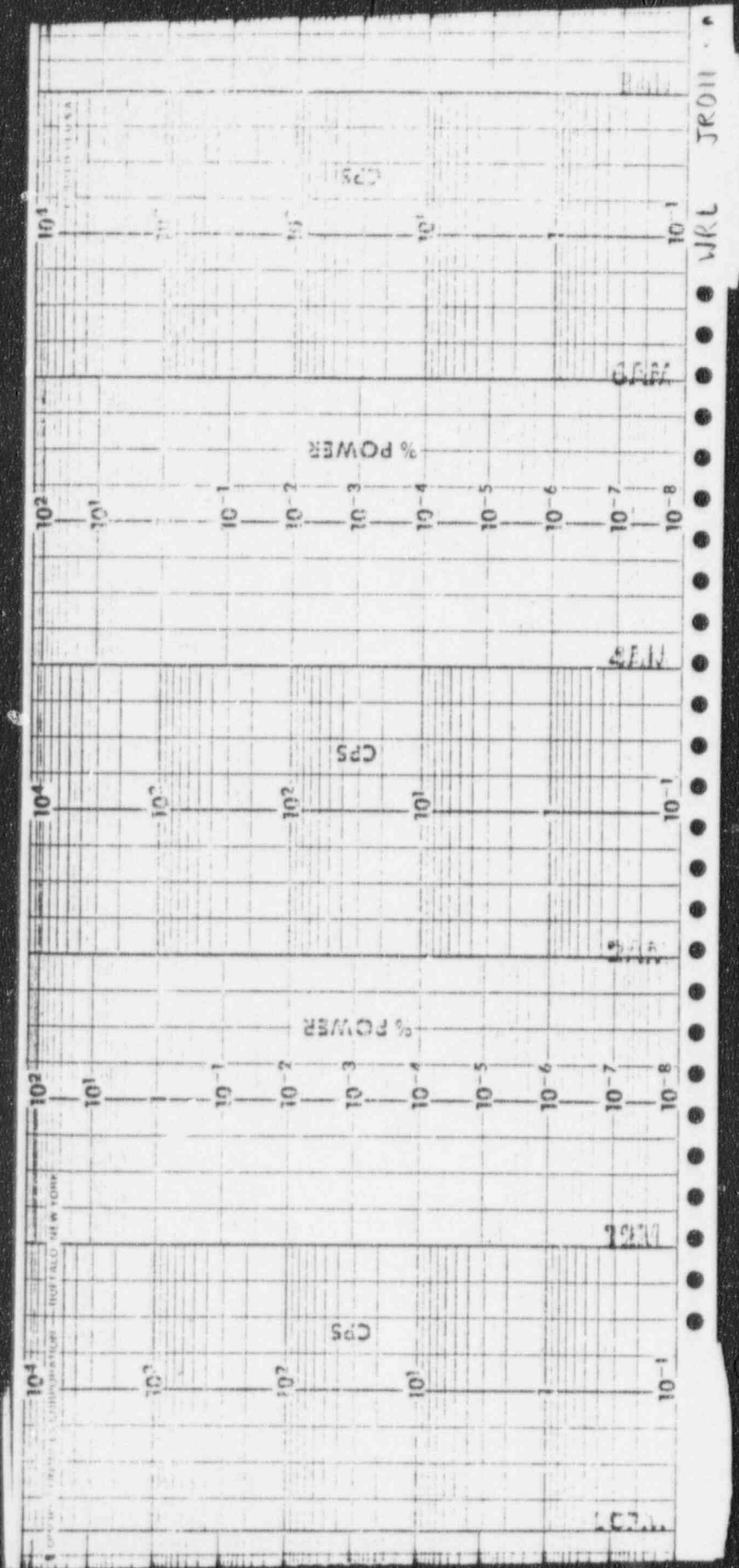
The flow path in question is via existing system piping and valves. The sole purpose of lines 3-CH-640 and 4-CH-639 is to provide for direct introduction of makeup water and/or boric acid solution to the common charging pump suction header. Thus, no change to the facility as described in the UFSAR has been introduced by this procedure.

As described in the UFSAR, there are several different modes of operation for controlling RCS boron concentration, including the manual mode. When operating in the manual mode the makeup flow can either be directed to the RCS via the VCT or it can be supplied directly to the charging pump suction header. Both of these flow paths are shown on UFSAR figure, 9.3-5. By bypassing the VCT and feeding directly to the charging pump suction header, the effects of the addition on RCS boron concentration occur more rapidly.

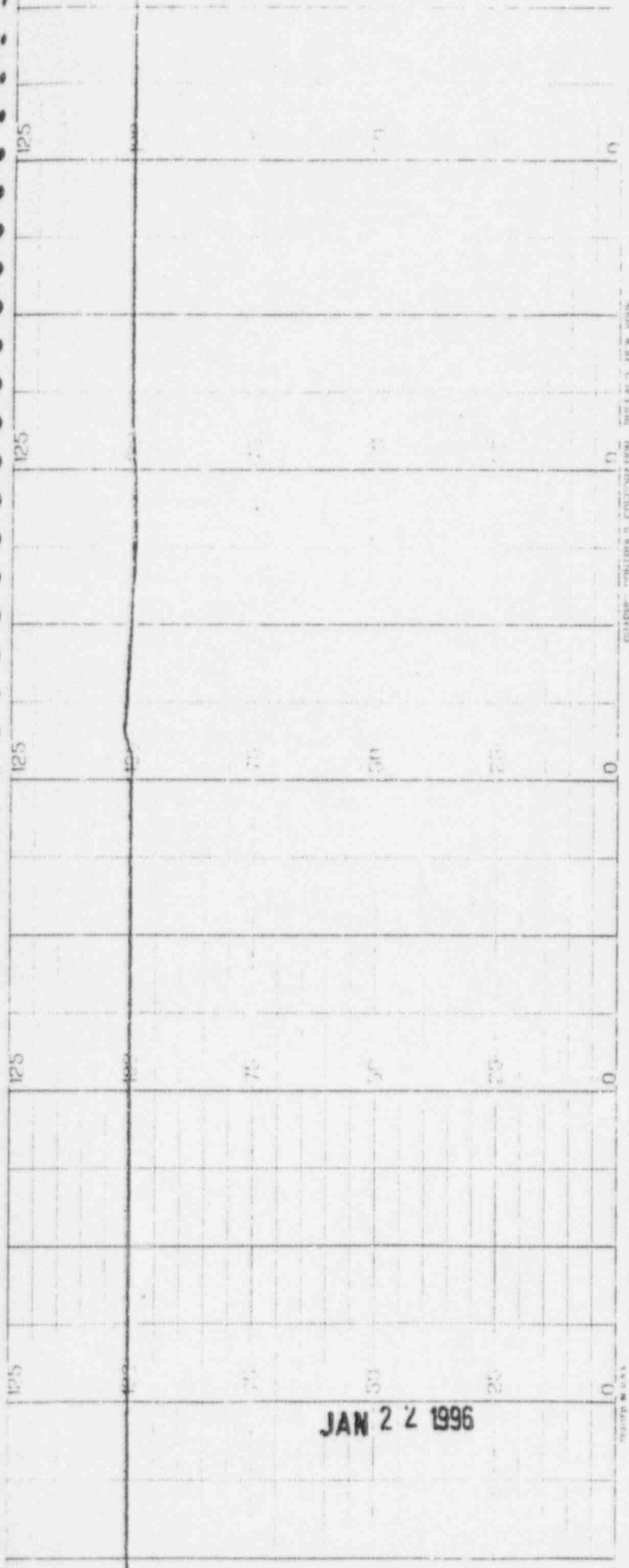
The boron dilution events of UFSAR section 15.2.4 were reviewed. As discussed in section 15.2.4.2.1, each of the six different events analyzed assumed that boron dilution resulted from the direct injection of unborated demineralized water into the RCS at the maximum rate possible (132 gpm = 3 charging pumps). As such, the UFSAR analysis does not assume that makeup flow is being directed through the VCT, since any flow via the VCT would result in a mixing of the makeup flow with the borated contents of the tank. The only means of direct injection of unborated makeup would be through the subject lineup.

The UFSAR chapter 15 analysis notes "Because of the procedures, involved and the numerous alarms and indications available to the operator, the probability of a sustained or erroneous dilution is very low." This statement is valid for all modes of operation of the makeup system. The subject plant procedure provides sufficient controls to ensure proper operation and control of boron concentration. In addition, there are numerous alarms and indications available to operators for any mode of operation.

In conclusion, the subject procedure is consistent with system design and operation as provided in the UFSAR.



MRL JRO11



STATE OF NEW YORK
 OFFICE OF THE COMPTROLLER
 J.K.010

JAN 27 1996

BY
 RAY
 S
 R
 W

ST. LUCIE PLANT
NP-700 PROBLEM REPORT 96-008

I. EVENT TITLE

High Reactor Coolant Temperature and Power Due To Excess Dilution

II. INITIAL PLANT CONDITIONS

Unit 1 was at 100% power, steady state operations.

III. EVENT SEQUENCE

At approximately 0220 on January 22, 1996 reactor fuel burnout resulted in an indicated reactor coolant cold leg temperature of 548.7F by the digital meter on Reactor Turbine Generator Board (RTGB)-104. The board Reactor Control Operator (RCO) decided to dilute the Reactor Coolant System (RCS) in order to restore cold leg temperature to programmed level of 549.0F. He commenced manual dilution in accordance with OP 1-0250020 "Boron Concentration Control - Normal Operation" Section 8.5, with Primary Makeup Water (PMW) directed to the suction of the 1B charging pump at approximately 0225. Shortly after, the board RCO left the RTGB area and went to the kitchen located adjacent to the control room to retrieve his meal. A few minutes later, the board RCO returned to the RTGB area, set his meal on the desk and heard the PMW integrator "clicking". He realized that he was still diluting the RCS and immediately secured the evolution. The RCO commenced borating in accordance with OP 1-0250020 to the suction of the charging pump for a total initial addition of approximately 26 gallons. He simultaneously informed the desk RCO and the Assistant Nuclear Plant Supervisor (ANPS) of his actions. At approximately 0235 annunciator M-16 "RCP CONT BLDOFF PRESS HIGH" alarmed, due to a higher than normal VCT pressure caused by letdown flow aligned to the VCT with charging pump suction aligned to PMW. Indicated RCS cold leg temperature was approximately 549.6F. The ANPS entered a two hour Action Statement to restore RCS cold leg temperature to less than or equal to 549F in accordance with Technical Specification Limiting Condition of Operation (LCO) 3.2.5. At approximately 0242 RTGB-104 indicated RCS cold leg temperature reached its highest value of 549.9F. Indicated plant MegaWatt (MW) reached 885, approximately 4 MW higher than before the dilution, and indicated average reactor power was approximately 100.2%. These levels were sustained for less than four minutes and then reduced as boration affected reactivity. At 0314 indicated cold leg temperature was 549F, and the LCO Action Statement was exited. All other parameters of reactor power and MW reached normal levels concurrently.

DRAFT

IV. CAUSE OF THE EVENT

The cause of this event was cognitive personnel error due to lack of attention to detail by the board RCO performing the dilution. He left the RTGB area with a reactivity evolution in progress.

Additional Deficiencies Noted:

- 1) The RCO performing the dilution left the RTGB area and did not inform the desk RCO or the ANPS that he was diluting the RCS. The "Short Term Turnover" process was less than adequate.
- 2) Expectations for reactivity control for dilution evolutions were not adhered to by the RCO.
- 3) The expectation for notification of Operations Management and Plant Management was not adhered to.

V. CORRECTIVE ACTIONS

1. A self critique was conducted by the control room crew immediately following the event.
2. An independent investigation team led by the Assistant Operations Supervisor gathered all facts associated with the event.
3. The RCO responsible for initiating the dilution was removed from licensed duties based on the results from the fact finding team.
4. The Site Vice President, Plant General Manager and Operations Manager conducted crew meetings with each of the operating crews to ensure all crews are aware of the Unit 1 dilution event. Additionally, excerpts from Zack Pate's "The Control Room" and site management's expectations with respect to conservative operation were discussed. These briefs were completed Friday morning, January 26, 1996.
5. An industry survey was conducted at 9 other nuclear facilities to determine the controls in place for reactivity additions to the reactor coolant system.
6. The "Conduct of Operations" procedure was revised to require direct supervision of reactivity changes by a Senior Reactor Operator which involve dilution to the reactor coolant system and the operation of Chemical Volume and Control System ion exchangers. Also required is a control board walkdown as part of the "Short Term Turnover" process.

7. The Condition Report from the 1993 Turkey Point Plant dilution event was reviewed for lessons learned.
8. Operations Management will personally re-enforce the expectation of communicating required plant events as required by the "Conduct of Operations" procedure. J. West - STAR 960146A - Due 1/30/96
9. The implications of this event will be reviewed by a team for applicability to other operation's activities both inside and outside the control room. J. West - STAR 960146B - Due 2/15/96
10. Plant Management will review its expectations for command and control using information obtained from other sites including Turkey Point. J. West - STAR 960146C - Due 2/15/96
11. St. Lucie's training program will be reviewed for adequacy in the area of reactivity management. W. Bladow - STAR 960146D - Due 1/29/96
12. An HPES evaluation of this event was conducted. A. Locke - STAR 960146E Complete
13. Quality Assurance will conduct an assessment of reactivity management at the St. Lucie Plant. W. Bladow - Complete

NRC QUESTION/RESPONSE FORM

QUESTION #:

DATE:

1/28/96

INSPECTOR:

(CIRCLE APPROPRIATE INSPECTOR'S NAME)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER

BOB SCHINN

UTILITY REPRESENTATIVE:

DIETZ

INSPECTION CATEGORY:

(CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST

DOCUMENTATION

WALKDOWN

PROCEDURES

NRC REQUEST OR CONCERN:

INDICATION/DOCUMENTATION OF "DDPS CALORIMETRIC POWER"
BEFORE, DURING, AND AFTER THE TRIP EVENT.
- SPECIFICALLY, WOULD LIKE DOCUMENTATION OF THE "ACTUAL
REACTOR THERMAL POWER" (PEAK) ACHIEVED DURING THE
EVENT.

RESPONDING INDIVIDUAL/DEPT:

DIETZ 1 LIC

UTILITY RESPONSE:

• PLANT COMPUTER (DDPS) PRINTOUTS PROVIDED
FOR INTERVAL 0200 - 0600, 1/22/96.

• THERE IS NOT PRINTOUT THAT SHOWS "CALORIMETRIC
POWER" AT THE TIME THAT THE PEAK WOULD HAVE
BEEN ACHIEVED.

02-001 1-22-95 F I L I E 1 1 1 5 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0 0

| GROUP | F I L I E (P R I)
09910(12) | F I L I E (P R I)
10910(17) | F I L I E (P R I)
11010(12) | F I L I E (P R I)
10910(12) | F I L I E (P R I)
10910(12) |
|-------|--------------------------------|--------------------------------|--------------------------------|--------------------------------|--------------------------------|
| 1 | 23,06 | 23,19 | 23,56 | 23,80 | 23,80 |
| 2 | 26,75 | 26,75 | 26,75 | 27,30 | 27,30 |
| 3 | 56,50 | 56,30 | 56,30 | DEL | DEL |
| 4 | 58,13 | 56,50 | 56,30 | DEL | DEL |
| 5 | 66,75 | 65,44 | 65,31 | 63,00 | 63,00 |
| 6 | 66,19 | DEL | DEL | 64,59 | 64,59 |
| 7 | 26,75 | 26,50 | 26,50 | 27,44 | 27,44 |
| 8 | 22,94 | DEL | DEL | 23,06 | 23,06 |
| 9 | 56,50 | DEL | 55,81 | 56,30 | 56,30 |
| 10 | 56,63 | 55,81 | 55,44 | 56,81 | 56,81 |
| 11 | 65,56 | 64,44 | 63,81 | 63,43 | 63,43 |
| 12 | 63,25 | 61,75 | 61,19 | 63,00 | 63,00 |
| 13 | 59,19 | 56,50 | 56,13 | 56,80 | 56,80 |
| 14 | 48,69 | 49,44 | 47,63 | 46,33 | 46,33 |
| 15 | DEL | 66,44 | 64,31 | 65,56 | 65,56 |
| 16 | DEL | DEL | DEL | DEL | DEL |
| 17 | 49,06 | 48,13 | 48,00 | 47,17 | 47,17 |
| 18 | 64,69 | 64,81 | 63,17 | 63,17 | 63,17 |
| 19 | DEL | 63,63 | DEL | 62,50 | 62,50 |
| 20 | 21,75 | 21,75 | 21,88 | 21,50 | 21,50 |
| 21 | DEL | 64,69 | 64,69 | 63,50 | 63,50 |
| 22 | 67,38 | 66,44 | DEL | 63,63 | 63,63 |
| 23 | 69,44 | 59,71 | 59,44 | 59,19 | 59,19 |
| 24 | DEL | 65,94 | 65,81 | 64,44 | 64,44 |
| 25 | DEL | DEL | DEL | DEL | DEL |
| 26 | 52,25 | 51,25 | 51,25 | 49,50 | 49,50 |
| 27 | 67,25 | 66,19 | 64,44 | 64,44 | 64,44 |
| 28 | DEL | DEL | DEL | DEL | DEL |
| 29 | 48,31 | 48,13 | DEL | 46,25 | 46,25 |
| 30 | 64,94 | 65,19 | DEL | DEL | DEL |
| 31 | 68,25 | 65,81 | 66,31 | 65,06 | 65,06 |
| 32 | 57,00 | 57,00 | DEL | 55,63 | 55,63 |
| 33 | 58,38 | 56,88 | 56,13 | DEL | DEL |
| 34 | 67,00 | 65,44 | 64,31 | DEL | DEL |
| 35 | 67,63 | 64,69 | DEL | DEL | DEL |
| 36 | 58,13 | 55,69 | 54,63 | 52,25 | 52,25 |
| 37 | 64,56 | 62,63 | DEL | DEL | DEL |
| 38 | DEL | 23,44 | 23,56 | 23,56 | 23,56 |
| 39 | 26,60 | 26,39 | 26,50 | 26,80 | 26,80 |
| 40 | DEL | DEL | DEL | DEL | DEL |
| 41 | 19,50 | 17,19 | 17,19 | 17,19 | 17,19 |

| | | | | | |
|----|-------|-------|-------|-------|-------|
| 1 | 101 | 101 | 101 | 101 | 101 |
| 41 | 57,75 | 50,17 | 50,17 | 57,75 | 57,75 |
| 44 | 25,75 | 25,00 | 25,00 | 25,75 | 25,00 |
| 45 | 25,94 | 25,86 | 25,86 | 25,94 | 25,86 |

02 01 END OF FUDK L06

02 01 1 2 2 045 CEH POSITIONS L06

| | | | | | | | | |
|-----|--------|--------|--------|--------|--------|--------|--------|--------|
| CEH | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 |
| 1 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 |
| 2 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 58 | 60 | 61 | 63 | 64 | 66 | 67 | 69 |
| 3 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 50 | 51 | 52 | 53 | 54 | 55 | 56 | 57 |
| 4 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 2 | 3 | 4 | 5 | | | | |
| 5 | 135,00 | 135,00 | 135,00 | 135,00 | | | | |
| CEH | 14 | 15 | 16 | 17 | | | | |
| 6 | 135,00 | 135,00 | 135,00 | 135,00 | | | | |
| CEH | 1 | 30 | 39 | 40 | 41 | 59 | 62 | 65 |
| 7 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 8 | | | | | | | |

RECURRING GROUP

SHUTTING GROUP

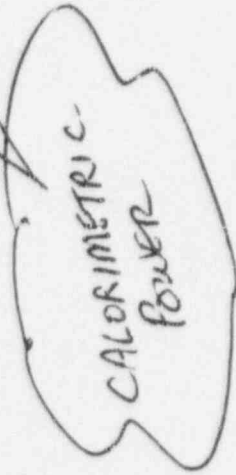
| | | | | | | | |
|-----|--------|--------|--------|--------|--------|--------|--------|
| CEH | 4 | 44 | 45 | 46 | 47 | 48 | 49 |
| 4 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CEH | 6 | 7 | 8 | | | | |
| B | 135,00 | 135,00 | 135,00 | | | | |

02 01 END OF CEH POSITIONS L06

02 01 1/22/95 REACTOR POISON AND LIGHT - 51 L06 II 1011 L06.1

| | | | | | | |
|--------|---------|-------------|---------|------------|------------|---------|
| 1001E | REHOR | REHOR AMETH | 14310.1 | 6095 F06P | 6095 F06P | 1110 |
| COB. % | CONC. % | POH | POBEP | 100%1 000P | 100%1 000P | |
| 101.57 | 100.4 | -272.3 | 192.05 | 93.91 | 93.92 | 2751.25 |

02 01 END OF REACTOR POISON AND POBEP L05



02 57 1/22/96 REACTOR POISON AND POWER LOG
 FLORIDA POWER AND LIGHT - ST LOUIS - UNIT 100.1

| | | | | | |
|---------|-------|---------------|---------|-----------|--------|
| LOGTIME | 2E00 | REACTOR DEPTH | DELTA T | 99% FBR | 1000 |
| CORE % | 100.4 | PCB | POWER | LAST HOUR | 2754.7 |
| 101.57 | | -2722 | 102.08 | 100.00 | |
| | | | | 99% FBR | |
| | | | | LAST HOUR | |
| | | | | 100.00 | |

02 57 END OF REACTOR POISON AND POWER LOG



FLORIDA POWER AND LIGHT - ST. LUCIE PLANT - UNIT 100 - 1

| | | | | | | | | | |
|-------|-----------|---------|----------|--------|--------|--------|--------|--------|--------|
| 51140 | GENERATOR | LETHBRN | PRESSURE | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 |
| | | CHARGE | CHARGE | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 |
| 4573 | B | 1541 | 2750 | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 | 54.108 |

03 00 END OF CALIF METRIC CALCULATION LOG

03 000 1/22/95 FLEX LUGS
 PULVERIZER AND LIGHT - ST. LIGHTE 110000 10000 10000 1

| GROUP | 110 (PHI)
10910(12) | 110 (PHI)
10910(12) | 110 (PHI)
10910(12) | 110 (PHI)
10910(12) | 110 (PHI)
10910(12) | 110 (PHI)
10910(12) |
|-------|------------------------|------------------------|------------------------|------------------------|------------------------|------------------------|
| 1 | 23.19 | 23.44 | 23.69 | 23.94 | 24.19 | 24.44 |
| 2 | 27.00 | 26.88 | 26.88 | 26.88 | 26.88 | 26.88 |
| 3 | 58.81 | 56.63 | 56.50 | 56.50 | 56.50 | 56.50 |
| 4 | 58.50 | 56.75 | 56.50 | 56.50 | 56.50 | 56.50 |
| 5 | 57.13 | 65.81 | 65.56 | 65.56 | 65.56 | 65.56 |
| 6 | 66.63 | DEL | DEL | DEL | DEL | DEL |
| 7 | 26.88 | 26.63 | 26.63 | 26.63 | 26.63 | 26.63 |
| 8 | 23.06 | DEL | DEL | DEL | DEL | DEL |
| 9 | 56.88 | DEL | DEL | DEL | DEL | DEL |
| 10 | 57.00 | 56.00 | 56.00 | 56.00 | 56.00 | 56.00 |
| 11 | 66.06 | 64.81 | 63.44 | 63.44 | 63.44 | 63.44 |
| 12 | 63.63 | 62.13 | 61.38 | 61.38 | 61.38 | 61.38 |
| 13 | 59.56 | 56.75 | 56.25 | 56.25 | 56.25 | 56.25 |
| 14 | 49.06 | 48.69 | 47.75 | 47.75 | 47.75 | 47.75 |
| 15 | DEL | 66.63 | 64.56 | 64.56 | 64.56 | 64.56 |
| 16 | DEL | DEL | DEL | DEL | DEL | DEL |
| 17 | 49.31 | 48.19 | 48.00 | 48.00 | 48.00 | 48.00 |
| 18 | 65.19 | 65.19 | 63.38 | 63.38 | 63.38 | 63.38 |
| 19 | DEL | 63.94 | 63.94 | 63.94 | 63.94 | 63.94 |
| 20 | 71.88 | 71.88 | 71.00 | 71.00 | 71.00 | 71.00 |
| 21 | DEL | 64.94 | 64.81 | 64.81 | 64.81 | 64.81 |
| 22 | 67.88 | 66.75 | 66.75 | 66.75 | 66.75 | 66.75 |
| 23 | 59.81 | 59.69 | 59.56 | 59.56 | 59.56 | 59.56 |
| 24 | DEL | 66.31 | 65.94 | 65.94 | 65.94 | 65.94 |
| 25 | DEL | DEL | DEL | DEL | DEL | DEL |
| 26 | 52.50 | 51.38 | 51.88 | 51.88 | 51.88 | 51.88 |
| 27 | 67.75 | 66.44 | 64.56 | 64.56 | 64.56 | 64.56 |
| 28 | DEL | DEL | DEL | DEL | DEL | DEL |
| 29 | 48.69 | 48.19 | 48.19 | 48.19 | 48.19 | 48.19 |
| 30 | 65.44 | 65.56 | 65.56 | 65.56 | 65.56 | 65.56 |
| 31 | 68.63 | 66.06 | 66.44 | 66.44 | 66.44 | 66.44 |
| 32 | 57.38 | 57.25 | 57.25 | 57.25 | 57.25 | 57.25 |
| 33 | 58.69 | 57.25 | 57.25 | 57.25 | 57.25 | 57.25 |
| 34 | 67.50 | 65.81 | 64.56 | 64.56 | 64.56 | 64.56 |
| 35 | 68.13 | 64.94 | 64.94 | 64.94 | 64.94 | 64.94 |
| 36 | DEL | 65.11 | 65.11 | 65.11 | 65.11 | 65.11 |

| | | | | |
|----|-------|--------|-------|--------|
| 37 | 34.74 | 35.187 | DEL | DEL |
| 38 | DEL | 35.44 | 35.03 | 35.56 |
| 39 | 37.00 | 37.50 | 36.50 | 36.100 |
| 40 | DEL | DEL | DEL | DEL |
| 41 | 67.00 | DEL | 65.31 | DEL |
| 42 | DEL | DEL | DEL | DEL |
| 43 | 58.13 | 56.38 | DEL | 55.56 |
| 44 | 25.00 | 27.00 | 25.75 | 27.00 |
| 45 | 24.06 | 24.69 | 24.06 | 23.69 |

03 01 END OF FLUX LOG

03 01 1/22/95 CEA POSITION LOG
 FLORIDA POWER AND LIGHT - ST. LOUIS PLANT - UNIT 101 - 1

REGULATING GROUP

| | | | | | | | | | |
|-----|--------|--------|--------|--------|--------|--------|--------|--------|--------|
| CEA | 25 | 27 | 28 | 29 | 30 | 31 | 32 | 33 | |
| 1 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | |
| CEA | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 | |
| 2 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | |
| CEA | 58 | 60 | 61 | 63 | 64 | 66 | 67 | 69 | |
| 3 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | |
| CEA | 50 | 51 | 52 | 53 | 54 | 55 | 56 | 57 | |
| 4 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | |
| CEA | 2 | 3 | 4 | 5 | | | | | |
| 5 | 135.00 | 135.00 | 135.00 | 135.00 | | | | | |
| CEA | 14 | 15 | 16 | 17 | | | | | |
| 6 | 135.00 | 135.00 | 135.00 | 135.00 | | | | | |
| CEA | 1 | 38 | 39 | 40 | 41 | 59 | 62 | 65 | 140 |
| 7 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 |

CEA
8

SHUTDOWN GROUP

| | | | | | | | | | |
|-----|--------|--------|--------|--------|--------|--------|--------|--------|--|
| CEA | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 | |
| A | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | 135.00 | |
| CEA | 6 | 7 | 8 | 9 | | | | | |
| B | 135.00 | 135.00 | 135.00 | 135.00 | | | | | |

03 01 END OF CEA POSITION LOG

03 01 1/22/96 REACTOR POSITION AND POWER LOG
 FLORIDA POWER AND LIGHT - ST. LUCIE - UNIT 10.1

| | | | | | | | | | |
|--------|---------|------|-------|---------|--------|-----|------|-------|---------|
| TIME | SECTOR | XERR | DEPTH | DELTA T | RWS | FOR | GRG | FOR | 4100 |
| 101.57 | CONF. 3 | PLR | -7722 | POWER | LAST | FOR | LAST | 24 | 1000.00 |
| | 100.4 | | | 101.45 | 100.13 | | | 99.97 | |

03 01 END OF REACTOR POSITION AND POWER LOG

*CALORIMETRIC-
POWER*

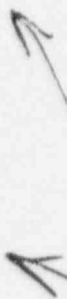


03 37 1/22/96 REACTOR POISON AND POWER LOGS
 FUELION POWER AND LIGHT - ST LUCIF

| TIME | ERRR | RETR DRTH | RETO I | 10% TOR | 10% TOR |
|--------|-------|-----------|--------|---------|---------|
| LOG % | LOG % | PCR | POWER | LOG TOR | LOG TOR |
| 101.59 | 100.4 | 5772 | 101.04 | 100.21 | 100.21 |

03 37 END OF REACTOR POISON AND POWER LOGS

CHALORIMETRIC
 POWER



04 00 1/22/95 CALORIMETRIC DATA LOG
 FLURID4 PUMPER AND LIGHT - ST. LOUIS 10111 100. 1

| | | | | | | | | | | | |
|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|
| PT. 301 | PT. 302 | PT. 303 | PT. 304 | PT. 305 | PT. 306 | PT. 307 | PT. 308 | PT. 309 | PT. 310 | PT. 315 | PT. 317 |
| DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F | DEG. F |
| 432.7 | 431.5 | 432.0 | 430.6 | 431.4 | 432.0 | 487.6 | 486.7 | 495.6 | 497.5 | 600.1 | 598.9 |

| | | | | | | | | | | |
|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|
| PT. 549 | PT. 541 | PT. 518 | PT. 519 | PT. 521 | PT. 522 | PT. 523 | PT. 524 | PT. 525 | PT. 526 | PT. 527 |
| DEB, F | DEB, F | FW | FW | DEB, F | DEB, F | DEB, F | DEB, F | DEB, F | DEB, F | DEB, F |
| 549.4 | 543.4 | DELET | 235.1 | 727.0 | 729.0 | 705.4 | 706.5 | 705.5 | 705.5 | 650.0 |
| | | | | | | | | | | |
| PT. 528 | PT. 529 | PT. 530 | PT. 531 | PT. 532 | PT. 534 | PT. 535 | PT. 536 | PT. 537 | PT. 538 | PT. 539 |
| PS16 | PS16 | PS16 | PS16 | FW | FW | FW | FW | FW | FW | FW |
| 818.5 | 814.4 | 821.3 | 818.9 | 4465 | 4314 | 4469 | 4474 | 4463 | 4463 | 4463 |
| | | | | | | | | | | |
| PT. 571 | PT. 572 | PT. 573 | PT. 574 | PT. 575 | PT. 577 | PT. 578 | PT. 579 | PT. 580 | PT. 581 | PT. 582 |
| 1.0069 | 1.0069 | FW | FW | FW | FW | FW | FW | FW | FW | FW |
| | | 576.9 | 577.1 | 1196.1 | 1196.2 | 1196.2 | 1196.2 | 1196.2 | 1196.2 | 1196.2 |
| | | | | | | | | | | |
| PT. 383 | PT. 384 | PT. 385 | PT. 386 | PT. 389 | PT. 391 | PT. 392 | PT. 393 | PT. 394 | PT. 395 | PT. 396 |
| KB10/H | KB10/H | KB10/H | KB10/H | DEB, F | PS16 | PS16 | KB10/H | KB10/H | KB10/H | KB10/H |
| 1372 | 2768 | 54.46 | 9.189 | 431.0 | 818.9 | 819.1 | 725.3 | 725.3 | 725.3 | 725.3 |

04 00 END OF CALORIMETRIC DATA LOG

04 00 1/22/95 CALORIMETRIC CALORIMETER DATA LOG

04 00 1/22/95 CALORIMETRIC CALORIMETER DATA LOG

04 00 END OF CALORIMETRIC CALORIMETER DATA LOG

04 00 1/22/95 FLUX LMS 0000 100 1

| GROUP | FLUX(PH1)
00010(12) | FLUX(PH2)
00010(12) | FLUX(PH3)
00010(12) | FLUX(PH4)
00010(12) |
|-------|------------------------|------------------------|------------------------|------------------------|
| 1 | 22.94 | 23.19 | 23.56 | 23.81 |
| 2 | 26.63 | 26.63 | 26.75 | 27.31 |
| 3 | 58.25 | 56.25 | 56.38 | DEL |
| 4 | 57.88 | 56.25 | 56.38 | DEL |
| 5 | 66.44 | 65.31 | 65.19 | 63.00 |
| 6 | NA, 11 | DEL | DEL | DEL, 11 |

| 7 | 26,63 | 26,30 | 26,50 | 27,22 |
|----|-------|-------|-------|-------|
| 3 | 22,01 | DEL | DEL | 25,00 |
| 4 | 56,25 | DEL | 55,69 | 55,01 |
| 10 | 56,38 | 55,56 | 55,31 | 55,01 |
| 11 | 65,31 | 64,13 | 63,63 | 63,11 |
| 12 | 67,88 | 61,50 | 61,06 | 63,00 |
| 13 | 58,94 | 56,75 | 56,00 | 56,88 |
| 14 | 48,44 | 48,31 | 47,50 | 46,01 |
| 15 | DEL | 66,19 | 64,19 | 65,56 |
| 16 | DEL | DEL | DEL | DEL |
| 17 | 48,81 | 47,88 | 47,88 | 47,11 |
| 18 | 64,31 | 64,56 | 63,00 | 62,11 |
| 19 | DEL | 63,38 | DEL | 62,50 |
| 20 | 71,63 | 71,63 | 71,88 | 71,31 |
| 21 | DEL | 64,44 | 64,44 | 63,50 |
| 22 | 67,00 | 66,19 | DEL | 63,63 |
| 23 | 58,06 | 58,06 | 58,19 | 59,11 |
| 24 | DEL | 65,69 | 65,56 | 64,41 |
| 25 | DEL | DEL | DEL | DEL |
| 26 | 57,00 | 51,00 | 51,63 | 49,91 |
| 27 | 67,00 | 65,94 | 64,31 | DEL |
| 28 | DEL | DEL | DEL | DEL |
| 29 | 48,13 | 47,88 | DEL | 46,25 |
| 30 | 64,56 | 64,94 | DEL | DEL |
| 31 | 67,88 | 65,56 | 66,06 | 65,06 |
| 32 | 56,75 | 56,88 | DEL | 55,63 |
| 33 | 58,00 | 56,75 | 58,00 | DEL |
| 34 | 66,63 | 65,19 | 64,19 | DEL |
| 35 | 67,25 | 64,44 | DEL | DEL |
| 36 | 57,88 | 55,56 | 54,56 | 52,11 |
| 37 | 64,19 | 62,38 | DEL | DEL |
| 38 | DEL | 73,31 | 73,56 | 73,56 |
| 39 | 26,75 | 26,75 | 26,31 | 26,88 |
| 40 | DEL | DEL | DEL | DEL |
| 41 | 66,31 | DEL | 65,06 | DEL |
| 42 | DEL | DEL | DEL | DEL |
| 43 | 57,50 | 55,94 | DEL | 55,56 |
| 44 | 26,63 | 26,75 | 25,63 | 26,88 |
| 45 | 23,81 | 23,44 | 23,94 | 23,69 |

04 01 END OF FLUX LOG

04 01 1-22-95 GEN POSITION LOGS
 FLOWER HED LIGHT - ST. LOCITE FLOW - 1011 100 1

REGULATING GROUP

| 04 01 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 |
|-------|--------|--------|--------|--------|--------|--------|--------|--------|
| 1 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| 04 01 | 10 | 19 | 29 | 21 | 22 | 23 | 24 | 25 |
| 1 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| 04 01 | 61 | 60 | 61 | 61 | 64 | 64 | 67 | 69 |

| | | | | | | | | |
|-----|---|---------|---------|---------|---------|---------|---------|---------|
| CEA | 1 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 2 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 3 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 4 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 5 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 6 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 7 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |

CONTINUOUS

| | | | | | | | | |
|-----|----|---------|---------|---------|---------|---------|---------|---------|
| CEA | 8 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 9 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CEA | 10 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |

04 01 END OF CEA POSITION LOG

04 01 122505 FUEL FOR POSITION AND POWER LOG

| | | | | | | | | |
|-------|--------|---------------------------------|------|------|------|------|------|------|
| 04 01 | 122505 | FUEL FOR POSITION AND POWER LOG | 100% | 100% | 100% | 100% | 100% | 100% |
| 04 01 | 122505 | FUEL FOR POSITION AND POWER LOG | 100% | 100% | 100% | 100% | 100% | 100% |
| 04 01 | 122505 | FUEL FOR POSITION AND POWER LOG | 100% | 100% | 100% | 100% | 100% | 100% |

04 01 END OF FUEL FOR POSITION AND POWER LOG

CALORIMETRIC

04 34 1/22 95 REACTOR POISON AND TIMER LOGS
FLOPIDA POWER AND LIGHT C. I. LOGS

| | | | | | |
|-------|-------|---------------|---------|---------|---------|
| 1000F | 5000H | REACTOR MIRTH | REACTOR | 90% FOP | 90% FOP |
| 1000F | 5000H | 1000 | 1000 | 1000 | 1000 |

001.57 1000.4 772 101.501 000.00 00.00

04 25 410 08 400.00 400.00 400.00 400.00

04 44 06 * 751 0800 PRESSURIZER PRESS FROM P-1104
04 45 36 751 0800 FROM 0800 PRESSURIZER PRESS FROM P-1104

05 00 1/22/96 COLUPHETIC DATA L05
 FLUORIDA POWER AND LIGHT - SULLOFT-10411-100. 1

PT. 301 PT. 302 PT. 303 PT. 304 PT. 305 PT. 306 PT. 307 PT. 308 PT. 309 PT. 310 PT. 311
 DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F
 432.9 433.8 433.2 432.1 432.2 432.2 432.2 432.2 432.2 432.2 432.2

PT. 340 PT. 341 PT. 342 PT. 343 PT. 344 PT. 345 PT. 346 PT. 347 PT. 348 PT. 349 PT. 350
 DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F DEB.F
 549.0 549.1 549.1 549.1 549.1 549.1 549.1 549.1 549.1 549.1 549.1

PT. 328 PT. 329 PT. 330 PT. 331 PT. 332 PT. 333 PT. 334 PT. 335 PT. 336 PT. 337 PT. 338
 PSDB PSDB PSDB PSDB PSDB PSDB PSDB PSDB PSDB PSDB PSDB
 821.3 821.0 825.1 825.1 822.7 822.7 822.7 822.7 822.7 822.7 822.7

PT. 371 PT. 372 PT. 373 PT. 374 PT. 375 PT. 376 PT. 377 PT. 378 PT. 379 PT. 380 PT. 381
 L.0063 L.0063 L.0062 L.0062 L.0062 L.0062 L.0062 L.0062 L.0062 L.0062 L.0062
 1.0063 1.0063 1.0062 1.0062 1.0062 1.0062 1.0062 1.0062 1.0062 1.0062 1.0062

PT. 383 PT. 384 PT. 385 PT. 386 PT. 387 PT. 388 PT. 389 PT. 390 PT. 391 PT. 392 PT. 393
 FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H FB10-H
 1.526 1.526 1.526 1.526 1.526 1.526 1.526 1.526 1.526 1.526 1.526

OS 00 1/22/96 CALORIMETRIC CALCULATION LOG 100 100 1

FLORIDA POWER AND LIGHT - S.I. LIGHT PLANT - 1001 100 1

| STEAM GENERATOR | LEIGHAN PRESSURIZER | PLATE TOR | COOL TOR | SAPHI TOR | PLATE TOR | COOL TOR |
|-----------------|---------------------|-----------|----------|-----------|-----------|----------|
| 4721 | 1326 | 54,44 | 9,139 | 203,0 | 92,95 | 20,676 |
| B | | PLATE | PLATE | PLATE | PLATE | PLATE |

OS 00 END OF CALORIMETRIC CALCULATION LOG

05 00 1/22/96 FLYK LUG
 FLORIDA POWER AND LIGHT - ST. LOUIS 19 001 - 1011 101. 1

| GROUP | FUDs(PH) | FUDs(PH) | FUDs(PH) | FUDs(PH) | FUDs(PH) |
|-------|-----------|-----------|-----------|-----------|-----------|
| | 10010(12) | 10019(12) | 10010(12) | 10010(12) | 10010(12) |
| 1 | 23.06 | 23.31 | 23.56 | 23.81 | 24.06 |
| 2 | 26.88 | 26.75 | 26.88 | 27.01 | 27.14 |
| 3 | 58.63 | 56.38 | 56.31 | DEL | DEL |
| 4 | 58.13 | 56.50 | 56.50 | DEL | DEL |
| 5 | 66.75 | 65.56 | 65.44 | 65.13 | 64.96 |
| 6 | 66.19 | DEL | DEL | 64.56 | 64.41 |
| 7 | 26.75 | 26.50 | 26.50 | 27.41 | 27.06 |
| 8 | 22.94 | DEL | DEL | 23.06 | 23.06 |
| 9 | 56.50 | DEL | 55.81 | 55.94 | 55.94 |
| 10 | 56.63 | 55.11 | 55.44 | 55.11 | 55.11 |
| 11 | 65.63 | 64.56 | 64.81 | 64.13 | 63.13 |
| 12 | 63.25 | 61.75 | 61.19 | 63.13 | 63.13 |
| 13 | 59.19 | 56.50 | 56.13 | 56.10 | 56.10 |
| 14 | 48.63 | 48.44 | 47.63 | 46.63 | 46.63 |
| 15 | DEL | 66.44 | 64.41 | 65.56 | 65.56 |
| 16 | DEL | DEL | DEL | DEL | DEL |
| 17 | 49.06 | 49.13 | 49.00 | 49.13 | 49.13 |
| 18 | 64.69 | 64.81 | 63.25 | 63.25 | 63.25 |
| 19 | DEL | 63.75 | DEL | 62.50 | 62.50 |
| 20 | 21.75 | 21.75 | 21.88 | 21.90 | 21.90 |
| 21 | DEL | 64.69 | 64.69 | 64.69 | 64.69 |
| 22 | 67.38 | 66.50 | 66.50 | 66.50 | 66.50 |
| 23 | 59.44 | 59.44 | 59.44 | 59.44 | 59.44 |
| 24 | DEL | 65.94 | 65.81 | 64.41 | 64.41 |
| 25 | DEL | DEL | DEL | DEL | DEL |
| 26 | 52.25 | 51.25 | 51.75 | 49.94 | 49.94 |
| 27 | 67.38 | 66.19 | 64.44 | 64.44 | 64.44 |
| 28 | DEL | DEL | DEL | DEL | DEL |
| 29 | 48.31 | 48.13 | 48.13 | 48.25 | 48.25 |
| 30 | 64.94 | 65.31 | 64.13 | 64.13 | 64.13 |
| 31 | 68.25 | 65.81 | 66.31 | 65.06 | 65.06 |
| 32 | 57.13 | 57.00 | 57.00 | 55.00 | 55.00 |
| 33 | 59.38 | 57.00 | 58.13 | DEL | DEL |
| 34 | 67.00 | 65.44 | 64.31 | 64.31 | 64.31 |
| 35 | 67.63 | 64.63 | 64.63 | 64.63 | 64.63 |
| 36 | 58.13 | 55.63 | 54.69 | 54.69 | 54.69 |
| 37 | 64.56 | 62.63 | 61.56 | 61.56 | 61.56 |
| 38 | DEL | 23.44 | 23.56 | 23.06 | 23.06 |
| 39 | 26.109 | 26.78 | 26.50 | 26.109 | 26.109 |
| 40 | DEL | DEL | DEL | DEL | DEL |
| 41 | 96.50 | DEL | 65.13 | DEL | DEL |
| 42 | DEL | DEL | DEL | DEL | DEL |

44
45

25,000
25,000

25,000
25,000

25,000
25,000

05 01 ERD OF FUGS L06

05 01 1/22/80S CER POSSESSION L06
FORBIDDEN FORER L06H1 - S.L. L06H2 - L06H3 - L06H4 - L06H5 - L06H6 - L06H7 - L06H8 - L06H9 - L06H10 - L06H11 - L06H12 - L06H13 - L06H14 - L06H15 - L06H16 - L06H17 - L06H18 - L06H19 - L06H20 - L06H21 - L06H22 - L06H23 - L06H24 - L06H25 - L06H26 - L06H27 - L06H28 - L06H29 - L06H30 - L06H31 - L06H32 - L06H33 - L06H34 - L06H35 - L06H36 - L06H37 - L06H38 - L06H39 - L06H40 - L06H41 - L06H42 - L06H43 - L06H44 - L06H45 - L06H46 - L06H47 - L06H48 - L06H49 - L06H50 - L06H51 - L06H52 - L06H53 - L06H54 - L06H55 - L06H56 - L06H57 - L06H58 - L06H59 - L06H60 - L06H61 - L06H62 - L06H63 - L06H64 - L06H65 - L06H66 - L06H67 - L06H68 - L06H69 - L06H70 - L06H71 - L06H72 - L06H73 - L06H74 - L06H75 - L06H76 - L06H77 - L06H78 - L06H79 - L06H80 - L06H81 - L06H82 - L06H83 - L06H84 - L06H85 - L06H86 - L06H87 - L06H88 - L06H89 - L06H90 - L06H91 - L06H92 - L06H93 - L06H94 - L06H95 - L06H96 - L06H97 - L06H98 - L06H99 - L06H100

PERMITS GROUP

| | | | | | | | | |
|-------|---------|---------|---------|---------|---------|---------|---------|---------|
| CER 1 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CER 2 | 38 | 39 | 40 | 41 | 42 | 43 | 44 | 45 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CER 3 | 58 | 59 | 60 | 61 | 62 | 63 | 64 | 65 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CER 4 | 80 | 81 | 82 | 83 | 84 | 85 | 86 | 87 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CER 5 | 2 | 3 | 4 | 5 | | | | |
| | 175,000 | 175,000 | 175,000 | 175,000 | | | | |
| CER 6 | 14 | 15 | 16 | 17 | | | | |
| | 175,000 | 175,000 | 175,000 | 175,000 | | | | |
| CER 7 | 1 | 28 | 29 | 30 | 31 | 32 | 33 | 34 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |

CER 8

SUPPORT GROUP

| | | | | | | | | |
|-------|---------|---------|---------|---------|---------|---------|---------|---------|
| CER 4 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 |
| | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 | 175,000 |
| CER 8 | 6 | 7 | 8 | 9 | | | | |
| | 175,000 | 175,000 | 175,000 | 175,000 | | | | |

05 01 ERD OF CER POSSESSION L06

OS 01 1/22/96 REACTOR FUELION AND FUELER LUBS
 FLOPIDA FUELER AND LIGHT - ST LUCIE - 0011 0011

| | | | | | | |
|--------|--------|--------------|--------|-----------|-----------|---------|
| 10010E | 10001 | 31001 000000 | 001001 | 0001 000 | 0001 000 | 1100 |
| 10015A | 10002 | 0000 | 000000 | 0001 0000 | 0001 0000 | |
| 101.57 | 1000.4 | -772 | 102.00 | 001.00 | 001.00 | 1000.00 |

OS 01 END OF REACTOR FUELION AND FUELER LUBS

OS 39 1/22/95 PERC FOR FOTSON AND POWEP LOG

FLOTION CUBER AND LIGHT

PERC FOR FOTSON AND POWEP LOG

PERC FOR FOTSON AND POWEP LOG

PERC FOR FOTSON AND POWEP LOG

PERC FOR FOTSON AND POWEP LOG

PERC FOR FOTSON AND POWEP LOG

PERC FOR FOTSON AND POWEP LOG

05 39 1/22/95 PERC FOR FOTSON AND POWEP LOG

OS GO 1/22/96 CALORIMETRIC DATA FOR
FLOTTA POWER AND LIGHT - SLIDE 11 (111) 100. 1

| | | | | | | | | | | | | | |
|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|---------|
| PT. 303 | PT. 305 | PT. 306 | PT. 307 | PT. 308 | PT. 309 | PT. 310 | PT. 311 | PT. 312 | PT. 313 | PT. 314 | PT. 315 | PT. 316 | PT. 317 |
| 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 |
| 433.9 | 433.2 | 433.3 | 433.9 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 | 433.3 |
| 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 | 549.0 |
| PT. 329 | PT. 331 | PT. 333 | PT. 334 | PT. 335 | PT. 336 | PT. 337 | PT. 338 | PT. 339 | PT. 340 | PT. 341 | PT. 342 | PT. 343 | PT. 344 |
| 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 |
| 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 |
| 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 |
| PT. 371 | PT. 372 | PT. 373 | PT. 374 | PT. 375 | PT. 376 | PT. 377 | PT. 378 | PT. 379 | PT. 380 | PT. 381 | PT. 382 | PT. 383 | PT. 384 |
| 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 |
| 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 |
| 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 |
| PT. 384 | PT. 385 | PT. 386 | PT. 387 | PT. 388 | PT. 389 | PT. 390 | PT. 391 | PT. 392 | PT. 393 | PT. 394 | PT. 395 | PT. 396 | PT. 397 |
| 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 | 106.1 |
| 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 | 577.9 |
| 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 | 1066.3 |

06 00 END OF CALORIE TIC DATA LOG

06 00 1/22/96 CALORIMETRIC CALCULATION LOG

STEAM GENERATOR LEADOUT PRESSURE FEEDTOP FEEDTOP FEEDTOP FEEDTOP
 (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP) (HP)

| DATE | TIME | FLUX | FLUX LOG | FLUX LOG | FLUX LOG |
|-------|---------------------------|-------|----------|----------|----------|
| 05 00 | END OF CALIBRATION PERIOD | 0.000 | 0.000 | 0.000 | 0.000 |
| 06 00 | 1/22/96 | 0.000 | 0.000 | 0.000 | 0.000 |
| 07 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 08 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 09 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 10 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 11 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 12 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 13 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 14 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 15 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 16 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 17 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 18 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 19 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 20 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 21 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 22 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 23 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 24 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 25 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 26 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 27 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 28 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 29 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 30 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 31 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 32 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 33 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 34 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 35 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 36 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 37 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 38 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 39 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 40 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 41 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 42 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 43 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 44 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 45 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 46 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 47 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 48 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 49 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 50 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 51 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 52 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 53 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 54 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 55 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 56 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 57 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 58 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 59 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |
| 60 00 | FLUX LOG | 0.000 | 0.000 | 0.000 | 0.000 |

06 00

1/22/96

FLUX LOG

FLUX LOG

FLUX LOG

FLUX LOG

| | | | | |
|----|-------|-------|-------|-------|
| 1 | 25.01 | 25.01 | 25.01 | 25.01 |
| 2 | 26.75 | 26.75 | 26.75 | 26.75 |
| 3 | 52.62 | 52.62 | 52.62 | 52.62 |
| 4 | 58.13 | 58.13 | 58.13 | 58.13 |
| 5 | 65.75 | 65.75 | 65.75 | 65.75 |
| 6 | 68.19 | 68.19 | 68.19 | 68.19 |
| 7 | 26.75 | 26.75 | 26.75 | 26.75 |
| 8 | 22.94 | 22.94 | 22.94 | 22.94 |
| 9 | 56.59 | 56.59 | 56.59 | 56.59 |
| 10 | 58.63 | 58.63 | 58.63 | 58.63 |
| 11 | 65.69 | 65.69 | 65.69 | 65.69 |
| 12 | 63.25 | 63.25 | 63.25 | 63.25 |
| 13 | 59.19 | 59.19 | 59.19 | 59.19 |
| 14 | 48.69 | 48.69 | 48.69 | 48.69 |
| 15 | DEL | DEL | DEL | DEL |
| 16 | DEL | DEL | DEL | DEL |
| 17 | 49.06 | 49.06 | 49.06 | 49.06 |
| 18 | 64.69 | 64.69 | 64.69 | 64.69 |
| 19 | DEL | DEL | DEL | DEL |
| 20 | 21.75 | 21.75 | 21.75 | 21.75 |
| 21 | DEL | DEL | DEL | DEL |
| 22 | 67.38 | 67.38 | 67.38 | 67.38 |
| 23 | 59.44 | 59.44 | 59.44 | 59.44 |
| 24 | DEL | DEL | DEL | DEL |
| 25 | DEL | DEL | DEL | DEL |
| 26 | 52.25 | 52.25 | 52.25 | 52.25 |
| 27 | 67.25 | 67.25 | 67.25 | 67.25 |
| 28 | DEL | DEL | DEL | DEL |
| 29 | 49.31 | 49.31 | 49.31 | 49.31 |
| 30 | 64.94 | 64.94 | 64.94 | 64.94 |
| 31 | 68.25 | 68.25 | 68.25 | 68.25 |
| 32 | 57.13 | 57.13 | 57.13 | 57.13 |
| 33 | 58.38 | 58.38 | 58.38 | 58.38 |
| 34 | 67.00 | 67.00 | 67.00 | 67.00 |
| 35 | 67.63 | 67.63 | 67.63 | 67.63 |
| 36 | 58.13 | 58.13 | 58.13 | 58.13 |
| 37 | 64.56 | 64.56 | 64.56 | 64.56 |
| 38 | DEL | DEL | DEL | DEL |
| 39 | 26.88 | 26.88 | 26.88 | 26.88 |
| 40 | DEL | DEL | DEL | DEL |
| 41 | 56.50 | 56.50 | 56.50 | 56.50 |
| 42 | DEL | DEL | DEL | DEL |
| 43 | 57.75 | 57.75 | 57.75 | 57.75 |
| 44 | 26.75 | 26.75 | 26.75 | 26.75 |
| 45 | 23.56 | 23.56 | 23.56 | 23.56 |

06 01 END OF FL06 L06

06 01 1/22/96 CEH POSITION L06
 FL0604 PAPER AND LIGHT - S1, L06 11 14 0001 0001 1001 1001 1001

PELOP ATHIS GROUP

43 27,79 26,1 25,45
 44 26,75 25,01 24,27
 45 25,34 23,56 22,81

06.01 END OF 1100 L06

005 001 1 22 505 CER FORS F I 000 L 003

REPAIRS (100) GROUP

| | | | | | | | | |
|-------|--------|--------|--------|--------|--------|--------|--------|--------|
| CER 1 | 26 | 27 | 28 | 29 | 30 | 31 | 32 | 33 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 2 | 18 | 19 | 20 | 21 | 22 | 23 | 24 | 25 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 3 | 28 | 29 | 30 | 31 | 32 | 33 | 34 | 35 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 4 | 31 | 32 | 33 | 34 | 35 | 36 | 37 | 38 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 5 | 2 | 3 | 4 | 5 | 6 | 7 | 8 | 9 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 6 | 14 | 15 | 16 | 17 | 18 | 19 | 20 | 21 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 7 | 1 | 2 | 3 | 4 | 5 | 6 | 7 | 8 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 8 | | | | | | | | |

SHORT-DURATION LABOUR

| | | | | | | | | |
|-------|--------|--------|--------|--------|--------|--------|--------|--------|
| CER 4 | 42 | 43 | 44 | 45 | 46 | 47 | 48 | 49 |
| | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 | 135,00 |
| CER 5 | 6 | 7 | 8 | 9 | | | | |
| | 135,00 | 135,00 | 135,00 | 135,00 | | | | |

06.01 END OF CER FORS F I 000 L 003

005-01 1-22-50S 143.45 100% 100% 100% 100% 100% 100% 100% 100% 100%

101.56 100% 100% 100% 100% 100% 100% 100% 100% 100%

101.55 100.3 101.84 90.3% 93.4% 100% 100% 100% 100%

06-03 END OF PEPTIDE FOI508 AND FOI861 LOG

05 03 1/22/96 REACTOR PUTSOUT AND POWER LOG
 FLORIDA POWER AND LIGHT - ST LUCIE - UNIT 1001

| | | | | | |
|---------|---------|-------------|---------|----------|---------------|
| 1001.4 | SENDR | XENON DEPTH | DELTA 1 | 49% PWR | 49% PWR |
| LOP 1.4 | CONC. 3 | FCR | POWER | 1951 HRR | LOST 1.4 HRRS |
| 101.66 | 100.3 | -2719 | 103.13 | 99.97 | 99.97 |

06 04 END OF REACTOR PUTSOUT AND POWER LOG

NRC QUESTION/RESPONSE FORM

QUESTION #: _____

DATE: _____

INSPECTOR: (~~CIRCLE APPROPRIATE INSPECTOR'S NAME~~)

(ADD INSPECTORS NAMES TO THIS FORM)

OTHER BOB S. HINN

UTILITY REPRESENTATIVE: DIETZ

INSPECTION CATEGORY: (CIRCLE APPROPRIATE CATEGORY)

INFO REQUEST DOCUMENTATION WALKDOWN PROCEDURES

NRC REQUEST OR CONCERN:

WHAT IS THE SAFETY ANALYSIS TRANSIENT THAT "BOUNDS" THE
1/22/96 EVENT, AND WHAT VALUES WERE ASSUMED
FOR INITIAL % POWER AND INITIAL RX INLET TEMP?

RESPONDING INDIVIDUAL/DEPT: DIETZ ^{PS/LIC}
(~~CONFIRMED BY WINNARD (SPN) AND KASADI (Nuclear Fuels)~~)

UTILITY RESPONSE: • ATTACHED UPSAR SECTIONS 15.2.4 (CVCS
MALFUNCTION - BORON DILUTION EVENT) AND
DESIGN BASIS DXX (DBD) SECTION 5.1.3.

• SPECIFIC PSL FOR INITIAL INLET TEMPS AND POWER
ARE NOT AVAILABLE - EVENT BOUNDED BY
OTHER MORE LIMITING DNS TRANSIENTS -
PROTECTION FROM SETPOINTS (TM/LP, VAR HI POWER)
VERIFIED FOR THE LIMITING EVENTS EACH
RELOAD CYCLE.

This AOO is comprised of the four possible events which can affect a single steam generator:

- Loss of load to one steam generator
- Excess load to one steam generator
- Loss of feedwater to one steam generator
- Excess feedwater to one steam generator

Each of these events has the potential for creating temperature differences between the four cold legs. This inlet temperature asymmetry would cause radial peaking and corresponding reductions in DNB and centerline to melt (CTM) margins. Of the four possible events the loss of load to one steam generator is the limiting asymmetric event. The loss of load would be caused by closure of a single MSIV. Pressure and temperature in the isolated steam generator rapidly increase to the opening pressure of the steam line safety valves. The turbine control system reacts to maintain speed by drawing more steam from the intact steam generator which leads to decreasing temperature and pressure. The asymmetric steam generator protective trip (ASGPT) is the primary means of mitigating this transient. The low steam generator level trip serves as a backup to ASGPT for this AOO.

The ASGPT function was added as part of the 2700 mW_{th} stretch power license amendment. A separate bistable trip unit (BTU) is not provided for this function. The ASGPT is processed via the TM/LP BTU. Whenever the pressure difference between the two steam generators exceeds 135 psi, the variable low pressure trip setpoint input signal to the TM/LP BTU is automatically increased to 2500 psia. This will result in an immediate trip since pressurizer pressure will be less than the trip setpoint.

Reference: St. Lucie Unit 1 UFSAR, Section 15.2.2

5.1.3 Boron Dilution

Boron dilution events are postulated to occur during all six operational modes. From modes 3-6 the time to achieve criticality due to boron dilution is dependent on the initial and critical boron concentrations, the boron reactivity worth and the dilution rate. From modes 1 and 2 the reactivity insertion rate due to boron dilution is dependent on the boron worth and rate of dilution. The dilution rate is a function of both the boron concentration and the rate at which demineralized water is being injected. The analysis assumes the maximum possible charging rate of 132 gpm with a corresponding letdown flow. For dilution events postulated to occur from subcritical initial conditions all CEAs are conservatively assumed to be withdrawn thus resulting in minimum shutdown margins and time to criticality. One other key assumption is that the boron concentration remains uniform throughout the RCS, and in particular through the core. This assumption is based on the solubility of boric acid, and

the mixing provided by either the shutdown cooling flow or the reactor coolant pumps.

For the subcritical boron dilution events the analysis demonstrates that there is sufficient time to criticality for the operator to recognize and take action to terminate the event. The Standard Review Plan identifies the minimum acceptable time to criticality for refueling events as 30 minutes and 15 minutes for dilution events occurring during all other modes.

For the at power boron dilution events the maximum reactivity insertion rate is small relative to the CEA withdrawal event. The VHPT provides adequate protection for the SAFDLs. Due to the slow rate of power increase the TM/LP trip may occur before the VHPT setpoint is reached. The low steam generator level and LPD trips provide backup protection.

Reference: St. Lucie Unit 1 UFSAR Section 15.2.4

5.1.4 Loss of Coolant Flow

Two reactor coolant pumps (RCPs), one per steam generator, are powered from each of the two 6.9 kV buses. In the event of a turbine generator trip, each 6.9 kV bus fast transfers to the corresponding startup transformer. Loss of more than two RCPs is unlikely. None the less a simultaneous loss of all four RCPs is postulated as the limiting case loss of coolant flow accident. The seized RCP shaft case is analyzed as a separate event and is discussed under the postulated accidents section. A reactor trip on loss of coolant flow is initiated by the steam generator differential pressure transmitters. The rate at which core coolant flow decreases is a critical factor in maintaining an acceptable DNBR margin. A gradual flow coastdown is supported by the high rotational inertia of the RCPs. A time delay of approximately two seconds occurs between loss of RCP power and when the low flow trip setpoint is reached. Protection against exceeding the DNBR limit for this event is provided by the initial steady state thermal margin maintained by compliance with the DNBR related technical specification LCOs, and the low flow reactor trip.

Reference: St. Lucie Unit 1 UFSAR, Section 15.2.5

5.1.5 Loss of Load

During a loss of load event the rapid reduction in the rate of heat removal from the steam generator initially causes a rapid increase in secondary side temperature and pressure. Primary to secondary heat transfer decreases as the temperature differential decreases. Within the first second of the event the RCS cold leg temperature begins to increase with corresponding sharp increases in pressurizer level and pressure. A reactor trip on high pressurizer pressure is generated within the first three seconds, and the safety valve