LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

DECEMBER, 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edision Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982 and commercial power operation was commenced on January 1, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

December 1-31

Dec. 1, 0001 Hours.	The Unit entered December with the reactor subcritical and Unit-off line in cold shutdown for first refuel outage.
Dec. 30, 2400 Hours.	Reactor in Cold Shutdown for First refueling outage.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   Amendment 31- Removed ECCS/RCIC testable check air operated bypass valves.

2. Facility or procedure changes requiring NRC approval.

   There were no Facility or Procedure Changes Requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L35636	LPRM String 24-41A	Failed Detector, Pegged Upscale.	LPRM Upscale	LPRM Detector String to be replaced.
L39494	LPRM String 24-25D	Failed Detector, Pegged upscale.	LPRM Upscale	LPRM detector String to be replaced.
L44671	LPRM String 40-25A	Failed Detector, Pegged Upscale.	LPRM Upscale	LPRM Detector String to be replaced.
L44768	DG Cooling Water Pump 0DG01P	Inboard Pump Packing Leaks Excessively.	The Equipment Drain Being plugged caused excessive water accumulation.	Unplugged equipment drain and inspected pump.
L45391	LPRM String 32-41C	Failed Detector, Pegged Upscale.	LPRM Upscale	LPRM Detector String to be Replaced.
L46931	LPCS Full Flow Test 1E12-F012.	Bad Cap and Bushing on valve.	The Valve stem Cover vibrates loose when flow is through the valve.	Remove old cap and bushing and replaced.
L48509	LPRM String 24-57B	Failed Detector, Pegged Upscale.	LPRM Upscale	LPRM Detector String to be Replaced.
L51947	Stack WRGM 0D18-R522	Bad Action Pak Relay. Loose in its socket.	Effluent Range Recordeder is sporadic and does not track the digital reading.	New retaining clips were installed.
L52447	Div-I Supp. Pool Temperature Recorder 1TR-CM037.	Bad Selector Swith and Range Select cards.	Recorder would intermittently drive downscale.	Replaced selector switch and Range Select cards.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53678	H_2 Recombiner Inlet Valve FV-1	Bad Torque Switch.	The Valve opened but would not close.	Replaced torque switch
L53820	H_2 Recombiner to Suppression Pool Stop Valve 1HG003.	Metal Shavings Present in Valve Housing and Stem	Valve was Mechanically bound, would not operate.	Removed Metal Shavings from Handwheel Components.
L53847	"A" VC Ammonia Detector 0XY-VC125B	Bad fiber optics board and Sample Pump.	Spurious Ammonia Detector Trip.	Replaced Fiber Optics Board and Sample Pump.
L53959	SBGT Inlet Valve 1VG001.	Misadjustment of contact on Torque Switch.	Valve won't electrically open.	Adjusted Torque switch
L54148	H_2 Recombiner X-Tie Valve 1HG009.	Pinched wire under the cover on limit switch.	Fuse for control breaker keeps blowing when breaker energized.	Repaired wire.
L54209	VC Chlorine Detector OAE-VC090A.	Detector would not alarm.	Failure of LIS-VC-01.	Cleaned wick and adjusted Drip Rate.
L54760	VC Damper 0VC13YB.	Misaligned Linkage.	Damper does not close fully.	Adjusted linkage on actuator to close damper.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, December 1 through December 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-066-00	11/06/85	Failed LLRT on feedwater check valve
85-067-00	11/20/85	Spurious chlorine detector actuation.
85-068-00	11/24/85	1E12-F009 valve isolation due to air bubble in sample line.
85-069-00	11/30/85	Surveillance not completed in time.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

5. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE January 10, 1986
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: December 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net): 1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	0:0	5757:5	12039:00
6.	REACTOR RESERVE SHUTDOWN HOURS	0:0	476:2	1642:00
7.	HOURS GENERATOR ON LINE	0:0	5585:35	11642:00
8.	UNIT RESERVE SHUTDOWN HOURS	0:0	0:0	0:0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	0.0	15390361	32213650
10.	GROSS ELEC. ENERGY GENERATED (MWH)	0.0	5028751	10499394
11.	NET ELEC. ENERGY GENERATED (MWH)	-9098	4809395	10004457
12.	REACTOR SERVICE FACTOR	0.0%	65.7%	68.5%
13.	REACTOR AVAILABILITY FACTOR	0.0%	71.2%	77.9%
14.	UNIT SERVICE FACTOR	0.0%	63.8%	66.3%
15.	UNIT AVAILABILITY FACTOR	0.0%	63.8%	66.3%
16.	UNIT CAPACITY FACTOR (USING MDC)	-1.18%	52.9%	54.9%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	-1.13%	50.9%	52.8%
18.	UNIT FORCED OUTAGE RATE	0.0%	17.4%	17.4%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH) The First Refueling, Maintenance, Surveillance and Modification outage began October 18, 1985 and will last 30 weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: May, 1986

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: January 10, 1986
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: DECEMBER 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-15	17.	-15
2.	-15	18.	-15
3	-15	19.	-14
4.	-13	20.	-13
5.	-14	21.	-14
6.	-14	22.	-14
7.	-15	23.	-16
8.	-15	24.	-12
9.	-14	25.	-9
10.	-14	26.	-6
11.	-14	27.	-5
12.	-14	28.	-5
13.	-14	29.	-3
14.	-15	30.	-5
15.	-14	31.	-6
16.	-15		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE JANUARY 10, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH DECEMBER 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS

No Unit Shutdowns or Power Reductions During This Reporting Period.

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-1288-85	1E22-F354	Remove Equipment
1-1296-85	1B DG	Repair 1DO014
1-1302-85	LPCS System	Remove 1E21-F333
1-1303-85	RI (F354)	Remove 1E51-F354/355
1-1322-85	"A" RHR Loop Fill Stop	xfer water to supp. Pool.
1-1341-85	HPCS Inj. Test Stop	Drain Line
1-1370-85	A RHR Hx Ws Side	Hydrolaze Hx.
1-1371-85	RCIC Pump 1E51-C001	Install New Seal
1-1372-85	1E21-F005	LES-EO-112
1-1374-85	1E51-F069	LES-EO-112
1-1375-85	1E51-F064	LES-EO-112
1-1376-85	1E51-F076	LES-EO-112
1-1377-85	1E51-F063	LES-EO-112
1-1378-85	1E51-F008	LES-EO-112
1-1379-85	1E51-F080	LES-EO-112
1-1380-85	1E51-F086	LES-EO-112
1-1381-85	1E51-F068	LES-EO-112
1-1382-85	1E22-F023	LES-EO-112
1-1383-85	1E22-F012	LES-EO-112
1-1384-85	1E22-F010	LES-EO-112
1-1385-85	1E21-F011	LES-EO-112
1-1386-85	1E21-F012	LES-EO-112
1-1387-85	1DG035	LES-EO-112
1-1390-85	1E22-F354	Disconnect Wires
1-1391-85	1E22-F004	LES-EO-112
1-1397-85	"1B" DG	Remove 1E22-F381B for use on 1A DG.
1-1401-85	"0" DG	Instrument Recalibration
1-1402-85	"0" DG	Prelube Mod.
1-1403-85	0DG006	Repack Valve
1-1404-85	"0" DG	Calibrate Protective Relaying
1-1405-85	"0" DG Feed 141Y	Replace sulfer piston
1-1406-85	"1C" RHR Pump	LES-EO-112
1-1407-85	LPCS Pump	LES-EO-112
1-1408-85	HPCS Pump	LES-EO-112
1-1409-85	"1B" DG	Oil and filter change
1-1410-85	HPCS DG	For Mod Package review
1-1413-85	"A" RHR	LES-EO-112
1-1414-85	1E12-F042A	Inspect work gear
1-1415-85	1E12-F004C	LES-EO-112
1-1416-85	1E12-F011A	LES-EO-112
1-1417-85	1E12-F011B	LES-EO-112
1-1418-85	1E12-F017A	LES-EO-112
1-1419-85	1E12-F026A	LES-EO-112
1-1420-85	1E12-F026B	LES-EO-112

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-1421-85	1E12-F042A	LES-EO-112
1-1422-85	1E12-F049A	LES-EO-112
1-1423-85	1E12-F049B	LES-EO-112
1-1424-85	1E12-F052A	LES-EO-112
1-1425-85	1E12-F052B	LES-EO-112
1-1426-85	1E12-F064C	LES-EO-112
1-1428-85	1E12-F068A	LES-EO-112
1-1429-83	1E12-F063B	LES-EO-112
1-1430-85	1E12-F068B	Motor Change Out
1-1431-85	1E12-F073A	LES-EO-112
1-1432-85	1E12-F073B	LES-EO-112
1-1433-85	1E12-F074B	LES-EO-112
1-1434-85	1E12-F074B	LES-EO-112
1-1435-85	1E12-F037A	LES-EO-112
1-1436-85	1E12-F087B	LES-EO-112
1-1437-85	1E12-F093	LES-EO-112
1-1438-85	1E12-F03A/47A	LES-EO-112
1-1444-85	"C" RHR Ws Pp	Throttle Disch.
1-1445-85	"C" RHR Ws Pp	Repair Motor
1-1452-85	1E51-F064	LES-EO-112

3. Off-Site Dose Calculation Manual

There were no significant changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

DECEMBER 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984 and commercial power operation was commenced on June 19, 1984.

This report was compiled by James P. Peters, telephone number (815)357-6761 extension 325.

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II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

November 1-30

Dec. 1, 0001 Hours.	The Unit entered December with the reactor subcritical and Unit-Off Line in a cold shutdown.
Dec. 22, 2110 Hours.	Reactor Sustained Criticality
Dec. 23, 2215 Hours.	Generator Synchronized to Grid, 18% (130 MWe)
Dec. 24, 2300 Hours.	Reactor Power Increased to 50% (444 MWe)
Dec. 25, 0000 Hours.	Entered 8 Hour Time-Clock Due to Containment 0_2 concentration.
Dec. 25, 0045 Hours.	Unusual Event declared, commenced Reactor shutdown.
Dec. 25, 0745 Hours.	Reactor Power at 15% (108 MWe), Main turbine taken off line due to high vibrations.
Dec. 25, 1025 Hours.	Generator Synchronized Back onto the Grid
Dec. 25, 1500 Hours.	Reactor Power Increased to 29% (272 MWe)
Dec. 25, 2300 Hours.	Reactor Power at 45% (455 MWe) Limited by feedwater control problems.
Dec. 26, 2300 Hours.	Reactor Power at 69% (725 MWe)
Dec. 27, 1500 Hours.	Reactor Power at 81% (905 MWe)
Dec. 27, 2000 Hours.	Reactor Power Reduced to Manipulate the Control Rods.
Dec. 27, 2300 Hours.	Reactor Power Restored to 59% (625 MWe).
Dec. 29, 1500 Hours.	Reactor Power Increased to 99% (1105 MWe)
Dec. 30, 2000 Hours.	Commenced Power Reduction to 78% (850 MWe) Due to Leak in 25A Heater.
Dec. 31, 0020 Hours.	25 A Heater Isolated and Power is increased to 82% (900 MWe)
Dec. 31, 2400 Hours.	Reactor Power at 88% (886 MWe).

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   No Amendments Changes Reported during this period.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure change requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L44242	LPRM String 56-41D	Bad connection under vessel on cable.	LPRM Pegged Upscale.	Replaced connection on cables under vessel.
L51318	LPRM, APRM B 2C51-K605 Cu.	LPRM Card 40-17-6B Had a Bad zener diode.	LPRM Downscale Alarm will not reset.	Replaced zener diode.
L51601	RBM A 2C51-K605GU	Bad Transistor on Second Level Multiplexer Card.	With Rod Group 2-2 Selected the APRM A Reads upscale.	Replaced Transistor.
L52460	LPRM String 48-09C.	Bad Transistor on circuit Card.	Core Display on 2H13-P603 shows downscale.	Replaced Transistor.
L53067	Post Loca Monitor 2AIR-CM048.	Misadjusted Flow Regulators.	Control Room indication does not match survey.	Readjusted flow regulators.
L53135	FW Testable Check 2B21-F032A.	The Valve Actuators were blown.	Valve actuation is stuck in Mid-Position.	Replace Air Actuators.
L53162	"B" LPCI Inj. Line Testable Check 2E12-F041B	Bad Air Cylinders and Misaligned open/closed Limit Switches.	Valve would not cycle then no control room indication.	Replaced Air cylinder and reset limit switches.
L53398	RWCU Isolation Timer 2E31-R621A/B	M-1-2-85-065	Prevents the LD Rileys Trips when RWCU isolates.	Installed RC Network across isolation timer motor.
L53451	MSIV Leakage Control Valve 2E22-F001J.	Loose Contacts and Lead Wires on Torque Switch.	Valve would not go full closed.	Tightened and cleaned contacts and wires.
L53503	2A RHR Pump 2E12-C002A.	Bad Relay that feeds the the Alarm.	Pump Auto Trip Light will not Reset.	Replaced Relay.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L53514	H_2 Recombiner Outboard Valve 2HG001B	TSC and TSO Setting Two High.	LLRT, LTS-100-23 Required	Reset TSC and TSO to 1.
L53517	RWCU Outboard Isolation valve 2G33-F004.	TSC setting to High.	LLRT, LTS-100-19 Required.	Reset TSC to 1 1/2.
L53659	MSIV Leakage control Valve 2E22-F003J.	Misadjusted Open contacts on Limit and Torque Switch.	Valve would not go full open, intermediate only.	Cleaned and adjusted open Torque and Limit Switch.
L53712	RHR Cont. Spray 2E12-F016A.	Small Packing Leak.	LLRT, LTS-100-27 Required	Repacked Valve.
L53906	RCIC Vacuum Isolation Valve 2E51-F086.	Internal Mechanical Binding.	Valve Movement in any direction, Torque Switch goes open.	Repacked Valve.
L54058	RCIC Discharge Valve 2E51-F013.	Bad Limitorque Valve.	Removed From U1 to Put in U2.	Replaced Valve.
L54084	VQ Vent and Purge Inlet valve 2VQ029.	Limitorque not positioned correctly.	Valve is leaking past the seat. LLRT, LTS-100-5 Required.	Rotated Limitorque 45° counterclockwise, and cleaned seats.
L54315	2E51-F069 Breaker.	Shorted Torque Switch.	250 VDC Ground when breaker is closed.	Replaced Torque Switch.
L54402	VQ Vent and Purge Inlet Valve 2VQ030.	TSC and TSO set Two High.	LLRT, LTS-100-5 Required.	Reset TSC and TSO to 1 1/4.
L54404	H_2 Recombiner Valve 2HG002A.	Internal Mechanical Binding.	Valve will not go full closed. LLRT, LTS-100-23 Required.	Repacked Valve.
L54411	"A" RHR Pump 2E12-F002A.	Bad Suction Gage	Suction Gage Reads too high.	Replaced and Recalibrated Gage.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L54473	RHR a Full Flow Test Valve 2E12-F024A.	Excessive Flow Through 2E12-F024A Valve.	The Min. Flow Valve will not open.	Set Geared Limit Switches on valve to bypass Torque Switch.
L54528	HPCS DG Cooling Water Pump 2E22-C002.	Breaker Trips.	Requested by NRC For Test Results.	Tested Breaker.
L54549	2VQ036 Purge Valve Breaker.	Pinched Wire on Motor.	Breaker Trips on Magnetics.	Repaired Bad Wire Leads.
L54551	H_2 Recombiner Inlet Valve 2HG002B.	Misadjusted Limited Switch	Valve will only Partially Open.	Reset applicable Limit Switch.
L54557	VQ Outlet Purge Isoltion Valve. 2VQ029.	Misadjusted Limit Switch.	Dual Indication when valve was full closed.	Reset Closed Limit Switch.
L54562	H_2 Recombiner 2HG002A.	Bad Torque Switch Lugs and misadjusted Torque Switch.	Valve will not open or close.	Replaced Torque Switch Lugs and set limits.
L54570	SDV Vent Valve 2C11-F380.	Loose packing at steam.	Valve Leaks are excessively.	Tightened Packing.
L54816	2DC06E MCC 221Y Compt. 3B,3C,5B and 7A.	Thermal Overload Elements, upside down.	Circuit Breaker would not operate on overload trip.	Reinstalled thermal overload elements right side up.
L54827	"B" DG AIR Start Motor 2DG061B.	Bad Valve internals.	When diesel Returned to service air start valve failed to fully close.	Replaced internals. in valve.
L54858	RCIC HI STM Flow Isolation 2E31-N007B.	Improper backfill of HI and Low sensing lines.	RCIC HI Steam flow Isolation During Start-up.	Backfilled the HI and Low sensing lines.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, December 1 through December 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-047-00	11/15/85	Missed Technical Specification sample on "B" RHR Service Water PRM.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE January 10, 1985
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: December 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	218:8	3777:6	5389:4
6.	REACTOR RESERVE SHUTDOWN HOURS	525:2	156i:8	1687.1
7.	HOURS GENERATOR ON LINE	191:1	3698:91	5236:31
8.	UNIT RESERVE SHUTDOWN HOURS	0:0	0:0	0:0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	433833.6	11000960	15508552
10.	GROSS ELEC. ENERGY GENERATED (MWH)	139634	3625083	5110069
11.	NET ELEC. ENERGY GENERATED (MWH)	128462	3430898	4823215
12.	REACTOR SERVICE FACTOR	29.4%	43.1%	51%
13.	REACTOR AVAILABILITY FACTOR	100%	60.9%	67%
14.	UNIT SERVICE FACTOR	25.7%	42.2%	49.6%
15.	UNIT AVAILABILITY FACTOR	25.7%	42.2%	49.6%
16.	UNIT CAPACITY FACTOR (USING MDC)	16.7%	37.8%	26.5%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	16.1%	36.3%	25.5%
18.	UNIT FORCED OUTAGE RATE	74.3%	30.2%	26.8%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): None
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: January 10, 1985
COMPLETED BY: James P. Peters
TELEPHONE: (815) 357-6761
MONTH: December

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-14	17.	-15
2.	-14	18.	-15
3	-15	19.	-13
4.	-13	20.	-12
5.	-13	21.	-13
6.	-13	22.	-14
7.	-14	23.	-5
8.	-15	24.	308
9.	-14	25.	237
10.	-14	26.	468
11.	-13	27.	805
12.	-14	28.	891
13.	-14	29.	1062
14.	-14	30.	1025
15.	-14	31.	863
16.	-14		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE January 10, 1986
COMPLETED BY James P. Peters
TELEPHONE (815)357-6761

REPORT MONTH DECEMBER 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
12.	12/25/85	F	2.7	HI Turbine Vibration	5	Inerted drywell and Increased Power Level. (15% causes excessive vibration).

DOCUMENT 0044r/0005r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT

There were no safety relief valves operated for Unit #2 during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1591-85	2E51-F068	Replace Limitorque Motor
2-1599-85	2E12-F027A	Rotate Motor Mounting
2-1603-85	2E51-F068	Replace Motor
2-1610-85	2E51-F080	Replace Jumper
2-1611-85	2A RHR Pp Motor	Change Oil
2-1626-85	2E51-F069	Troubleshoot Ground
2-1627-85	2E12-F027A	Prevent Inadvertent Draining of Reactor
2-1633-85	2E12-F336A	Replace Contactor
2-1635-85	2E12-F024A	Inspection
2-1643-85	2E12-F004B	Inspection
2-1644-85	2E12-F042B	Inspection
2-1645-85	2E12-F004C	Inspection
2-1646-85	2E12-F016B	Inspection
2-1647-85	2E12-F049B	Inspection
2-1648-85	2E12-F042C	Inspection
2-1649-85	2E12-F064C	Inspection
2-1652-85	2E12-C002C	Inspection
2-1653-85	2E12-C002B	Inspection
2-1672-85	2E51-F091	Inspect MCC Wiring
2-1673-85	2E12-F023	Inspect MCC Wiring
2-1675-85	2E51-F086	Inspection
2-1679-85	2B RHR Pp	Inspection
2-1680-85	2C RHR Pp	Inspection
2-1686-85	"B" DG Clg Wtr. Pp	Test Circuit Breaker
2-1687-85	HPCS	Change Oil, Meggar
2-1689-85	2E51-F019	Repack
2-1690-85	2E51-F069	Repack
2-1691-85	2E51-F045	Repack
2-1695-85	2E51-F045	Troubleshoot and Set Limits
2-1696-85	2E51-F069	Lube and Cycle.
2-1697-85	LPCS	Meggar
2-1700-85	2A RHR	Meggar
2-1711-85	2A DG	Repair Air Line
2-1717-85	2A DG	Repair 2DG061B
2-1719-85	2B DG Clg Water Pp.	Repair Motor controller
2-1723-85	RCIC Water Leg Pp	Inspection
2-1725-85	RCIC	Prevent Auto Initiation
2-1729-85	2E51-F066	Prevent Scram
2-1730-85	2CB05AA	Repair Weld

3. Off-Site Dose Calculation Manual

There were no changes to the ODCM during this reporting period.

4. Radioactive Waste Treatment Systems.
There were no changes to the radioactive waste treatment system during this reporting period.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

January 9, 1986

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period December 1, 1985 through December 31, 1985.

Very truly yours,

G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
L. J. Anastasia, PIP Coordinator SNED
J. E. Ellis, GE Resident
J. M. Nowicki, Asst. Comptroller
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle

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