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J-7.

The bus bar failure was found to be similar to the fracture of "B" phase which occurred on "C" RCP during December 15, 1983. The exact cause for these failures could not be determined.

All the main load connection bus bars will be replaced in "C" Reactor Coolant Pump prior to the unit restart.

PDR

Updated	Report-Previous	Report	Dated	10	26/84
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POW 28-06-01

U.S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVE	NT REPORT (LER) TEXT CONTINU	EXT CONTINUATION APPROVED OMB NO. 3150- EXPIRES: 8/31/85					
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6	PAGE (3)				
	A REAL REAL AND AND AND	YEAR SEQUENTIAL NUMBER	NUMBER		T		
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EXT (If more space is required, use additional NRC Form 386A's) (1)							
REACTO	TRIP ('C' REACTOR COOL	ANT PUMP)					
I Description of the Fre							
1. Description of the Eve	int						
On September 26 was initiated when "C instantaneous ground:	, 1984, at 2227, with unit " RCP breaker opened. The ng of all phases.	l at 80% power breaker trippe	, a read d due to	ctor ti	rip		
Following the mactuate, however, the	eactor trip, the Rod Bottom IRPI indication worked cor	Light for Rod rectly.	J-7 die	d not			
All other prote	ction and control systems w	ere noted to f	unction	prope	cly.		

All other protection and control systems were noted to function properly. Operators followed appropriate plant procedures and stabilized the plant following the reactor trip.

2. Safety Consequences and Implications

Technical Specifications require that a sufficient number of reactor coolant pumps be operating to provide coastdown core cooling flow in the event of a loss of reactor coolant flow accident. The loss of one pump from a nominal reactor coolant system heat output of 100% (2441 MWt) with three loops operating is an analyzed event in the Design Basis of Technical Sepcifications. Further more, the reactor protection logic is designed to maintain sufficient margin above a DNBR of 1.30, with loss of RCS flow.

Technical Specifications require the rod position indication system be operable and capable of determining the control rod positions within \pm 12 steps. In addition, all other safety related systems remained operable during the event and plant parameters remained within the bounds of the accident analysis. Therefore, this event did not constitute an unreviewed safety question nor affect the health and safety of the public.

3. Cause

The reactor trip was due to the 'C' RCP Trip on instantaneous ground fault on all phases. A preliminary inspection of the motor leads has revealed a complete separation of "A" phase main load connection bus bar. This failure initiated the instantaneous ground fault on all phases. The exact cause for this failure could not be determined. A visual inspection, liquid dye penetrant test, was performed on all phases for a'l pumps during the outage.

The failure of the rod bottom light for Rod "J-7" was due to a sticking relay.

Updated Report-Previous Report Dated 10/26/84

POW 28-06-01

(9-83) LICENSEE EVENT R	EPORT (LER) TEXT CONTINU	JATION APPROVED O EXPIRES 8/3	ULATORY COMMISSION MB NO. 3150-0104 1/85	
FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)	PAGE (3)	
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REACTOR TR	IP ('C' REACTOR COOL	ANT PUMP)		
Operators performed Restoration Procedures to In addition, the RPI syste	all appropriate Emerge ensure the plant was r em gave immediate and c	ncy Procedures and Fun eturned to a stable co orrect indication of F	nction ondition. Rod "J-7"	

Also, the STA performed the critical safety function status tree reviews to ensure specific plant parameters were noted and the appropriate procedures were used to maintain those parameters within safe bounds.

5. Additional Corrective Actions

All main load connection Bus Bars were replaced in "C" RCP. The relay controlling the rod bottom light was reseated and cycled satisfactory. No additional corrective actions are deemed necessary for the rod bottom light relay.

6. Action Taken to Prevent Recurrence

Due to the extent of damage of the bus bar, a meaningful failure analysis could not be performed.

7. Generic Implications

position.

A similar failure occurred on "C" RCP motor on "B" phase main load connection Bus Bar on December 15, 1983.

Vepco

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VIRGINIA ELECTRIC AND POWER COMPANY

Surry Power Station P. O. Box 315 Surry, Virginia 23883

September 5, 1985

U. S. Nuclear Regulatory Commission Document Control Desk 016 Phillips Building Washington, D. C. 20555

Serial	No:	84-35A
Docket	No:	50-280
License	No:	DPR-32

Gentlemen:

Pursuant to Surry Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report update for Surry Unit 1.

REPORT NUMBER

84-020-01

This report has been reviewed by the Station Nuclear Safety and Operating Committee and will be reviewed by Safety Evaluation and Control.

Very truly yours,

R. F. Saunders Station Manager

Enclosure

cc: Dr. J. Nelson Grace Regional Administrator Suite 2900 101 Marietta Street, NW Atlanta, Georgia 30323

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