Virginia Electric and Power Company North Anna Power Station P. O. Box 402 Mineral, Virginia 23117

November 6, 1996

U. S. Nuclear Regulatory Commission Document Control Desk Washington, D.C. 20555 NAPS: MPW Docket Nos. 50-338 50-339 License Nos. NPF-4 NPF-7

Dear Sirs:

Pursuant to North Anna Power Station Technical Specifications, Virginia Electric and Power Company hereby submits the following Licensee Event Report applicable to North Anna Units 1 & 2.

Report No. 50-338/96-009-00

This Report has been reviewed by the Station Nuclear Safety and Operating Committee and will be forwarded to the Management Safety Review Committee for its review.

Very truly yours,

W Rufathe

W. R. Matthews Station Manager

Enclosure:

cc: U.S. Nuclear Regulatory Commission 101 Marietta Street, N.W. Suite 2900 Atlanta, Georgia 30323

> R. D. McWhorter NRC Senior Resident Inspector North Anna Power Station

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NCR	FOR	M	366	(5-	92)

LICENSEE EVENT REPOR	APPROVED BY OMB NO. 3150-0104 EXPIRES 5/31/95 ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWARD COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (MNBB 7714). U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET, WASHINGTON, DC 20503.						
FACILITY NAME (1)							
		YEAR	SEQUENTIAL	REVISION	TAGE (0		
North Anna Power Station Units 1 & 2	05000338	96	009	00	2 OF	3	
TEXT (If more space is required, use additional copies of NRC	Form 366A) (17)	A		terre constant of the second		in and so the	
1.0 Description of the Event							
As a consequence of the Salem ATV established the requirement for the a reactor trip breakers. Subsequently, concluded that Technical Specification changes should explicitly require ind attachments during power operation manual switch contacts during each Amendment Nos. 81 and 69 were ap Power Station, Units 1 & 2, implement definition of reactor trip breaker oper requirements for the reactor trip and scram switches. On October 10, 1996, it was determine System- JD, Component-CL) had pre- breakers (EIIS System-JD, Component manual shunt trip was tested. In this for a very short time and connected to testing the local manual shunt trip. To placing the bypass breaker in service	VS event, Item 4.3 of automatic actuation Item 4.3 became the on (TS) changes she ependent testing of and independent terefueling outage. The proved, June 9, 190 hting the GL criteria ability and clarificat reactor trip by pass ned that testing of the eviously been condu- ent-BKR) racked in a configuration the b to the rod control sy Therefore, operabilities.	of Gene of the subju- ould be the und esting of echnics 86, by the loca ucted w and mo- bypass l ystem (I	eric Letter (C shunt trip att ect of GL 85 proposed. dervoltage a f the contro al Specificat the NRC fo changes pro- changes pro- changes pro- turveillance ers, and the l manual sh with the reac omentarily c breaker is a EIIS System not being ve	GL) 83-28, tachment fo 5-09 and The and shunt the room tion r North Anno by ided furth test manual funt trip (Ell tor trip byp losed befor ctually in se or fied prior	or rip na er S ass e the ervice to to		
This event posed no significant safet breaker manual shunt trip is proven breaker during solid state protection public were not affected at any time	v implications beca operable immediate testing. Therefore, during this event.	use the ely upor the he	e reactor trip closing the alth and saf	bypass bypass ety of the			
Since the reactor trip bypass breake this event is reportable pursuant to 1 the plant Technical Specifications.	rs were not tested p 0CFR50.73(a)(2)(i)	orior to (B) for	placing then a condition	n in service prohibited b	ру		
3.0 Cause of the Event							
The cause of the event has been der which did not require operability verif in service. The surveillance procedu following the approval to change the open." When TS Amendment Nos. 8 surveillance procedures were determ	termined to be inad ication prior to placi ire test sequence w normal position of 31 and 69 were app nined not necessary	equate ing the vas revis the byp proved i y. The	surveillance reactor trip sed in Febru ass breake n June 198 probable ca	e test proce bypass bre uary 1985, r to "racked 6, changes use for not	edures akers I in and to the		

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LICENSEE EVENT REI	ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THI INFORMATION COLLECTION REQUEST: 50.0 HOURS. FORWATIO COMMENTS REGARDING BURDEN ESTIMATE TO THE INFORMATIO AND RECORDS MANAGEMENT BRANCH (MNBB 7714). U.S. NUCLEA REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND T THE RAPERWORK BEDICTION DECATION AND AND THE						
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North Anna Power Station Units 1 & 2	05000338	96	NUMBER 009	NUMBER 00	3 OF 3		
TEXT (If more space is required, use additional copies revising the test procedures wa applied to the bypass breakers. Reactor Trip Breaker Testing, a interpretation of the term "in set 4.0 Immediate Corrective Additional Corrective Additite Additite Additional Additional Corrective Additional	s of NRC Form 366A) (17) as a mis-interpretation of a subsequent revision also failed to note the im- arvice." ctions ne station deviating repor- ctions tor trip bypass breaker to prior to placing them in s equence for testing the in urrence	the term to UFS, proper to ting sys esting wa ervice. reactor t	as revised t The UFSAF	e" and how 7.2.2.2.1.7 ce due to th R will be breakers.	it , ne mis-		
None							