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ComEd

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January 15, 1997

U.S. Nuclear Regulatory Commission Document Control Desk Washington, D. C. 20555

Subject: ComEd Braidwood Station Unit 2 Fifth Refuel Outage Steam Generator Inservice Inspection Report Docket No. STN 50-457

References: (1) NUREG-1276, Technical Specifications, Braidwood Station, Unit Nos. 1 and 2

Specification 4.4.5.5.b of reference (1) requires that within 12 months following the completion of each inservice inspection of steam generator tubes, the complete results of the inspection shall be submitted to the Commission in a Special Report pursuant to Specification 6.9.2. This report shall include the number and extent of tubes inspected, location and percentage of wall-thickness penetration for each indication of an imperfection, and identification of tubes plugged or repaired. The Braidwood Unit 2 steam generator eddy current inspection was completed on April 12, 1996.

During this inspection, 100% of the tubing received a full length bobbin coil eddy current inspection. Also, the inspection included a Rotating Pancake Coil (RPC) inspection of 28% of the hot-leg tep-oftubesheet roll transition regions, 100% of the Row 1 and Row 2 U-Bend regions, 20% of the tubes expanded at Preheater Baffles 'B' and 'D' in the 2A Steam Generator, and 64% of the dents greater than 5 volts at the hot-leg tube support plates. All examinations exceeded the minimum requirements of the Technical Specifications and commitments made in the response to Generic Letter 95-3, Circumferential Cracking of Steam Generator Tubes.

Enclosed is a summary of the inspection results. Included are a guide to the abbreviations used in the indication list, indication lists and maps, and the qualification requirements of the certified personnel performing the eddy current examinations.

Please direct any questions regarding this submittal to Doug Huston, Braidwood Licensing Supervisor, (815) 458-2801, extension 2511.

Very truly yours. J. Tulon

Station Manager Braidwood Nuclear Station

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Enclosure

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