



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D. C. 20555

August 9, 1985

Docket No. 50-219
LS05-85-08-013

Mr. P. B. Fiedler
Vice President and Director
Oyster Creek Nuclear Generating Station
Post Office Box 388
Forked River, New Jersey 08731

Dear Mr. Fiedler:

SUBJECT: INSTRUMENTATION FOR DETECTION OF INADEQUATE CORE COOLING, GENERIC
LETTER 84-23 AND TMI ACTION PLAN ITEM II.F.2

Re: Oyster Creek Nuclear Generating Station

Generic Letter (GL) 84-23, "Reactor Vessel Water Level Instrumentation in BWRs," identified two categories of potential improvements in BWR water level instrumentation which would give increased assurance that this instrumentation will provide the inadequate core cooling instrumentation required by NUREG-0737, Item II.F.2, and thereby satisfy this NUREG-0737 requirement. These improvements are: (1) improvements that will reduce level indication errors caused by high drywell temperature and (2) replacement of the mechanical level indication equipment with analog level transmitters unless operating experience confirms high reliability.

By letters dated December 18, 1984, and May 17, 1985, and a meeting with the staff on February 14, 1985, GPU Nuclear Corporation (the licensee) responded to GL 84-23. By this meeting and the letter dated May 17, 1985, the licensee stated that the existing fuel zone water level instrumentation is density compensated up to saturation conditions, that when saturation is detected in the reference leg this instrumentation turns itself off and that the thermal effects of nearby steam and feedwater lines on the reference leg are minimal, and, therefore, the instrumentation readings are representative of the warmer fluid in the reference leg. This existing fuel zone water level instrumentation is acceptable to the staff as the means of implementing the first improvement described in GL 84-23.

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In response to the second improvement, the licensee stated in its letter dated December 18, 1984, that they planned to replace existing mechanical level indication equipment (Yarway indicators and Barton switches) during the next refueling outage at Oyster Creek. The licensee stated that preliminary engineering studies for this replacement indicate the use of either local level transmitters with a remote analog trip system or new switches with improved performance characteristics such as Static-O-Ring or other comparable model. This is acceptable to the staff as means of implementing the second improvement described in GL 84-23.

Based on the above, the licensee has provided acceptable means of implementing the two categories of potential improvements identified in GL 84-23. Therefore, the existing water level instrumentation at Oyster Creek with the replacement of mechanical level indication equipment during the next refueling outage meets the requirements of NUREG-0737 Item II.F.2. The issuance of this letter closes out the staff's action on NUREG-0737, Item II.F.2, and GL 84-23.

Sincerely,

Original signed by:

John A. Zwolinski, Chief
Operating Reactors Branch No. 5
Division of Licensing

cc: See next page

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Mr. P. B. Fiedler
Oyster Creek Nuclear Generating Station

Oyster Creek Nuclear
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