



**OPPD**

Omaha Public Power District  
444 South 16th Street Mall  
Omaha NE 68102-2247

January 10, 1997  
LIC-97-001

U. S. Nuclear Regulatory Commission  
Attn: Document Control Desk  
Mail Station P1-137  
Washington, DC 20555

- References:
1. Docket No. 50-285
  2. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated June 23, 1993 (LIC-93-0119)
  3. Letter from OPPD (W. G. Gates) to NRC (Document Control Desk) dated January 13, 1994 (LIC-93-0304)
  4. Letter from OPPD (T. L. Patterson) to NRC (Document Control Desk) dated June 12, 1995 (LIC-95-0123)

SUBJECT: Average Load Capacity Factor for Fort Calhoun Station (FCS) Cycle 16

The Omaha Public Power District (OPPD) monitors the long term load capacity factor for FCS on a cycle-by-cycle basis to ensure that the current fluence/pressurized thermal shock (PTS) and other fluence dependent analyses remain valid. In Reference 3, OPPD committed to notify the NRC of the average load capacity factor achieved since the start of Cycle 15. Reference 4 provided data for Cycle 15. The load capacity factor for Cycle 16 operation (including the refueling outage) was 0.82.

A revised fluence evaluation utilizing the ENDF-B/VI cross sections (per Reference 2) was completed in 1995. This evaluation, in conjunction with the load capacity factors of completed fuel cycles and projected capacity factors of future cycles, continues to be utilized to assess the projected end of life 10 CFR 50.61 RT<sub>95</sub> value. These projections assume future fuel cycle load capacity factors of 0.85.

The pressure-temperature (P-T) limit curves and the reactor vessel beltline materials have been reviewed based on the revised fluence evaluation and actual operation through the end of Cycle 17. The current Technical Specification P-T limits are still valid for Cycle 17 operation based on the fluence values used in the analysis for the 20 Effective Full Power Years (EFPY), as approved in Amendment 114 to Facility License No. DPR-40. The welds and plate material in the reactor vessel beltline region are projected to remain below the 10 CFR 50.61 PTS screening criteria through the end of licensed operation.

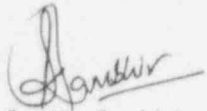
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Please contact me if you have any questions.

Sincerely,



S. K. Gambhir  
Division Manager  
Production Engineering

TCM/tcm

c: Winston & Strawn  
L. J. Callan, NRC Regional Administrator, Region IV  
L. R. Wharton, NRC Project Manager  
W. C. Walker, NRC Senior Resident Inspector