

TABLE 15.4.1-2 (Continued)

| | <u>Test</u> | <u>Frequency</u> |
|---|--|-------------------------|
| 24. Integrity of Post Accident Recovery Systems Outside Containment | Evaluate | Yearly |
| 25. Containment Purge Supply and Exhaust Isolation Valves | Verify valves are locked closed | Monthly (9) |
| 26. Reactor Trip Breakers | a. Verify independent operability of automatic shunt and undervoltage trip functions. | Monthly (9) |
| | b. Verify independent operability of manual trip to shunt and undervoltage trip functions. | Each refueling shutdown |
| 27. Reactor Trip Bypass Breakers | a. Verify operability of the undervoltage trip function. | Prior to breaker use |
| | b. Verify operability of the shunt trip functions. | Each refueling shutdown |
| | c. Verify operability of the manual trip to undervoltage trip functions. | Each refueling shutdown |
| (1) Required only during periods of power operation. | | |
| (2) E determination will be started when the gross activity analysis of a filtered sample indicates $\geq 10\mu\text{Ci/cc}$ and will be redetermined if the primary coolant gross radioactivity of a filtered sample increases by more than $\geq 10\mu\text{Ci/cc}$. | | |
| (3) Drop test shall be conducted at rated reactor coolant flow. Rods shall be dropped under both cold and hot condition, but cold drop tests need not be timed. | | |
| (4) Drop tests will be conducted in the hot condition for rods on which maintenance was performed. | | |
| (5) As accessible without disassembly of rotor. | | |
| (6) Not required during periods of refueling shutdown. | | |
| (7) At least once per week during periods of refueling shutdown. | | |
| (8) At least three times per week (with maximum time of 72 hours between samples) during periods of refueling shutdown. | | |
| (9) Not required during periods of cold or refueling shutdown. | | |
| (10) During end-of-cycle period of operation when boron concentration is less than 100 ppm, this test may be waived due to operational limitations. | | |
| (11) Sample to be taken after a minimum of 2 EFPD and 20 days power operation since the reactor was last subcritical for 48 hours or longer. | | |

15.6.8 PLANT OPERATING PROCEDURES (Continued)

15.6.8.4 The following programs shall be established, implemented, and maintained.

A. Post-Accident Sampling*

A program**which will ensure the capability to obtain and analyze reactor coolant, containment atmosphere, and in-plant gaseous effluent samples under accident conditions. The program shall include the following:

- (i) Training of personnel;
- (ii) Procedures of sampling and analysis; and
- (iii) Provisions for maintenance of sampling and analysis equipment.

*Post-Accident Coolant Sampling and Post-Accident Containment Atmospheric Sampling Systems.

**It is acceptable if the licensee maintains details of the program in plant operation manuals.

G. Failure of Containment High-Range Radiation Monitor

A minimum of two in-containment radiation-level monitors with a maximum range of 10^8 rad/hr (10^7 /hr for photons only) should be operable at all times except for cold shutdown and refueling outages. This is specified in Table 15.3.5-5, item 7. If the minimum number of operable channels are not restored to operable condition within seven days after failure, a special report shall be submitted to the NRC within thirty days following the event outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to operable status.

H. Failure of Main Steam Line Radiation Monitors

If a main steam line radiation monitor (SA-11) fails and cannot be restored to operability in seven days, prepare a special report outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the channel to operable status within thirty days of the event.