NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

# LICENSEE EVENT REPORT (LER)

| FACILITY NAME (1) | DOCKET NUMBER (2) | PAGE (3) |
|---|---|---|
| R.E. Ginna Nuclear Power Plant | 0 5 0 0 0 2 4 4 | 1 OF 0 3 |

TITLE (4)
Inoperable rod position indicating system

| EVENT DATE (5) | | | LER NUMBER (6) | | | REPORT DATE (7) | | | OTHER FACILITIES INVOLVED (8) | |
|---|---|---|---|---|---|---|---|---|---|---|
| MONTH | DAY | YEAR | YEAR | SEQUENTIAL NUMBER | REVISION NUMBER | MONTH | DAY | YEAR | FACILITY NAMES | DOCKET NUMBER(S) |
| | | | | | | | | | | 0 5 0 0 0 |
| 0 9 | 1 5 | 8 5 | 8 5 | — 0 1 6 | — 0 0 | 1 0 | 1 4 | 8 5 | | 0 5 0 0 0 |

| OPERATING MODE (9) | | THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11) | | | |
|---|---|---|---|---|---|
| N | | 20.402(b) | 20.405(c) | 50.73(a)(2)(iv) | 73.71(b) |
| POWER LEVEL (10) | | 20.405(a)(1)(i) | 50.36(c)(1) | 50.73(a)(2)(v) | 73.71(c) |
| 1 0 0 | | 20.405(a)(1)(ii) | 50.36(c)(2) | 50.73(a)(2)(vii) | OTHER (Specify in Abstract below and in Text, NRC Form 366A) |
| | | 20.405(a)(1)(iii) | X 50.73(a)(2)(i) | 50.73(a)(2)(viii)(A) | |
| | | 20.405(a)(1)(iv) | 50.73(a)(2)(ii) | 50.73(a)(2)(viii)(B) | |
| | | 20.405(a)(1)(v) | 50.73(a)(2)(iii) | 50.73(a)(2)(x) | |

LICENSEE CONTACT FOR THIS LER (12)

| NAME | TELEPHONE NUMBER |
|---|---|
| E. Clair Edgar, I&C Supervisor | AREA CODE 3 1 5 — 5 2 4 - 4 4 4 6 |

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

| CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | | CAUSE | SYSTEM | COMPONENT | MANUFACTURER | REPORTABLE TO NPRDS | |
|---|---|---|---|---|---|---|---|---|---|---|---|
| X | I U | - R J X | P 3 1 4 | Y | | | | | | | |
| | | | | | | | | | | | |

| SUPPLEMENTAL REPORT EXPECTED (14) | | EXPECTED SUBMISSION DATE (15) | MONTH | DAY | YEAR |
|---|---|---|---|---|---|
| YES (If yes, complete EXPECTED SUBMISSION DATE) | X NO | | | | |

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)


On September 15, 1985 with the plant operating at 100% power, and 20 minutes after an automatic reactor water makeup, a control rod deviation alarm occurred. The measured position signal for bank "B" rods I07 and E07 and bank "D" rods C07 and K07 identified them as the cause for the alarm. An intermittent fluctuation problem with RPI indication, coupled with the reactor coolant system temperature change from the makeup, were assessed as the cause for the deviation. The RPI's were aligned to their respective bank counters.

On September 16, 1985 with the plant operating at 100% power, the control rod position indication system was made inoperable to replace the + 13 VDC and - 13VDC power supplies which were suspected as the cause for the RPI fluctuations. The system was inoperable for a total of 30 minutes; 3 minutes with loss of indication and 27 minutes for RPI alignment.

These conditions were a violation of R.E. Ginna Technical Specification section 3.10.5.




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TEXT (If more space is required, use additional NRC Form 366A's) (17)

At 1955 hours on September 15, 1985 with the plant operating at 100% power, and 20 minutes after an automatic reactor water makeup, a control rod deviation alarm occurred. The measured position signal for bank "B" rods I07 and E07 and bank "D" rods C07 and K07 identified them as the cause for the alarm. An intermittent fluctuation problem with RPI indication that had been present for a couple days, and was under investigation by the I&C Dept., coupled with the reactor coolant system temperature change from the makeup were assessed as the cause for the deviation. The RPI's were aligned to their respective bank counters within approximately 2 1/2 hours of the receipt of the deviation alarm.

On September 16, 1985, investigation continued into the cause of the RPI intermittent fluctuation. During this period, a full core flux map was obtained by the Reactor Engineer. All parameters measured were acceptable with no anomalies noted. At 1730 hours the rod position indicating system was made inoperable to replace the + 13VDC and - 13VDC power supplies (OEM Power Mate Corp. RD-12) which were suspected as the cause for the fluctuations. The system was inoperable for a total of 30 minutes; 3 minutes with loss of indication and 27 minutes for RPI alignment. As specified in the approved Emergency Maintenance procedure (EM-419) for the replacement of the power supplies the Reactor Engineer obtained selected flux traces prior to the replacement of the supplies and again after replacement, with no abnormalities noted. The RPI fluctuation problem was corrected with the replacement of the power supplies. The faulty power supplies were bench tested for three days with no noted problems, and have subsequently been disposed of.

An existing procedure PT-1 for verifying the proper operability of each of the full length control rod assemblies, the rod drive mechanism and the associated control and indication circuits was used for post maintenance testing. During the performance of this test it was identified that the 1BD full length rod control system power cabinet was constantly selecting group A (bank B) mechanism regardless of the logic cabinet demands. This incident and corrective actions are being reported in LER 85-017.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

A cause investigation meeting was held on 9/30/85 for the above incidents and a corrective action plan developed to preclude reoccurrence. Some of the items identified were the need for additional supervision involvement in certain decision making, post RPI alignment testing, and additional operations surveillances. Procedure changes have been submitted to address these items.

ROCHESTER GAS AND ELECTRIC CORPORATION • 89 EAST AVENUE, ROCHESTER, N.Y. 14649-0001

ROGER W. KOBER
VICE PRESIDENT
ELECTRIC & STEAM PRODUCTION

TELEPHONE
AREA CODE 716 546-2700

October 15, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: LER-85-016, Inoperable Rod Position Indicating System

R.E. Ginna Nuclear Power Plant
Docket No. 50-244

In accordance with 10 CFR 50.73, Licensee Event Report System, item (a) (2) (i) (B) "Any operation or condition prohibited by the plant's Technical Specifications" the attached Licensee Event Report LER 85-016 is hereby submitted.

Very Truly Yours,

Roger W. Kober

Roger W. Kober

xc: U.S. Nuclear Regulatory Commission
Region 1
631 Park Avenue
King of Prussia, PA 19406

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