NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
EDWIN I. HATCH, UNIT II	0 5 0 0 0 3 6 6	1 OF 0 2

TITLE (4)

ENGINEERED SAFETY FEATURE ACTUATION (RWCU ISOLATION).

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 7	0 7	8 5	8 5	— 0 2 1	— 0 0	0 8	0 2	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)			
POWER LEVEL (10) 1 0 0	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
	AREA CODE
Steven B. Tipps, Superintendent of Regulatory Compliance	9 1 2 — 3 6 7 7 8 5 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At approximately 1415 CDT on 07/07/85, with the unit operating at 2436 MWt (approximately 100% power), and following performance of the "REACTOR WATER CLEANUP DEMINERALIZER" procedure (HNP-2-1326), plant personnel were placing the reactor water cleanup (RWCU) system in service per the "REACTOR WATER CLEANUP SYSTEM" procedure (HNP-2-1325) they noted that the RWCU pump inboard (2G31-F001) and outboard (2G31-F004) suction isolation valves had isolated on high system differential flow.

After an investigation, plant personnel determined that valves 2G31-F126A and 2G31-F127A had not been completely closed after HNP-2-1326 had been performed because their position indicators were misaligned. Consequently, when the RWCU system was started, RWCU water flowed past the valves, and into radwaste cleanup phase separator "2A", thus causing the RWCU system to isolate on high differential flow.

After an investigation, valves 2G31-F126A and 2G31-F127A were closed.

The RWCU system was then satisfactorily placed in service per HNP-2-1325 at approximately 1645 CDT on 07/07/85.

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FACILITY NAME (1)	DOCKET NUMBER (2)	LER NUMBER (6)			PAGE (3)
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

This 30 day LER is reportable per 10 CFR 50.73(a)(2)(iv) because this event was the unplanned actuation of an engineered safety feature (primary containment isolation valves 2G31-F001 and 2G31-F004).

At approximately 1415 CDT on 07/07/85, with the unit operating at 2436 MWt (approximately 100% power) following performance of the "REACTOR WATER CLEANUP DEMINERALIZER" procedure (HNP-2-1326), plant personnel were placing the reactor water cleanup (RWCU) system in service per the "REACTOR WATER CLEANUP SYSTEM" procedure (HNP-2-1325), when they noted that the RWCU pump inboard (2G31-F001) and outboard (2G31-F004) suction isolation valves had isolated on high system differential flow.

After an investigation, plant personnel determined that the RWCU strainer (2G31-D004A) manually operated drain valves 2G31-F126A and 2G31-F127A had not been completely closed upon completion of procedure HNP-2-1326, because the two valves' local position indicators had become misaligned from repeated use. Consequently, when the RWCU system was placed in service, the RWCU water flowed past the open valves, and into radwaste cleanup phase separator "2A", which resulted in a high differential flow condition, thus the RWCU system isolated on high differential flow (i.e., isolation valves 2G31-F001 and 2G31-F004 closed).

Subsequent to the investigation, operations personnel entered the valve nest and closed valves 2G31-F126A and 2G31-F127A on 07/07/85 at approximately 1630 CDT. Repair is in progress for the position indicators on valves 2G31-F126A and 2G31-F127A.

The RWCU system was then satisfactorily placed in service per HNP-2-1325 on 07/07/85 at approximately 1645 CDT.

This event is the result of the local position indicators on valves 2G31-F126A and 2G31-F127A becoming misaligned.

This event had no actual or potential adverse safety consequences (i.e., the isolation of valves 2G31-F001 and 2G31-F004 had no impact on safe operation of the plant); nor were other systems in Unit 1 or Unit 2 affected.

There have been no past occurrences where valves 2G31-F001 and 2G31-F004 isolated on high RWCU system differential flow because drain valves 2G31-F126A and 2G31-F127A were not completely closed. However, similar events in the past where the RWCU system has isolated on high differential flow for other reasons are described in the following LERs: 50-321/1984-010 on 08/20/84; 50-366/1984-007 on 09/08/84; 50-366/1984-031 on 10/31/84; 50-321/1985-019 on 04/30/85. Since the causes for these similar events differ from the cause for this event, the previous corrective actions did not prevent recurrence of this event.

This event did not affect the health and safety of the public.

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Georgia Power

Edwin I. Hatch Nuclear Plant

August 2, 1985
LR-MRG-073-0785

PLANT E. I. HATCH
Licensee Event Report
Docket No. 50-366

United States Nuclear Regulatory Commission
Document Control Desk
Washington, D. C. 20555

Attached is Licensee Event Report No. 50-366/1985-021. This report is required by 10CFR 50.73(a)(2)(iv).

H. C. Nix
General Manager

HCN/SBT/vlz

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