

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) H. B. Robinson, Unit No. 2	DOCKET NUMBER (2) 0 5 0 0 0 2 6 1 1	PAGE (3) 1 OF 0 2
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TITLE (4)  
Reactor Trip due to a false low Reactor Coolant flow

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
09	17	85	85	021	00	10	10	85			0 5 0 0 0
											0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following): (11)									
POWER LEVEL (10) 1 0 0	<input type="checkbox"/> 20.402(b)	<input type="checkbox"/> 20.406(c)	<input checked="" type="checkbox"/> 80.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)						
	<input type="checkbox"/> 20.405(a)(1)(i)	<input type="checkbox"/> 80.38(c)(1)	<input type="checkbox"/> 80.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)						
	<input type="checkbox"/> 20.405(a)(1)(ii)	<input type="checkbox"/> 80.38(c)(2)	<input type="checkbox"/> 80.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)						
	<input type="checkbox"/> 20.405(a)(1)(iii)	<input type="checkbox"/> 80.73(a)(2)(ii)	<input type="checkbox"/> 80.73(a)(2)(vii)(A)							
	<input type="checkbox"/> 20.405(a)(1)(iv)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(viii)(B)							
<input type="checkbox"/> 20.405(a)(1)(v)	<input type="checkbox"/> 80.73(a)(2)(iii)	<input type="checkbox"/> 80.73(a)(2)(ix)								

LICENSEE CONTACT FOR THIS LER (12)	
NAME Carson L. Wright	TELEPHONE NUMBER AREA CODE: 8 0 3    3 8 1 3 - 1 4 5 2 4

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)										
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	

SUPPLEMENTAL REPORT EXPECTED (14)			EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)	<input checked="" type="checkbox"/> NO					

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On September 17, 1985, the Reactor was at 100% power. At 1259 hours, a Reactor Trip occurred due to a false low Reactor Coolant flow signal.

Testing on the loop 1 Reactor Coolant Flow Protection circuitry was in progress. One of the three bistables for loop 1 was tripped for testing. An Instrument Bus (IB) #2 voltage spike occurred generating a false second loop 1 low flow signal. This provided the necessary 2 out of 3 signals required for the one loop low flow Reactor Trip.

An extensive walkdown of IB #2 was performed, and no specific cause for the voltage spike was identified. A special Plant Nuclear Safety Committee (PNSC) meeting was held to review the event. The PNSC determined that the investigation had been thorough, that no other action could be taken to determine the cause of the voltage spike, and that it was safe to restart the Unit. As of October 8, 1985, voltage spikes on IB#2 have not reoccurred.

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PDR ADOCK 05000261  
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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1)  H. B. Robinson, Unit No. 2	DOCKET NUMBER (2)  0 5 0 0 0 2 6 1	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
		8 5	- 0 2 1	- 0 0	0 2	OF	0 2

TEXT (If more space is required, use additional NRC Form 368A's) (17)

On September 17, 1985, the Reactor was at 100% power. At 1259 hours, a Reactor Trip occurred due to a false low Reactor Coolant flow signal.

Maintenance Surveillance Test, (MST)-006 (Reactor Coolant Flow Protection Channel Testing), was in progress. Reactor Coolant Loop 1 circuitry was being tested. The low flow bistable for FC-414 (1 of the 3 flow comparators for Loop 1) was tripped for testing. An Instrument Bus (IB) #2 voltage spike occurred generating a false second loop 1 low flow signal. This provided the necessary coincident 2 out of 3 signals required for the one loop low flow Reactor Trip.

An IB #2 voltage spike was indicated by the momentary N-32 source range trip signal, as well as the number of annunciators that lit momentarily.

A second spike occurred at 1345 hours generating a Source Range, N-32, and Intermediate Range, N-36, trip signal (both are powered from IB #2). The trip breakers were open from the trip occurring at 1259 hours and, therefore, this spike did not result in another Reactor Trip.

An extensive walkdown of IB #2 and #7 was performed, and no specific cause for the voltage spike was identified (IB#7 is powered from IB#2). A special Plant Nuclear Safety Committee (PNSC) meeting was held to review the event.

The PNSC determined that the investigation had been thorough, that no other action could be taken to determine the cause of the voltage spike, and that it was safe to restart the Unit. As of October 8, 1985, voltage spikes on IB#2 have not reoccurred.



Carolina Power & Light Company

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Serial: RNP/85-3418

United States Nuclear Regulatory Commission  
Document Control Desk  
Washington, D.C. 20555

H. B. ROBINSON STEAM ELECTRIC PLANT, UNIT NO. 2  
DOCKET NO. 50-261  
LICENSE NO. DPR-23  
LICENSEE EVENT REPORT 85-021

Dear Sir:

In accordance with 10CFR50.73, Licensee Event Report, the enclosed Licensee Event Report is submitted. This report fulfills the requirements for a written report within (30) days of a reportable event and is in accordance with the format set forth in NUREG-1022, September, 1983.

Very truly yours,

R. E. Morgan  
General Manager  
H. B. Robinson S. E. Plant

CLW:sdm

Enclosure

cc: INPO  
J. N. Grace  
H. E. P. Krug

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