



Nebraska Public Power District

COOPER NUCLEAR STATION
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NLS970011

January 10, 1997

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555-0001

Dear Sir:

Cooper Nuclear Station Licensee Event Report 96-009 Supplement 2 is forwarded as an attachment to this letter.

Sincerely,

M. F. Peckham
Plant Manager

/wrv

Attachment

cc: Regional Administrator
USNRC - Region IV

Senior Project Manager
USNRC - NRR Project Directorate IV-1

Senior Resident Inspector
USNRC- Cooper Nuclear Station

NPG Distribution

INPO Records Center

W. Turnbull
MidAmerica Energy

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Correspondence No: NLS970011

The following table identifies those actions committed to by the District in this document. Any other actions discussed in the submittal represent intended or planned actions by the District. They are described to the NRC for the NRC's information and are not regulatory commitments. Please notify the Licensing Manager at Cooper Nuclear Station of any questions regarding this document or any associated regulatory commitments.

COMMITMENT	COMMITTED DATE OR OUTAGE
Additional [LER] supplements will be provided, as necessary, based on the ongoing Appendix R reevaluation [a final non-abstract supplement will be provided following the completion of the CNS Appendix R Safe and Alternate Shutdown reanalysis].	None

LICENSEE EVENT REPORT (LER)

(See reverse for required number of
digits/characters for each block)ESTIMATED BURDEN PER RESPONSE TO COMPLY WITH THIS MANDATORY INFORMATION
COLLECTION REQUEST: 50.0 HRS. REPORTED LESSONS LEARNED ARE INCORPORATED INTO
THE LICENSING PROCESS AND FED BACK TO INDUSTRY. FORWARD COMMENTS REGARDING
BURDEN ESTIMATE TO THE INFORMATION AND RECORDS MANAGEMENT BRANCH (T-6 F33),
U.S. NUCLEAR REGULATORY COMMISSION, WASHINGTON, DC 20555-0001, AND TO THE
PAPERWORK REDUCTION PROJECT (3150-0104), OFFICE OF MANAGEMENT AND BUDGET,
WASHINGTON, DC 20503.

FACILITY NAME (1)

COOPER NUCLEAR STATION

DOCKET NUMBER (2)

05000298

PAGE (3)

1 OF 1

TITLE (4)

Appendix R Safe Shutdown Analysis Vulnerabilities

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAME	DOCKET NUMBER
07	25	96	96	009	02	1	10	97	FACILITY NAME	DOCKET NUMBER
OPERATING MODE (9)		N	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more) (11)							
POWER LEVEL (10)		100	20.2201(b)		20.2203(a)(2)(v)		50.73(a)(2)(i)		50.73(a)(2)(viii)	
			20.2203(a)(1)		20.2203(a)(3)(i)		50.73(a)(2)(ii)		50.73(a)(2)(x)	
			20.2203(a)(2)(i)		20.2203(a)(3)(ii)		50.73(a)(2)(iii)		73.71	
			20.2203(a)(2)(ii)		20.2203(a)(4)		50.73(a)(2)(iv)		<input checked="" type="checkbox"/> OTHER	
			20.2203(a)(2)(iii)		50.36(c)(1)		50.73(a)(2)(v)		Specify in Abstract below or in NRC Form 366A	
			20.2203(a)(2)(iv)		50.36(c)(2)		<input checked="" type="checkbox"/> 50.73(a)(2)(vii)			

LICENSEE CONTACT FOR THIS LER (12)

NAME

William R. Victor, Licensing and Compliance Specialist

TELEPHONE NUMBER (Include Area Code)

(402) 825-3811

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS

SUPPLEMENTAL REPORT EXPECTED (14)

<input checked="" type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE).	<input type="checkbox"/> NO	EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
			02	24	97

ABSTRACT (Limit to 1400 spaces, i.e., approximately 15 single-spaced typewritten lines) (16)

As part of a corrective action for LER 94-016 Supplement 1, a reevaluation of the Cooper Nuclear Station (CNS) Appendix R Safe and Alternate Shutdown Analysis Report has been in progress. It was determined that the following fire induced vulnerabilities did not have acceptable coping strategies:

- Given a fire on 931' Reactor Building North and assuming multiple ground cable faults, the Appendix R credited Reactor Core Isolation Cooling (RCIC) System controller could become disabled. ENS notification made on 7/25/96. Circuit modifications have been completed which have corrected this condition.
- Assuming multiple ground cable faults during an alternate shutdown fire could result in the failure of the Diesel Generator No. 2 (DG-2) control power fuse prior to isolation of the DG-2 control circuits from the fire area. ENS notification made on 7/25/96. Circuit modifications have been completed which have corrected this condition.
- Several additional vulnerabilities stemming from the observation that four types of assumptions may not have been fully addressed in the original Appendix R Safe and Alternate Shutdown Analysis Report. Supplemental ENS notification made on 8/13/96. The identified vulnerabilities have been corrected.
- RHR-MOV-MO39B was potentially unable to open from its Alternate Shutdown Panel due to an interlock with RHR-MOV-MO15B during an alternate shutdown fire. This would challenge the ability to initiate Suppression Pool Cooling within the credited 3-hour time period, and required entry into Technical Specification (TS) 3.2.1.2. CNS plans to modify the RHR-MOV-MO39B control circuitry to eliminate this vulnerability. Supplemental ENS notification made on 11/12/96. This LER fulfills the reporting requirements of TS 3.2.1.2.
- During a Cable Spreading Room or Control Room fire, fire-induced faults could prevent the Appendix R credited diesel-driven fire pump from starting (or could stop it if it were already running). Circuit modifications have been completed which have corrected this condition. Supplemental ENS notification made on 12/11/96.

The CNS Appendix R reevaluation effort is continuing. This LER is being submitted in abstract form pending the completion of this reanalysis. Additional supplements will be provided, as necessary, based on the ongoing Appendix R reevaluation.