

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Waterford 3 Steam Electric Station										DOCKET NUMBER (2) 0 5 0 0 0 3 8 2				PAGE (3) 1 OF 03		
TITLE (4) Automatic Actuation of Reactor Protective System Due to Cut of Range ASI																
EVENT DATE (5)			LER NUMBER (6)				REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)						
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES			DOCKET NUMBER(S)				
0	7	0	4	8	5	0	3	2	N/A			0 5 0 0 0				
0	7	0	4	8	5	0	3	2	N/A			0 5 0 0 0				
OPERATING MODE (9)		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)														
1		20.402(b)				20.406(c)				<input checked="" type="checkbox"/> 50.73(a)(2)(iv)		73.71(b)				
POWER LEVEL (10)		20.406(a)(1)(i)				50.36(c)(1)				50.73(a)(2)(v)		73.71(c)				
01016		20.406(a)(1)(ii)				50.36(c)(2)				50.73(a)(2)(vii)		OTHER (Specify in Abstract below and in Text, NRC Form 366A)				
		20.406(a)(1)(iii)				50.73(a)(2)(i)				50.73(a)(2)(viii)(A)						
		20.406(a)(1)(iv)				50.73(a)(2)(ii)				50.73(a)(2)(viii)(B)						
		20.406(a)(1)(v)				50.73(a)(2)(iii)				50.73(a)(2)(x)						
LICENSEE CONTACT FOR THIS LER (12)																
NAME										TELEPHONE NUMBER						
L. Myers, Operations Superintendent										AREA CODE		5 0 4 4 6 4 - 3 1 1 8				
COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)																
CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS						
SUPPLEMENTAL REPORT EXPECTED (14)												EXPECTED SUBMISSION DATE (15)		MONTH	DAY	YEAR
<input type="checkbox"/> YES (If yes, complete EXPECTED SUBMISSION DATE)												<input checked="" type="checkbox"/> NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

ABSTRACT

At 2130 hours on July 4, 1985, Waterford 3 Steam Electric Station was at 6% reactor power when an uncomplicated reactor trip occurred due to the Axial Shape Index being out of the range allowed by the Core Protection Calculator. Plant conditions were subsequently stabilized in mode 3 (hot standby).

Plant Personnel have taken short term, and will also take long term, corrective action to prevent this condition from recurring.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

APPROVED OMB NO. 3150-0104

EXPIRES: 8/31/86

FACILITY NAME (1) Waterford 3 Steam Electric Station	DOCKET NUMBER (2) 0 5 0 0 0 3 8 2 8 5 - 0 3 2 - 0 0 0 2 OF 0 3	LER NUMBER (6)			PAGE (3)		
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			

TEXT (If more space is required, use additional NRC Form 366A's) (17)

NARRATIVE

At 2130 hours on July 4, 1985, Waterford 3 Steam Electric Station was at 6% reactor power when an uncomplicated reactor trip occurred due to the Axial Shape Index being out of the range allowed by the Core Protection Calculator. Operations Personnel immediately entered procedures OP-902-000, Emergency Entry Procedure, and OP-902-001, Uncomplicated Reactor Trip Recovery. Plant conditions were subsequently stabilized in mode 3 (hot standby).

The Axial Shape Index was out of range due to the Xenon distribution which occurred as a result of a reactor trip earlier in the day (see Licensee Event Report 85-031). Since the value of the Axial Shape Index is not calculated by the Core Protection Calculator below 6% reactor power, Operations Personnel could not determine that the Axial Shape Index was out of range until they started to increase reactor power above 6%.

SAFETY CONSEQUENCES AND IMPLICATIONS

The above event resulted in an automatic actuation of the Reactor Protective System during initial Startup testing in which no primary system parameters were exceeded. Since the Control Element Assemblies and the Reactor Protective Systems functioned as designed, the event in no way placed Waterford 3 in a degraded safety condition.

CORRECTIVE ACTION

In an effort to prevent this condition from recurring, Plant Operators have been given temporary guidance on returning to power when the potential for adverse Xenon distribution exists. Also, Louisiana Power & Light has enlisted the support of Middle South Utilities who developed a computer program which will allow Plant

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		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER			
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TEXT (If more space is required, use additional NRC Form 366A's) (17)

Operators to calculate the Axial Shape Index below 6% reactor power. These are temporary solutions, however, since the Core Protection Calculator/Core Operating Limit Supervisory System will be upgraded during the second fuel cycle such that the actual value of Axial Shape Index will be available to Plant Operators.

SIMILAR EVENTS

NONE

PLANT CONTACT

L. Myers, Operations Superintendent, 504/464-3118



LOUISIANA
POWER & LIGHT

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NEW ORLEANS, LOUISIANA 70174-8008 • (504) 386-2345

August 2, 1985

W3P85-1448
A4.05

Director, Office of Nuclear Reactor Regulation
ATTENTION: Document Control Desk
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Subject: Waterford 3 SES
Docket No. 50-382
License No. NPF-38
Reporting of Licensee Event Report

Dear Sirs:

Attached is Licensee Event Report Number LER-85-032-00 for the Waterford 3 Steam Electric Station. This Licensee Event Report is submitted per 10CFR50.73(a)(2)(iv).

Very truly yours,

K.W. Cook
Nuclear Support & Licensing Manager

KWC:GEW:sms

Attachment

cc: R.D. Martin, G.W. Knighton, D.M. Crutchfield, NRC Resident Inspectors
Office, INPO Records Center (J.T. Wheelock), B.W. Churchill,
W.M. Stevenson

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