QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

SEPTEMBER 1985

COMMONWEALTH EDISON COMPANY

AND

IOWA-ILLINOIS GAS & ELECTRIC COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

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I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company ano Iowa-Illinois Gas & Electric Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for Unit Two.

This report was compiled by Becky Brown and Carol Kronich, telephone number 309-654-2241, extensions 127 and 193.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

September 1-8

Unit One began the month of September on Economic Generation Control (EGC). On September 1, at 0122 hours, EGC was tripped and load was decreased at 100 MWe/hour to 700 MWe for MSIV and Turbine surveillances. Seven hundred MWe was reached at 0210 hours and held until 0715 hours when load was increased at 100 MWe/hours to 775 MWe, and then increased 5 MWe/hour. At 1040 hours 788 MWe was reached and the unit was placed on EGC. On September 3, at 0530 hours, EGC was tripped, load was increased to 815 MWe at 5 MWe/hour, and held steady. On September 4, at 1845 hours, load was decreased to 785 MWe and the unit was placed on EGC.

On September 5, at 0610 hours, EGC was tripped and load was increased at 5 MWe/hour. At 1135 hours, 817 MWe was reached and held steady, until September 5, at 2352 hours, wher the unit was placed on EGC. On September 7, at 1120 hours, EGC was tripped, as per the Load Dispatcher and load was increased at 5 MWe/hour until 1500 hours, when 800 MWe was reached and held steady. On September 8, at 0120 hours, the unit was placed on EGC. EGC was tripped at 0705 hours, and load was increased at 5 MWe/hour. At 0800 hours load was holding at 804 MWe until 1640 hours, when load was dropped to 790 MWe in preparation for EGC. At 1650 hours, 790 MWe was reached and at 1700 hours the unit was placed on EGC.

September 9-11

EGC was tripped on September 9, at 1218 hours. Load was increased to 790 MWe and then 5 MWe/hour. At 1420 hours, 805 MWe was reached and held until 1830 hours, when load was dropped to 790 MWe in preparation for EGC. At 1840 hours, 788 MWe was reached, and the unit was placed on EGC at 1855 hours. EGC was tripped on September 10, at 0815 hours. At 0820 hours load began an increase at 5 MWe/hour until 808 MWe was reached at 1145 hours.

On September 11, at 0145 hours, load was decreased at 140 MWe/hour to 600 MWe as per Load Dispatcher. Six hundred MWe was reached at 0315 hours. At 0500 hours load began an increase to 720 MWe at 100 MWe/hour. At 0745 hours, load was increased 20 MWe to 748 MWe, and then began an increase at a rate of 5 MWe/hour. At 1100 hours, load was increased 15 MWe to 775 MWe, and then continued to increase at 5 MWe/hour to maximum power. Two thousand five hundred and eleven MWt was reached at 1930 hours.

A. Unit One (continued)

September 12-15

On September 12, at 0025 hours, load was dropped to 790 MWe for EGC. At 0040 hours, 790 MWe was reached and the unit was placed on EGC. At 0315 hours, EGC was tripped at 750 MWe. Load began a drop to 600 MWe at 150 MWe/hour as per the Load Dispatcher. At 0420 hours, 600 MWe was reached, and at 0455 hours load began an increase at 100 MWe/hour to 750 MWe. At 0625 hours, 750 MWe was reached, and load began an increase at 5 MWe/hour. Eight hundred and twelve MWe was reached at 1720 hours. At 2200 hours, load began a decrease to 200 MWe for required scram timing at 150 MWe/hour from 810 MWe. At 0700 hours, load was holding at 200 MWe.

On September 13, at 1420 hours, the control rod pattern change began. At 1830 hours, load reached 500 MWe and was held until 2000 hours when load began an increase to 550 MWe. Five hundred and fifty MWe was reached at 2040 hours, and at 2200 hours load was increasing at a rate of 5 MWe/hour. On September 14, at 0940 hours, load was increased to 600 MWe, and then continued at 5 MWe/hour on September 15, at 0915 hours, 795 MWe was reached and held steady.

September 16-21

On September 16, at 1135 hours, load was increasing at a rate of 5 MWe/hour until 1500 hours, when 819 MWe was reached and continued to hold steady. On September 17, at 1235 hours, load was dropped to 795 MWe in preparation for EGC. The unit was placed on EGC at 1254 hours.

EGC was tripped on September 18, at 1040 hours, at 823 MWe. Load was held steady until 1310 hours, when a load drop to 700 MWe began for a Feedwater Pump swap. Seven hundred MWe was reached at 1325 hours, and load began an increase at 1345 hours to 810 MWe. At 1425 hours, 810 MWe was reached, and load continued to increase at 5 MWe/hour to maximum power. At 1630 hours, 823 MWe was reached and held steady until September 19, at 0130 hours, when load began a decrease to 700 MWe for a Feedwater Pump swap. At 0230 hours, load was increased to 750 MWe, and at 0235 hours the unit was placed on EGC. At 1340 hours, EGC was tripped and load was increased to 800 MWe, and then continued at 5 MWe/hour. At 1705 hours load was dropped to 795 MWe in preparation for EGC and the unit was placed on EGC at 1717 hours.

A. Unit One (continued)

September 22-30

On September 22, at 0050 hours, EGC was tripped and load began a decrease to 750 MWe for surveillances, at 150 MWe/hour. Seven hundred and fifty MWe was reached at 0130 hours, and at 0240 hours the unit returned to EGC. On September 23, at 0545 hours, EGC was tripped and load was increased to 800 MWe as per Load Dispatcher. At 1210 hours, the unit was returned to EGC, until September 26, at 1005 hours, when EGC was tripped, and load was increased to 800 MWe as per Load Dispatcher. At 1020 hours, load reached 800 MWe and held steady until 1158 hours, when the unit was placed on EGC.

On September 28, at 0045 hours, EGC was tripped and load began a drop to 700 MWe at 100 MWe/hour for a control rod pattern adjustment. At 0135 hours, load was increased to 750 MWe. At 0156 hours, load reached 750 MWe, and continued to increase at 5 MWe/hour until 2000 hours when 825 MWe was reached.

On September 29, at 1040 hours, load was dropped to 800 MWe, and at 1105 hours the unit was placed on EGC. On September 30 at 0920 hours EGC was tripped and load was increased to full power as per the Load Dispatcher. At 1310 hours load was dropped to 800 MWe in preparation for EGC. The unit was placed on EGC at 1325 hours and remained through the end of September.

B. Unit Two

September 1-4

Unit Two began the month of September on Economic Generation Control (EGC). On September 1, at 0025 hours, EGC was tripped and load began a drop to 400 MWe for surveillances, at 200 MWe/hour. At 0130 hours the load drop was terminated at 575 MWe as per the Operating Engineer and Shift Engineer due to a Feedwater Regulation Valve problem. At 0700 hours, load began a normal increase to full power. Eight hundred and nineteen MWe was reached at 1930 hours. At 2000 hours load was decreased to 800 MWe in preparation of EGC. The unit was placed on EGC at 2150 hours.

On September 2, at 0136 hours, EGC was tripped and load was holding at 780 MWe until 0500 hours when the unit returned to EGC. EGC was tripped on September 3, at 0510 hours, as per Load Dispatcher. At 0600 hours, load began increasing 5 MWe/hour, until 0805 hours when load reached 804 MWe. Load was held steady until September 4, at 1845 hours when load was decreased to 800 MWe and the unit was placed on EGC.

B. Unit Two (continued)

September 5-15

On September 5, at 1602 hours, EGC was tripped and load was increased to 808 MWe. The unit returned to EGC on September 6 at 0305 hours. EGC was tripped at 0355 hours and load was held at approximately 810 MWe until September 7, at 0110 hours, when the unit was placed on EGC. At 1120 hours, EGC was tripped as per Load Dispatcher, and load was increased 5 MWe/hour until 1325 hours, when maximum power 2511 MWt was obtained. At 2320 hours load began a drop at 50 MWe/hour to 600 MWe for MSIV testing. At 0110 hours, on September 8, the load drop was increased to 150 MWe/hour. Six hundred MWe was reached at 0200 hours. Due to the failure of the MSIV's, a normal shutdown began at 0820. At 1140 hours, load was held steady at 280 MWe to repair the MSIV's. At 1315 hours a normal increase to full power began.

Eight hundred and thirteen MWe was reached on September 9, at 1805 hours. On September 12, at 1415 hours, the unit was placed on EGC, until September 15 at 0025 hours, when EGC was tripped. Load began a drop at 150 MWe/hour to 550 MWe as per Load Dispatcher. Five hundred and fifty MWe was reached at 0130 hours, and at 0150 hours, load was decreased further at 100 MWe/hour to 500 MWe. The unit was holding at 500 MWe at 0300 hours. At 0545 hours, load began increasing at 100 MWe/hour to 750 MWe until 0745 hours when the unit began holding at 690 MWe as per Load Dispatcher. At 1600 hours load was dropped to 600 MWe as per Load Dispatcher, and began to hold steady at 1700 hours. At 1830 hours, the unit began a normal increase to full power.

September 16-22

Eight hundred and twenty-five MWe was reached on September 16, at 1545 hours. At 2130 hours, load was decreased to 800 MWe in preparation for EGC. The unit was placed on EGC at 2153 hours. At 2320 hours, the unit was taken off EGC, and began a load drop at 100 MWe/hour to 600 MWe as per the Load Dispatcher. On September 17, at 0115 hours, 600 MWe was reached. At 0500 hours load was increased at 100 MWe/hour to 750 MWe. At 0850 hours, the unit was placed on EGC until September 18, at 1035 hours, when EGC was tripped and load was increased to full power.

On September 19, at 0144 hours, the unit returned to EGC. At 1340 hours, EGC was tripped and the unit was brought up to full power. At 1700 hours, load was dropped to 800 MWe in preparation for EGC, and the unit was placed on EGC at 1720 hours.

B. Unit Two (continued)

September 16-22 (continued)

The unit was taken off EGC at 0800 hours, at 735 MWe for a Condensate pump test. The unit was returned to EGC at 0845 hours. At 1655 hours, EGC was tripped for a condenser flow reversal, and returned to EGC at 1735 hours.

September 23-30

On September 23, at 0545 hours, EGC was tripped at 800 MWe. Load was held steady until 1212 hours when the unit returned to EGC, until September 26, at 1005 hours, when EGC was tripped and load was increased to full power. The unit returned to EGC at 1157 hours.

On September 30, at 0810 hours, the unit was taken off EGC, and load was increased to maximum power. Eight hundred and twenty-eight MWe was reached at 0835 hours, and was held steady for the remainder of September.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

D. Corrective Maintenance of Safety Related Equipment

The following represents a tabular summary of the major safety related maintenance performed on Units One and Two during the reporting period. This summary includes the following: Work Request Numbers, Licensee Event Report Numbers, Components, Cause of Malfunctions, Results and Effects on Safe Operation, and Action Taken to Prevent Repetition.

UNIT 1 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q41378	85-002 85-014	1-1701A Fuel Pool Radiation Monitor	Loose capacitor and an open light emitting diode in sensor unit.	Monitor spiked high, immediately reset, Standby Gas Treatment auto-started; Reactor ventilation isolated.	Loose capacitor and open LED were replaced.
Q42745		1-1601-63 Torus Valve	Air valve operator out of adjustment	Valve would not open. Emergency (normal) position is closed.	Air valve operator adjusted to open within ten seconds.
Q43889		1-2301-48 HPCI Cooling Water Valve	No problem found in valve operator.	Valve would not open or close from Control Room. Manually left open so HPCI was available, if needed.	Full maintenance investigation revealed no problem. Valve tested satisfactorily. AIR 4-84-30 written to investigate.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q42992		250 Volt Battery Charger	Water falling on battery charger from above room (mopping in progress).	Battery charger tripped.	Circuit breaker and transformer replaced like-for-like. Holes in ceiling sealed.
Q43285	85-15	2-1735A Reactor Bldg. Radiation Monitor	Failed Geiger-Mueller tube in sensor and convertor unit.	Monitor drifted and tripped upscale. Initiated Standby Gas Treatment and Reactor Bldg. Vent System tripped.	Tube replaced like-for-like and monitor tested satisfactorily.
Q43286	85-15	2-5742A Reactor Bldg. Vent Isolation Valve	Unknown.	Damper failed to close.	None. Tested satisfactorily.
Q43288	85-15	2A-5741 Reactor Bldg.	Failed Versa Valve operated solenoid.	Damper failed to close.	Solenoid replaced like-for-like, and tested satisfactorily.
Q43518		590-107C IRM 13 / APRM 3 Relay	Relay coil over-heated causing the relay to burn up.	Relay failed, causing Channel 'A' half-scram.	Relays are being upgraded to model with longer lifetime.
Q43683		2-1601-20A Valve Indicator	Dirty water found in accumulator. Dirt caused the breaker valve to "lock-up".	Vacuum breaker valve did not open. In-line valve closed to assure Primary Containment.	Accumulator tank was drained. Vacuum breaker valve was successfully tested.

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UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q38978		2-503C Condenser Low Vacuum Scram Pressure Switch	Instrument set-point drift.	Tripping point below normal set-point.	Because of bad history, this pressure switch was replaced like-for-like.
Q42098		2-6601 Diesel Generator	Wiring circuitry in starting logic.	No affects. Diesel still would have started manually or automatically.	Wiring was changed to correct design.
Q42166	85-007	2-8804 Oxygen Analyzer Valve	Small scratches on the valve seat.	Failed Local Leak Rate Test.	Lapped valve seat. Retested satisfactorily.
Q42167	85-007	AO 2-8801C Oxygen Analyzer Valve	Very dirty (rusty) valve internals.	Failed Local Leak Rate Test.	Cleaned valve internals and lapped valve seat to reduce leakage.
Q42536	85-007	2-1601-20B Vacuum Breaker Valve	Worn packing on shaft.	Failed Local Leak Rate Test.	Replaced packing.
Q42664		2-300-J5 CRD 34-19 HCU 2-305-112 Valve	Broken valve stem. Blocked flow of above piston water to Scram Discharge Volume.	While scram timing, control rod J-5 exceeded maximum inserting time.	Like-for-like replacement of valve stem.
Q42672		2-3999-515C RHR Service Water Vault Sump Check Valve	Valve disc was stuck in the OPEN position.	Excessive leak when tested.	Cleaned valve seats, honed side wall.

UNIT 2 MAINTENANCE SUMMARY

W.R. NUMBER	LER NUMBER	COMPONENT	CAUSE OF MALFUNCTION	RESULTS & EFFECTS ON SAFE OPERATION	ACTION TAKEN TO PREVENT REPETITION
Q43715		2-302-109D Scram Discharge Barton Level Transmitter	Incorrect assembly by manufacturer.	Transmitter gave inconsistent low readings.	Transmitter O-ring replaced, tested, and re-installed.
Q44164		IV-LS-82G Scram Discharge Volume North Dump Tank	Level switch circuit board's transformer failed.	Gave a "Channel A/B Discharge Volume Hi-Level" alarm. Channel 'A' half-scram.	Circuit board replaced. Tested satisfactorily.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements as set forth in sections 6.6.B.1 and 6.6.B.2 of the Technical Specifications.

UNIT ONE

Licensee Event Report Number	Date	Title of Occurrence
85-15*	8-30-85	RCIC Turbine Trip
85-16	9-05-85	1B Refuel Floor Monitor Spike

*This LER was incorrectly identified as 85-13 in the August Monthly Performance Report.

UNIT TWO

Licensee Event Report Number	Date	Title of Occurrence
85-19	9-08-85	MSIV Closure Times; Orderly Shutdown
85-20	9-17-85	Condensate Pit Circulating Water Pump Trip

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

OPERATING DATA REPORT

DOCKET NO. 50-254

UNIT ONE

DATE OCTOBER 2 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE 309/654-2241X193

OPERATING STATUS

0000 090185

1. Reporting period: 2400 093085 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	720.0	6302.0	94624.4
6. Reactor reserve shutdown hours	0.0	0.0	3421.9
7. Hours generator on line	720.0	6244.4	91279.7
8. Unit reserve shutdown hours.	0.0	0.0	909.2
9. Gross thermal energy generated(MWH)	1682638	14671459	190417849
10. Gross electrical energy generated(MWH)	551130	4827916	61605367
11. Net electrical energy generated(MWH)	527941	4622562	57578264
12. Reactor service factor	100.0	96.2	80.6
13. Reactor availability factor	100.0	96.2	83.5
14. Unit service factor	100.0	95.3	72.8
15. Unit availability factor	100.0	95.3	78.5
16. Unit capacity factor (Using MDC)	95.4	91.8	63.8
17. Unit capacity factor (Using Des.MWe)	92.9	89.4	62.2
18. Unit forced outage rate	0.0	4.6	6.0

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

#UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT TWO

DATE OCTOBER 2 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE 309/654-2241X193

OPERATING STATUS

1. Reporting period: 0000 090185 2400 093085 Gross hours in reporting period: 720

2. Currently authorized power level (MWt): 2511 Max.Depend capacity (MWe-Net): 769* Design electrical rating (MWe-Net): 789

3. Power level to which restricted(if any)(MWe-Net): NA

4. Reasons for restriction (if any):

	This Month	Yr.to Date	Cumulative
5. Number of hours reactor was critical	720.0	4439.8	89345.8
6. Reactor reserve shutdown hours	0.0	0.0	2985.8
7. Hours generator on line	720.0	4367.9	86417.1
8. Unit reserve shutdown hours.	0.0	0.0	702.9
9. Gross thermal energy generated(MWH)	1685565	10130868	181649935
10. Gross electrical energy generated(MWH)	550405	3293884	57947295
11. Net electrical energy generated(MWH)	527434	3149633	54467618
12. Reactor service factor	100.0	67.8	76.7
13. Reactor availability factor	100.0	67.8	79.3
14. Unit service factor	100.0	66.7	74.2
15. Unit availability factor	100.0	66.7	74.8
16. Unit capacity factor (Using MDC)	95.3	62.5	60.8
17. Unit capacity factor (Using Des.MWe)	92.8	60.9	59.3
18. Unit forced outage rate	0.0	4.2	8.2

19. Shutdowns scheduled over next 6 months (Type,Date,and Duration of each):

20. If shutdown at end of report period,estimated date of startup NA

*The MDC may be lower than 769 MWe during periods of high ambient temperature due to the thermal performance of the spray canal.

*UNOFFICIAL COMPANY NUMBERS ARE USED IN THIS REPORT

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-254
UNIT ONE
DATE OCTOBER 2 1985
COMPLETED BY CAROL L. KRONICH
TELEPHONE 309/654-2241X193

MONTH September 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	730.3	17.	767.8
2.	748.1	18.	780.8
3.	770.5	19.	729.0
4.	764.5	20.	742.6
5.	773.7	21.	743.2
6.	761.2	22.	750.3
7.	758.8	23.	747.0
8.	753.1	24.	746.3
9.	732.5	25.	755.1
10.	748.3	26.	756.6
11.	742.8	27.	727.9
12.	712.3	28.	760.5
13.	311.8	29.	769.4
14.	614.0	30.	748.4
15.	737.2		
16.	780.7		

INSTRUCTIONS

On this form,list the average daily unit power level in MWe-Net for each day in the reporting month.Compute to the nearest whole megawatt.
These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit,there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line).In such cases,the average daily unit power output sheet should be footnoted to explain the apparent anomaly

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO. 50-265

UNIT TWO

DATE OCTOBER 2 1985

COMPLETED BY CAROL L. KRONICH

TELEPHONE 309/654-2241X193

MONTH September 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	683.0	17.	703.0
2.	748.8	18.	720.6
3.	754.7	19.	790.7
4.	750.5	20.	737.5
5.	761.3	21.	738.8
6.	752.6	22.	740.1
7.	766.5	23.	752.1
8.	514.0	24.	737.8
9.	697.1	25.	751.6
10.	756.2	26.	753.8
11.	788.3	27.	738.3
12.	745.6	28.	751.8
13.	748.6	29.	736.3
14.	721.7	30.	774.5
15.	614.6		
16.	737.4		

INSTRUCTIONS

On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-254

UNIT NAME Quad-Cities Unit 1

COMPLETED BY C. Kronich

DATE October 7, 1985

REPORT MONTH SEPTEMBER 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-49	850901	S	0.0	B	5		CD	VALVEX	Reduced load to 700 MWe for MSIV surveillance
85-50	850911	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load to 600 MWe per Load Dispatcher
85-51	850912	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load to 600 MWe per Load Dispatcher
85-52	850913	S	0.0	B	5		RB	CONROD	Reduced load to 200 MWe for required scram timing
85-53	850918	S	0.0	B	5		CH	PUMPXX	Reduced load to 700 MWe for a Feedwater Pump swap
85-54	850919	S	0.0	B	5		CH	PUMPXX	Reduced load to 700 MWe for a Feedwater Pump swap
85-55	850928	S	0.0	B	5		RC	CONROD	Reduced load to 700 MWe for a Control Rod Pattern adjustment

APPROVED
AUG 16 1982
QCOSR

ID/5A

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

QTP 300-S13
Revision 6
August 1982

DOCKET NO. 050-265

UNIT NAME Quad-Cities Unit 2

COMPLETED BY C. Kronich

DATE October 7, 1985

REPORT MONTH SEPTEMBER 1985

TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT NO.	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
85-39	850901	S	0.0	B	5		CD	VALVEX	Reduced load to 400 MWe for MSIV surveillance (load drop was terminated at 575 MWe due to 2A Feedwater Regulation Valve problem)
85-40	850908	S	0.0	B	5		CD	VALVEX	Reduced load to 600 MWe for MSIV surveillance
85-41	850908	F	0.0	B	5		CD	VALVEX	Reduced load to 280 MWe to repair MSIV's
85-42	850915	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load to 500 MWe per Load Dispatcher
85-43	850916	S	0.0	H	5		ZZ	ZZZZZZ	Reduced load to 600 MWe per Load Dispatcher

APPROVED
AUG 16 1982
QCOSR

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the commission:

A. Main Steam Relief Valve Operations

Relief valve operations during the reporting period are summarized in the following table. The table includes information as to which relief valve was actuated, how it was actuated, and the circumstances resulting in its actuation.

Unit: One

Date: September 13, 1985

Valves Actuated	No. & Type of Actuation
1-203-3A	1 Manual
1-203-3B	1 Manual
1-203-3C	1 Manual
1-203-3D	1 Manual
1-203-3E	1 Manual

Plant Conditions: Reactor Pressure - 940 lbs

Description of Events: Surveillance Technical Specification 4.5.D.1.a

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram Timing for the reporting period. All scram timing was performed with Reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS

PERFORMED ON UNIT 1 & 2 CONTROL

ROD DRIVES, FROM 1-1 TO 12-31-85

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
9-13-85	88	0.29	0.66	1.43	2.53	2.91 (C-4)	Unit 1 Sequence A Hot Scram Timing

0027H/0061Z

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities, and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 1
March 1978

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q1 Reload: 7 Cycle: 8

2. Scheduled date for next refueling shutdown: 1-2-86

3. Scheduled date for restart following refueling: 4-2-86

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   NOT AS YET DETERMINED.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   DECEMBER 19, 1985; IF LICENSING ACTION REQUIRED.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   NONE PLANNED AT PRESENT TIME.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2340

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

APPROVED

APR 20 1978

Q.C.O.S.R.

QTP 300-S32
Revision 1
March 1973

QUAD-CITIES REFUELING INFORMATION REQUEST

*

1. Unit: Q2 Reload: 7 Cycle: 8

2. Scheduled date for next refueling shutdown: 10-6-86

3. Scheduled date for restart following refueling: 12-22-86

4. Will refueling or resumption of operation thereafter require a technical specification change or other license amendment:

   Yes. A routine MAPLHGR amendment has been approved as a preparatory change to allow a 10 CFR 50.59 review.

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   Not Applicable.

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   None planned at present time.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 176

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity: 2003

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APR 20 1978

Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM	-	Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI	-	American National Standards Institute
APRM	-	Average Power Range Monitor
ATWS	-	Anticipated Transient Without Scram
BWR	-	Boiling Water Reactor
CRD	-	Control Rod Drive
EHC	-	Electro-Hydraulic Control System
EOF	-	Emergency Operations Facility
GSEP	-	Generating Stations Emergency Plan
HEPA	-	High-Efficiency Particulate Filter
HPCI	-	High Pressure Coolant Injection System
HRSS	-	High Radiation Sampling System
IPCLRT	-	Integrated Primary Containment Leak Rate Test
IRM	-	Intermediate Range Monitor
ISI	-	Inservice Inspection
LER	-	Licensee Event Report
LLRT	-	Local Leak Rate Test
LPCI	-	Low Pressure Coolant Injection Mode of RHRS
LPRM	-	Local Power Range Monitor
MAPLHGR	-	Maximum Average Planar Linear Heat Generation Rate
MCPR	-	Minimum Critical Power Ratio
MFLCPR	-	Maximum Fraction Limiting Critical Power Ratio
MPC	-	Maximum Permissible Concentration
MSIV	-	Main Steam Isolation Valve
NIOSH	-	National Institute for Occupational Safety and Health
PCI	-	Primary Containment Isolation
PCIOMR	-	Preconditioning Interim Operating Management Recommendations
RBCCW	-	Reactor Building Closed Cooling Water System
RBM	-	Rod Block Monitor
RCIC	-	Reactor Core Isolation Cooling System
RHRS	-	Residual Heat Removal System
RPS	-	Reactor Protection System
RWM	-	Rod Worth Minimizer
SBGTS	-	Standby Gas Treatment System
SBLC	-	Standby Liquid Control
SDC	-	Shutdown Cooling Mode of RHRS
SDV	-	Scram Discharge Volume
SRM	-	Source Range Monitor
TBCCW	-	Turbine Building Closed Cooling Water System
TIP	-	Traversing Incore Probe
TSC	-	Technical Support Center

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TKT-85-51

October 1, 1985

Director, Office of Inspection & Enforcement
United States Nuclear Regulatory Commission
Washington, DC 20555

Attention Document Control Desk

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of September, 1985.

Respectfully,

COMMONWEALTH EDISON COMPANY
QUAD-CITIES NUCLEAR POWER STATION

T. K. Tamlyn
Services Superintendent

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Enclosure

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