TENNESSEE VALLEY AUTHORITY DIVISION OF NUCLEAR POWER SEQUOYAH NUCLEAR PLANT

MONTHLY OPERATING REPORT TO THE NUCLEAR REGULATORY COMMISSION May 1, 1985 - May 31, 1985

> UNIT 1 DOCKET NUMBER 50-327 LICENSE NUMBER DPR-77

> UNIT 2 DOCKET NUMBER 50-328 LICENSE NUMBER DPR-79

> > P.R. Wallan

Submitted by:

P. R. Wallace, Plant Manager

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Operations Summary

May 1985

The following summary describes the significant operational activities for the month of May. In support of this summary, a chronological log of significant events is included in this report.

Unit 1

Unit 1 remained in the scheduled ice weighing/maintenance outage the entire month. All major activities were completed except the assembly of the electrical generator. There are 76.4 full power days estimated remaining until the end of cycle 3 fuel. The cycle 3 refueling outage is scheduled to begin September 13, 1985.

The scheduled returned to service date is June 15, 1985.

Unit 2

Unit 2 was critical for 488.5 hours, produced 476,610 MWH (gross), resulting in an average hourly gross load of 1,046,919 kW during the month. There are 234.3 full power days estimated remaining until the end of cycle 3 fuel. The capacity factor for the month was 54.2 percent. With a capacity factor of 85 percent, the target EOC exposure would be reached March 3, 1986.

During the month, the unit experienced two reactor scrams, no manual shutdowns or power reductions.

Significant Operational Events

		Unit 1
Date	Time	Event
05/01/85	0001	The reactor was in mode 5 with the ice weighing/maintenance outage continuing.
05/20/85	1300	The reactor entered mode 4. Electrical generator reassembly continued.
05/31/85	2359	The reactor was in mode 4. Reassembly and leak testing of the electrical generator continued.

Significant Operational Events (Cont.)

Unit 2

Date	Time	Event
05/01/85	0001	The reactor was in mode 1 at 100% power producing 1170 MWE.
05/03/85	1209	The reactor tripped following generator/turbine trip due to the loss of stator cooling flow. The 'B' stator cooling pump was placed in service and the 'A' pump was placed in standby. The 'A' pump power supply cable caught fire. The ASE shut down the 'B' pump assuming the operating pump was on fire, and tried to start the 'A' pump. Since the 'A' pump power supply had failed, these actions resulted in a total loss of stator cooling water.
05/03/85	1856	The reactor was taken critical.
05/04/85	0553	Feedwater valves 2-FCV-3-33 and -100 were found with broken stems. The failure was caused by the limit switches being improperly set following a NUREG 0588 modification. The problem was specific to fast acting valves that have enough momentum to carry the stem into the backseat after the limit switch has cut off the motor.
	0905	The reactor entered mode 3.
05/05/85	1741	The reactor entered mode 4 to repair the feedwater valves.
05/10/85	1236	The reactor entered mode 3.
05/11/85	1127	Rod N-11 in shutdown bank "D" fell 25 steps.
	1148	Rod N-11 was declared inoperable.
	1515	Began cooling down to lift the center missile barrier.
	1918	The reactor entered mode 4.
	2245	The reactor entered mode 5.

Significant Operational Events (Cont.)

Unit 2

Date	Time	Event
05/12/85	1945	The reactor entered mode 4.
05/13/85	0144	The reactor entered mode 3.
	1721	The reactor was taken critical
05/14/85	0042	The unit was tied on-line.
	0430	The unit obtained 30% reactor power.
	1406	Began power ascension.
	2101	The reactor was holding 58% reactor power. Condenser circulating water pump 2C was declared inoperable.
05/15/85	0001	Began power ascension
	0239	With the reactor at 80% power, #3 Htr Dr Tk Pmp 2C failed to start.
	0443	Increased reactor power to 87% #3 Htr Dr Tk Pmp 2C was not pumping at full capacity.
	0500	Reduced reactor power to 85%.
	0617	Reactor power 83%.
	0840	Began power ascension.
05/16/85	1400	The reactor obtained 100% power.
05/22/85	1329	The reactor tripped on an overpower Δt (OP Δt) signal generated while IMs were connecting M&TE in the process protection set II, rack 6 per TI-2.
05/23/85	0800	The reactor was taken critical
	1055	LCV-3-171 failed
	1104	The reactor entered mode 3 for repairs to LCV-3-171.
	1650	The reactor was taken critical.
05/24/85	0141	The unit was tied on-line.
	0230	The reactor obtained 30% power.

Significant Operational Events (Cont.)

Unit 2

Date	Time	Event
05/25/85	0700	Began power ascension
	1630	Held 72% power for maintenance to the A bank of the intermediate heaters.
	1800	Began power ascension.
05/20/85	0220	The reactor obtained 100% power
05/27/85	1315	Began reducing power to remove #3 Htr Dr Tk Pmp 2C from service for repairs.
	1355	The reactor was holding 84% power.
	1605	Began power ascension
	1930	The reactor obtained 100% power.
05/31/85	2359	The reactor was in mode 1 at 100% power producing 1170 MWE.

Spent Fuel Pit Storage Capabilities

Sequoyah has the capability to store 1,386 spent fuel assemblies. Two-hundred-seventy-six assemblies are presently stored in the SFP with the capacity to store an additional 1,110 assemblies.

There were three shipments of Cycle 4 (36 bundles) fuel received during the month. All fuel is presently stored in the new fuel storage vault. Thus far, no discrepancies have been found. The final three shipments are scheduled for the month of June.

PORVs and Safety Valves Summary

No PORVs nor safety valves were challenged during the month.

Licensee Events and Special Reports

The following licensee event reports (LER) were sent during May 1985 to the Nuclear Regulatory Commission.

On April 4, 1985, with both units operating at 100% power, an inadverted auxiliary building isolation (ABI) occurred during the calibration of 0-RM-90-101 (Auxiliary Building Stack Radiation Monitor). The isolation was identified and reset approximately one and one half hours after the incident occurred.

Licensee Events and Special Reports (Continued)

- 1-85015 On April 4, 1985 at 1800 CST, with unit 1 in mode 1 at 100% power, the hourly fire watch for the unit 1 auxiliary building supply air fan room could not be conducted because door A-123 would not open. The pin that secures the lower dogging pin to the dogging pin release rod had come out. The pin was replaced the door was verified operable. The fire watch resumed at 2300 CST.
- 1-85016 On April 13, 1985, Unit 1 proceeded to shutdown from 36% RTP for a scheduled outage. During the decrease in power several events occurred which were unexpected. These were:
 - 1. Simultaneous alarms of hi steam flow bistables on all four steamline loops.
 - 2. Failure of AFW to start on demand.
 - Failure of a pressurizer spray valve in the operable position.
 - 4. Inadvertent opening of the reactor trip breakers with the unit in mode 4.
- 1-85017 With unit 1 in cold shutdown (mode 5) and unit 2 at full power, an inadvertent auxiliary building isolation (ABI) occurred at 0725 CST on April 22, 1985. A second isolation occurred on May 6, 1985, at 0650 CST with unit 1 in mode 5 and unit 2 in hot shutdown (mode 4). The cause was an electromagnetic frequency feedback from welding activities causing radiation monitor RM-90-101 to indicate a high radiation signal.
- 1-85018 During an inspection of conduit to ensure compliance with the fire program requirement for sealant, three conduits were found that were not sealed on both sides of the fire barriers.
- 2-85006 On April 6, 1985, the air control valve for the essential raw cooling water valve 2-FCV-67-187 was found in the closed position. This would have prevented the supply of cooling water to CS pump room cooler 2B-B and resulted in inoperability of the CS pump. The valve was returned to its normal position.
- 2-85008 At 1200 CST on April 23, 1985, with unit 2 in mode 1 at 100% power, the hourly firewatch could not be conducted for the unit 2 auxiliary building supply air fan room or the unit 2 auxiliary building supply air intake filter room because door A-132 could not be opened. The door was opened by maintenance and the fire watch resumed with the 1700 CST hour. The cause was determined to be the failure of the door handle. The door was declared operable at 1641 CST.

Special Reports

There was one special report transmitted during the month.

85-04 The biological shield blocks were removed and the unit 1 reactor building equipment hatch was opened for the ice weighing outage. This condition was identified as a breach of a fire barrier penetration and remained non-functional for the duration of the outage.

Diesel Generator Failure Report

There were no diesel generator failure reports transmitted during the month.

Offsite Dose Calculation Manual Changes

No changes were made to the Sequoyah Offsite Dose Calculation Manual during the month.

OPERATING DATA REPORT

DOCKET NO. 50-327 DATE JUNE 13 1985 COMPLETED BY M. G. EDDINGS TELEPHONE (615) 870-6248

0.00 68.04 0.00 65.19

0.00 65.19

65.73

59.84

59.84

16.67

OPERATING STATUS

1. UNIT NAME: SEQUOYAH NUCLEAR PLANT, UNIT 1

2.	REPORT PERIOD: MAY 1985		NOTES.	
-	LICENSED THERMAL POWER (MWT): 3411.0			
	NAMEPLATE RATING (GROSS MWE): 1220.6			
	DESIGN ELECTRICAL RATING (NET MWE):			
6.	MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0		
7.	MAXIMUM DEPENDABLE CAPACITY (NET MWE):	1148.0		
8.	AF CHANGES OCCUR IN CAPACITY RATINGS(I	TEMS NUMBERS		
	3 THROUGH 7) SINCE LAST REPORT, GIVE RE	ASONS:		
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY	(NET MWE):		
4.0				
10.	REASONS FOR RESTRICTIONS, IF ANY:			
		THIS MONTH	YRTO-DATE	CUMULATIVE
11.	HOURS IN REPORTING PERIOD	744.00	3623.00	34344.00
12.	NUMBER OF HOURS REACTOR WAS CRITICAL	0.00	2467.10	23114.76
13.	REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00
14.	HOURS GENERATOR ON-LINE	0.00	2465.10	22574.05
	With the same the will be with the will		0.00	0.00
16.	GROSS THERMAL ENERGY GENERATED (MWH)	0.00	8139835.76	72817521.71
17.	GROSS ELECTRICAL ENERGY GEN. (MWH)	0.00	2818410.00	24554826.00
	NET ELECTRICAL ENERGY GENERATED (MWH)	0.00	2711314.00	23592944.00
	UNIT SERVICE FACTOR	0.00	68.04	65.73
1985	A CARLOR MAN AND A SECOND AND AND AND AND AND AND AND AND AND A	The state of the s		

24. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH):

Refueling/Modification September 13, 1985 51 days

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

NOTE THAT THE THE YR. -TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

20. UNIT AVAILABILITY FACTOR

23. UNIT FORCED OUTAGE RATE

21. UNIT CAPACITY FACTOR (USING MDC NET)

22. UNIT CAPACITY FACTOR(USING DER NET)

OPERATING DATA REPORT

DOCKET NO. 50-328
DATE JUNE 12 1985
COMPLETED BY D.C.DUPREE
TELEPHONE (615)870-6248

OPERATING STATUS

2. 3. 4. 5. 6. 7.	NAMEPLATE RATING (GROSS MWE): 1220.6 DESIGN ELECTRICAL RATING (NET MWE): 1148.0 MAXIMUM DEPENDABLE CAPACITY (GROSS MWE): 1183.0					
9.	POWER LEVEL TO WHICH RESTRICTED, IF ANY	(NET MWE):				
10.	REASONS FOR RESTRICTIONS, IF ANY:					
		THIS MONTH	YRTO-DATE	CUMULATIVE		
11.	HOURS IN REPORTING PERIOD NUMBER OF HOURS REACTOR WAS CRITICAL	744.00	3623.00	26304.00		
12.	NUMBER OF HOURS REACTOR WAS CRITICAL	488.52	3325.22	20020.34		
13.	REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	0.00		
14.	HOURS GENERATOR ON-LINE	455.25	3260.17	19530.35		
15.	UNIT RESERVE SHUTDOWN HOURS GROSS THERMAL ENERGY GENERATED (MWH)	0.00	0.00	0.00		
16.						
17.	GROSS ELECTRICAL ENERGY GEN. (MWH)	476610.00		21287220.00		
18.	NET ELECTRICAL ENERGY GENERATED (MWH)	453681.00		20479627.60		
19.	UNIT SERVICE FACTOR UNIT AVAILABILITY FACTOR	61.19	89.99	74.25		
20.	UNIT AVAILABILITY FACTOR	61.19	89.99	74.25		
21.	UNIT CAPACITY FACTOR (USING MDC NET)	53.12	83.16	67.82		
22.	UNIT CAPACITY FACTOR (USING DER NET)	53.12	83.16	67.82		
23.	UNIT FORCED OUTAGE RATE	38.81	9.86	8.80		
24.	SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS	(TYPE, DATE,	AND DURATION	OF EACH):		

NOTE THAT THE THE YR. -TO-DATE AND CUMULATIVE VALUES HAVE BEEN UPDATED.

25. IF SHUTDOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP:

UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-327

UNIT NAME Sequoyah One
DATE June 13, 1985

COMPLETED BY M. G. Eddings
TELEPHONE (615) 870-6248

REPORT MONTH MAY

	No.	Date	Type1	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code4	Component Code 5	Cause & Corrective Action to Prevent Recurrence
-9-	2	850413	5	744	В	од 4				Ice Weighing Outage
			Street Commence							

1

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Maintenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3 Mathad

Method: 1-Manual

2-Manual Scram.

3-Automatic Scram.

4-Cont. of Existing

Outage

5-Reduction

9-Other

4

Exhibit G-Instructions for Preparation of Data Entry Sheets for Licensee

Event Report (LER) File (NUREG-

0161)

5

Exhibit I-Same Source

(9/77)

UNIT SHUTDOWNS AND POWER REDUCTIONS

50-328 DOCKET NO. Sequoyah Two UNIT NAME June 13, 1985 DATE COMPLETED BY D. C. Dupree (615) 870-6248 TELEPHONE

REPORT MONTH MAY

-	No.	Date	Type1	Duration (Hours)	Reason ²	Method of Shutting Down Reactor ³	Licensee Event Report #	System Code4	Component Code 5	Cause & Corrective Action to Prevent Recurrence
	7	850503	F	252.25	A	3				Loss of Stator Cooling Water
-10-	8	850522	F	26.50	Н	3				Maintenance connecting test equipment improperly (personnel error).
			Screen v Cook							

F: Forced

S: Scheduled

Reason:

A-Equipment Failure (Explain)

B-Magntenance or Test

C-Refueling

D-Regulatory Restriction

E-Operator Training & License Examination

F-Administrative

G-Operational Error (Explain)

H-Other (Explain)

3

Method:

1-Manual 2-Manual Scram.

3-Automatic Scram.

4-Cont. of Existing

Outage

5-Reduction

9-Other

Exhibit G-Instructions for Preparation of Data

Entry Sheets for Licensee

Event Report (LER) File (NUREG-

0161)

Exhibit I-Same Source

(9/77)

SQNP AI-18 Apendix A Page 3 of 8 Rev. 24

ATTACHMENT 1 AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55 REPORT REQUIREMENTS DOCKET NO. 50-327
UNIT 1
DATE June 12, 1985
COMPLETED BY M. G. Eddings
TELEPHONE 615-870-6421

1	MAY 1985		
Day	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	0	17	0
2	0	18	0
3	0	19	0
4	0	20	0
5	0	21	0
6	0	22	0
7	0	23	0
8	0	24	0
9	0	25	0
10	0	26	0
11	0	27	0
12	0	28	0
13	0	29	0
14	0	30	0
15	0	31	0
16	0		

Ice Weighing/Maintenance Outage Continues INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

SQNP AI-18 Apendix A Page 3 of 8 Rev. 24

ATTACHMENT 1 AVERAGE DAILY UNIT POWER LEVEL

FILE PACKAGE NO. 55 REPORT REQUIREMENTS DOCKET NO. 50-328

UNIT 2

DATE June 12, 1985

COMPLETED BY D. C. Dupree

TELEPHONE 615-870-6933

1	MONTH MAY 1985		
Day	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1	1138	17	1130
2	1140	18	1127
3	1060	19	1125
4	NA	20	1129
5	NA	21	1130
6	NA	22	1087
7	NA	23	NA
8	NA	24	255
9	NA	25	608
10	NA	26	1093
11	NA	27	1003
12	NA	28	1133
13	NA	29	1134
14	409	30	1133
15	1036	31	1133
16	1131		

INSTRUCTIONS

On this format, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt.

TVA 7382A (DNP-6-81) NUCLEAR PLANT OPERATING STATISTICS

Sequoyah Nuclear Plant

11 11 11 11 11 11 12 2	tem No. 1 2 3 4 5 6 7 8 9 10 11 12 13 14 15 16	Unit No. Average Hourly Gross Load, kW Maximum Hour Net Generation, MWh Core Thermal Energy Gen, GWD (t) ² Steam Gen. Thermal Energy Gen., GWD (t) ² Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min. Capacity Factor, Percent	UNIT ONE 0 0 0 0 0 0 8,174 -8,174 N/A 11,264 0	UNIT TWO 1,046,919 1,148 58,4129 58,6740 476,610 22,929 453,681 4,81 4,967 4,784,715 4,806,104	PLANT 1,046,9 1,1 58.41 58.67 476,6 31,1 445,5 6. 16,2
11 11 11 11 11 12 2	1 2 3 4 5 6 7 8 9 10 11 11 12 13 14 15 16	Average Hourly Gross Load, kW Maximum Hour Net Generation, MWh Core Thermal Energy Gen, GWD (t) ² Steam Gen. Thermal Energy Gen., GWD (t) ² Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 0 0 0 0 8,174 -8,174 N/A 11,264 0	1,046,919 1,148 58.4129 58.6740 476,610 22,929 453,681 4.81 4,967 4,784,715	1,046,9 1,1 58.41 58.67 476,6 31,1 445,5 6. 16,2 4,784,7
	3 4 5 6 7 8 9 10 11 12 13 14 15 16	Maximum Hour Net Generation, MWh Core Thermal Energy Gen, GWD (t) ² Steam Gen. Thermal Energy Gen., GWD (t) ² Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 0 0 0 8,174 -8,174 N/A 11,264 0	1,148 58,4129 58,6740 476,610 22,929 453,681 4,81 4,967 4,784,715	1,1 58.41 58.67 476,6 31,1 445,5 6. 16,2 4,784,7
	3 4 5 6 7 8 9 10 11 12 13 14 15 16	Core Thermal Energy Gen, GWD (t) ² Steam Gen. Thermal Energy Gen., GWD (t) ² Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 0 0 8,174 -8,174 N/A 11,264 0	58.4129 58.6740 476,610 22.929 453,681 4.81 4.967 4.784,715	58.41 58.67 476,6 31,1 445,5 6. 16,2 4,784,7
	4 5 6 7 8 9 10 11 12 13 14 15 16 17	Steam Gen. Thermal Energy Gen., GWD (t) ² Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 0 8,174 -8,174 N/A 11,264 0	58.6740 476,610 22,929 453,681 4.81 4,967 4,784,715	58.67 476,6 31,1 445,5 6 16,2 4,784,
	5 6 7 8 9 10 11 12 13 14 15 16 17	Gross Electrical Gen., MWh Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 8,174 -8,174 N/A 11,264 0	476,610 22,929 453,681 4,81 4,967 4,784,715	476,6 31,1 445,9 6 16,2 4,784,
	6 7 8 9 10 11 12 13 14 15 16 17	Station Use, MWh Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	8,174 -8,174 N/A 11,264 0	22,929 453,681 4.81 4,967 4,784,715	31,1 445,5 6,1 16,2 4,784,1
	7 8 9 10 11 12 13 14 15 16 17	Net Electrical Gen., MWh Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	-8,174 N/A 11,264 0	22,929 453,681 4.81 4,967 4,784,715	31,1 445,5 6,1 16,2 4,784,1
	8 9 10 11 12 13 14 15 16 17	Station Use, Percent Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	-8,174 N/A 11,264 0	453,681 4.81 4,967 4,784,715	445,5 6,1 16,2 4,784,
	9 10 11 12 13 14 15 16 17	Accum. Core Avg. Exposure, MWD/Ton ¹ CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	N/A 11,264 0 0	4.81 4.967 4.784.715	6, 16,2 4,784,7
1 1 1 1 1 1 2	10 11 12 13 14 15 16 17	CTEG This Month, 10 ⁶ BTU SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	11,264 0 0	4,967 4,784,715	16,2 4,784,7
1 1 1 1 1 1 2	11 12 13 14 15 16 17	SGTEG This Month, 10 ⁶ BTU Hours Reactor Was Critical Unit Use, Hours-Min.	0 0	4,784,715	4,784,
1 1 1 1 1 1 1 2	12 13 14 15 16 17	Hours Reactor Was Critical Unit Use, Hours-Min.			
1 1 1 1 1 1 1 2	12 13 14 15 16 17	Hours Reactor Was Critical Unit Use, Hours-Min.		7,000,101	4,806,
1 1 1 1 1 2	14 15 16 17	Unit Use, Hours-Min.	0		7,000,
1 1 1 1 1 2	14 15 16 17	Unit Use, Hours-Min.		488.52	488.5
1 1 1 1 1 2	15 16 17		0	455:15	455:
1 1 1 2	16 17		0	54.2	27.0
1 2	17	Turbine Avail. Factor, Percent	0	87.53	43.
1		Generator Avail. Factor, Percent	0	98.29	49.
2	10	Turbogen. Avail. Factor, Percent	0	85.82	42.9
2	10	Reactor Avail. Factor, Percent	0	82.24	CONTRACTOR OF STREET
	19		0	61.19	41.
	20	Unit Avail. Factor, Percent		01.19	30
	21	Turbine Startups	0	1	
	22	Reactor Cold Startups	0	1	155
	23	Unit Service Hours	37/1	10.0/0	455.
-	24	Gross Heat Rate, Btu/kWh	N/A	10,040	10,0
1	25	Net Heat Rate, Btu/kWh	N/A	10,550	10,7
-	26	Gross Heat Rate, BTU/KWH (Without			10,00
+	27	Net Heat Rate, BTU/KWH (Without	The state of the s		10,7
-	28	Throttle Pressure, psig	0	876.50	876.
-	29	Throttle Temperature, °F	0	528.83	528.
-	30	Exhaust Pressure, InHg Abs.	0	2,5	2
-	31	Intake Water Temp., °F	0	68.1	68
all recoverage	32				
3	33	Main Feedwater, M lb/hr	0	13.6	13
3	34				
3	35				
3	36				
3	37	Full Power Capacity, EFPD	370.00	363.65	733.
3	38	Accum. Cycle Full Power Days, EFPD	293,6167	129.3210	422.93
3	39	Oil Fired for Generation, Gallons			1,7
4	40	Oil Heating Value, Btu/Gal.			138,0
4	41	Diesel Generation, MWh			
	42				
		Max. Hour Net Gen. Max. D	ay Net Gen. Loa	d	
		MWh Time Date MWh	Date Factor		
4	43	1148 2400 5-1-85 27,36	0 5-2-85 52	.16	
Re	lemai		NAME AND ADDRESS OF THE OWNER, THE PARTY OF	alue is MWD/MTU.	
		² (t) indicates Thermal Energy.			
	-				
	-				
-				~~	
		nitted Date Revised		P.R. Wall	4.

UNIT OUTAGE AND AVAILABILITY

	Nuclear Plant	
Licensed Reactor PowerMW(th)	Unit No. TWO	
Generator RatingMW(e)		Month/Year
Design Gross Electrical Rating MW		Period Hours
	Unit	

-	-	-		-	-	-	_	-	-	-	-	-	-	-	-	-	-	_	-	-	-	-	-	-	-	-	-	-	_	_	-	-	-	_	-
	TAKEN TO DREVENT	REPETITION																																	
TINIT	STATUS	DURING	OUTAGE			MODE #4																			MODE #3										1
METHOD OF	SHUTTING	DOWN	REACTOR			AUTO SCRAM																			AUTO SCRAM										
	OHTAGE CAUSE	מסו שמר השמפר				LOSS OF STATION COOLING WATER																			0.P.A.T.					00 00					The second secon
Unit	Time	In	Hrs Min	1			-	-		-	-					-	00 42										01.41	-	-	00 100				-	X
7	Time	Out	Hrs Min			12 109		-	_		-	_			-				-	-	-			-	13 129		-			-	-		-	-	X
		Unit	Hrs Min					24 00							24 100			001 00	_	00 00		00 00	-	00 00	10 31		01 141		00 1 00	-	00 00	00 00	00 00		rx
	Available	Reactor	Hrs Min	00 1 00	00 1 00	-	69 45		-	00 00	-	-	06 52	-			-	00 00		00 00	00 00		00 ; 00	00,00	-	08,10	-	-	00 00	00 + 00	-	00 1 00	-	- +	16
	Time Not Available	Gen.	Hrs Min	00 , 00	00 00	-	*	-	00 : 00		00 1 00	-	00 00	-	00 , 00	-	-	00 00	-	00 00	00 00	-	00 00	00 1 00		00 1 00	-	-	00 00	-	00 00		-	00 1 00	-
		Turbine	Hrs , Min	00 , 00		-	20 1 30	-			-	00 00	-	-	00 00	-		-		00 ' 00		-	-		00 00		-	-		00 00	-	-	-	-	-
	ple	Not Used	Hrs Min		-	-			-			-	_		-		-		-			-			,		-	-	-		-	-			
	Time Unit Available	Gen.	Min	24 00	24 1 00	12 109		-	-	00,00	-	00 1 00	00 1 00	-		-	23 18	24 00	24 00		24 1 00	-	-	24 00	-	-		-	24 1 00	24 00	-	24 1 00		24 100	-
,	Tim	Total	Min	24 00	00		1 00	1 00	1 00		00 '	00 00	00	1 00		00	118	00	1 00	00	00	00	00	00	1 29	00	61	00	00	1 00	00 1	24 00	00	00	15
			Day	1	2	2	4	5	9	7	80	6	10	11	12	13	14	15	16	171	18	19	20	21	22	23	24	25	26	27	28	29	30	31	Total 1

UNIT OUTAGE AND AVAILABILITY

SEQUOYAH	Nuclear Plant
Unit NoONE	

Generator Rating 122,05MW(e)

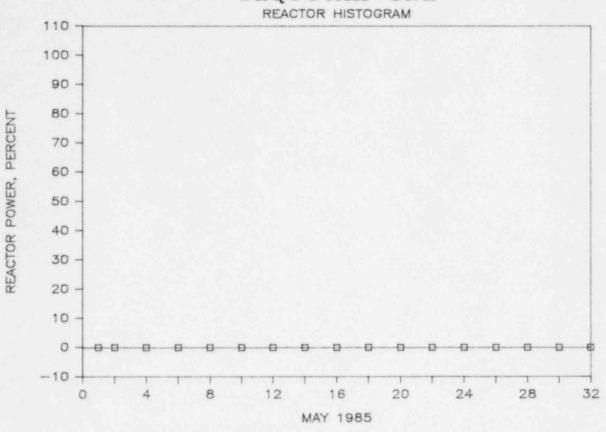
Licensed Reactor Power _, 3411 MW(th)

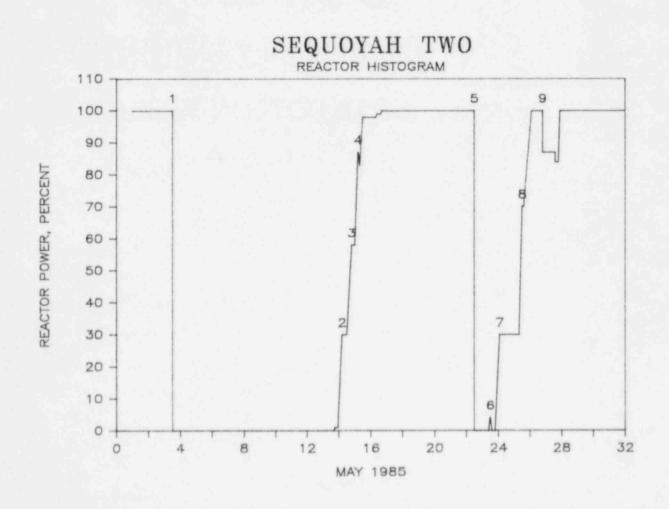
Design Gross Electrical Rating 1183 MW

Month/Year MAY 1985

							100	1	Jnit		METHOD OF	LIBOT	
	Total	me Unit Avai	Not Used		Gen.	Reactor	Unit	Time Out	Time In	OUTAGE CAUSE	METHOD OF SHUTTING DOWN	UNIT STATUS DURING	CORRECTIVE ACTION TAKEN TO PREVENT REPETITION
Day	Hrs Min		Hirs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min	Hrs Min		REACTOR	OUTAGE	REPETITION
1	00 1 00	00 00		24 00 24 00	24 '00	24 100	24 100	1		GENERATOR OUTAGE CONTINUES		MODE #5	
2	00 00	00 100	1	24 00	24 00	24 100	24 100 24 100	1	1			110012 113	
3	00 00	00 100		24 , 00	24 100	24 100	24 00						
4	00 00	00 ,00		24 1 00	24 1 00	24 00	24 00 24 00 24 00 24 00 24 00 24 00	1	1				
5	00 1 00	00 100		24 ' 00	24 ,00	24 100	24 .00	1	1				
6	00 00	00 100	1	24,00	24 100	24 !00	24 100						
7	00 00	00 00		24 . 00	24 00	24 100	24 100		1				
8	00 00	00 00		24 1 00	24 1 00	24 100	24 100	1					
			1	24 , 00	24 .00	24 100	24 100	1					
10	00 1 00	00 100 00 00 00 00 00 00 00 00 00 00 00 00 00 00 00 00 00 00		24 1 00	24 00	24 100	24 00 24 100 24 100 24 100 24 100 24 100 24 100						
11	00 1 00	00 00		24 1 00	24 1 00	24 100	24 100	1					
12	00 ! 00	00 00		24 00	24 100	24 100	24 100	1					
13	00 00	00 00		24 ! 00	24 ,00	24 100	24 .00	1	1				
14	00 00	00 100		24 00	24 00	24 100	24 00						
15	00 00	00 00		24 1 00	24 ! 00	24 00	24 100						
16	00 1 00	00 00	1	24 . 00	24 00	24 00 24 00	24 100	1	1				
17	00 1 00	00 00		24 00	24 100	24 00	24 00	1	1				
18	00 00	00 100		24 ' 00	24 .00	24 100	24 .00	1					
19	00 ,00	00 100		24 , 00	24 1 00	24 100	24 00	1					
20	00,00	00 00	i	24 00	24 00	24 .00	24 100	1 :	1				
21	00 00	00 00		24 , 00	24 00	24 .00	24 100	1	1				
22	00 00	00 100		24 00	24 00	24 100	24 00	1	1				
23	00 00	00 100	1	24 00	24 100	24 '00	24 00		1				
24	00 00	00 100		24 00	24 100	24 100	24 100	i	1				
25	00 00	00 100		24 1 00	24 00	24 100	24 100	1					
26	00,00	00 100		24 00	24 100	24 100	24 100	1	1				
27	00 1 00	00 00		24 00	24 100	24 100	24 00	1					
28	00 1 00	00 100		24 00	24 00		24 00	1	1				
29	00 00	00 100	1	24 1 00	24 00	24 100	24 100	1	1				
30	00 . 00	00 .00		24 00	24 100	24 00	24 100	1	1				
28 29 30 31	00 00 00 00 00 00	00 1 00	-	24 100	24 1 00	24 100	24 00 24 00 744100	1	1				
ot al	00 00	00 100	1	100	-4100	124 100	24 :00	k de	1				

SEQUOYAH ONE





Reactor Histogram Comments

May 1985

Unit 1

The reactor remained in the scheduled ice weighing/maintenance outage throughout the month.

Unit 2

- The reactor tripped following a turbine/generator trip due to the loss of stator cooling water.
- 2. Chemistry hold.
- 3. Stopped the power ascension for maintenance on condensate circulating water pump 2C.
- 4. Stopped the power ascension and reduced power to repair #3 Htr Dr Tk Pmp 2C.
- 5. The reactor tripped on an overpower ΔT signal that was generated as IMs were installing M&TE in Rack six of the process protection set II.
- FCV-3-171 was declared inoperable. The reactor was taken to mode
 3.
- 7. Chemistry hold.
- 8. Maintained 72% power for maintenance to the A bank of the intermediate heaters.
- Reduced power to remove #3 Htr Dr Tk Pmp 2C from service for repairs.

			ELECTRICAL MAINTENANCE FAILURE DESCRIPTION	MONTHLY REPORT FOR MAY CAUSE OF FAILURE	CORRECTIVE ACTION	PAGE 1 MR.NO
	04-11-85	2-FSV-62-74-A	VALVE IS OPEN WHILE IN P-AUTO	BAD RELAY KSIC	REPLACED RELAY KSIC	A528004
	04-16-8	0-BATA-202	REPLACE OR REPAIR BRIDGE ARM COTTER PIN	CAUSE UNKNOWN	REPLACED COTTER PIN	A298448
	04-22-85	1-LSV-62-118A	VERIFY THAT COIL INSTALLED IS A DC COIL	NO FAILURE	VERIFIED READING ON AC AND DC COILS	A520054
	04-22-85	1-FCV-63-3	MOTOR THERMALS OUT	TERMINAL BLOCK HAD GROUND	REMOVED TERMINAL BLOCK AND CLEANED	A518457
	04-24-85	0-CHGB-250-9H- T	CHECK RESISTORS AND CAPACITORS ON VITAL BATTERY CHARGER III AFTER RECHARGING BATTERY III	150 OHMS RESISTOR BURNED UP DURING RECHARGE	REPLACED 150 DHMS RESISTOR AND CHECKED FOR PROPER OPERATION	A538871
	04-25-85	0-L0CL-13-612		PHOTO DETECTORS ON XS-13-165A & B WERE BAD		A553740
-81	04-26-85	0-BCTB-32-25	ADJUST SPARE AIR COMPRESSOR B FROM 1A1-A 480V SHUTDOWN BOARD TRIP SETTING AND PLACE IN COMPARTMENT 3D ON 1A2-A SHUTDOWN BOARD		ADJUSTED TRIP SETTINGS TO MATCH SETTINGS ON BAD BREAKER FOR COMPRESSOR A	A527519
	04-26-85	0-BCTB-32-25	INVESTIGATE CAUSE OF COIL FAILURE ON 1A2-A 480V SHUTDOWN BOARD	COIL BURNED UP DUE TO DEFECTIVE COIL	INVESTIGATED AND FOUND TRIP COIL RECEIVED A VALID TRIP SIGNAL	A527520
	04-27-85	2-FCV-74-16	INSPECT AND REPAIR LOOSE FLOW CONTROL VALVE LIMIT SWITCH	LOOSE MOUNTING BOLTS	TIGHTENED LODSE MOUNTING BOLTS	A535210
	04-27-85	0-L0CL-13-606	ALARM IN ON PANEL, ZONE	BAD PANEL CARD	REPLACE 2A 30 CARD	A528029

	the late and the second		ELECTRICAL MAINTENANCE I	A CONTRACT OF THE PROPERTY OF	CORRECTIVE ACTION	PAGE 2 MR. NO
			87			
	04-28-85	2-FSV-70-153	1A CONDUIT 2V2557B IS BROKEN AT ENTRANCE OF BOX 1234 TRAIN B		REMOVED BROKEN PORTION OF THREADS AND MADE CONDUIT BACK UP TO FITTING	A234561
	04-29-85	1-FCV-43-64	VALVE WILL NOT SWITCH TO OPEN POSITION	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	A540510
	04-29-85	2-FCV-63-84	REPAIR LIMIT SWITCH	ACTUATOR ARM OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH ACTUATOR ARM	A526405
	05-02-85	1-PCV-68-334	BOTH RED AND GREEN LIGHTS ARE ON WHILE VALVE IN CLOSED POSITION	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH	A528851
	05-03-85	1-TS-13-198A	BROKEN THERMO SENSOR CONNECTED BY CONDUIT FE 4706	CAUSE OF FAILURE UNKNOWN	REPLACED THERMAL DETECTOR	A523865
10	05-04-85	2-FSV-63-70	LIMIT SWITCH WILL NOT CLEAR WHEN VALVE FULLY OPEN		ADJUSTED LIMIT SWITCH AND CHECKED FOR PROPER OPERATION	A300186
	05-04-85	2-FSV-63-167	VALVE WILL NOT INDICATE OPEN POSITION	VALVE ACTUATOR ARM OUT OF ADJUSTMENT	ADJUSTED ACTUATOR ARM ON VALVE AND CHECKED FOR PROPER OPERATION	A518362
	05-07-85	2-L0CL-13-630	PERFORM SI-234.7 ON ZONES 366, 367, 358, 359, 362, 363, 370, 371, 353 NEXT TIME UNIT 2 IS DOWN	NO FAILURE, SURVEILLANCE INSTRUCTION	PERFORMED SI-234.7 FOR LISTED ZONES	A532814
	05-08-85	2-MVDP-3-47	REPLACE BROKEN TORQUE SWITCH	CAUSE UNKNOWN	REPLACED BROKEN TORQUE SWITCH	A542214

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		ELECTRICAL MAINTENANCE FAILURE DESCRIPTION	MONTHLY REPORT FOR MAY CAUSE OF FAILURE	CORRECTIVE ACTION	PAGE 3 MR.NO
05-09-85	1-BATB-82-UC-B	TORQUE THE END STRINGER BOLTS ON DIESEL GENERATOR BATTERY 18-8		TORQUED END STRINGER BOLTS	A538936
05-09-85	1-FCV-3-126-A	VALUE NOT OPERATING PROPERLY FROM HAND SWITCH IN CONTROL ROOM	BAD VALVE HANDWHEEL	REINSTALLED HANDWHEEL	A527566
05-10-85	1-ZS-1-22	LIGHT INDICATOR DOES NOT COME ON WHEN VALVE IS PARTIALLY STROKED	BAD LIGHT SOCKET	CHANGED INDICATING LIGHT SOCKET	A121544
05-10-85	0-CHR-250	REPLACE DEFECTIVE COMPONENTS ON VITAL BATTERY CHARGER I	CAUSE UNKNOWN	REPLACED 3 WIRES TO CAPACITOR BANK AND RELUGGED TWO EXISTING WIRES. INSTALLED AC FAILURE RELAY	A538934
05-12-85	1-FCV-62-85	RED AND GREEN LIGHTS ARE ON WITH HAND SWITCH IN OPEN POSITION	STEM BLOCK ROLLED OFF LIMITS	LINED STEM BLOCK UP AND CHECKED FOR PROPER OPERATION	A518456
05-14-85	1-CMP-311	ADJUST AGASTAT RELAY 62/173 TO ALLOW 4 MIN. TIME DELAY PICKUP	CAUSE UNKNOWN	CALIBRATED RELAY TO 4	A126810
05-14-85	2-COMP-311	ADJUST AGASTAT RELAY 62/175 TO ALLOW 4 MIN. TIME DELAY ON PICKUP	CAUSE LINKNOWN	CALIBRATED TIME DELAY RELAY TO 4 MIN.	A126811
05-14-85	1-COMP-311	ADJUST AGASTAT RELAY 62/172 TO ALLOW 4 MIN. TIME DELAY ON PICKUP	CAUSE UNKNOWN	CALIBRATED TIME DELAY RELAY TO 4 MIN.	A126809
05-16-85	1-PCV-68-334	BOTH RED AND GREEN INDICATOR LIGHTS ARE ON	REED SWITCH OUT OF ADJUSTMENT	ADJUSTED REED SWITCH	A527582

20-

			FAILURE DESCRIPTION	MONTHLY REPORT FOR MAY CAUSE OF FAILURE	CORRECTIVE ACTION	PACE 4 MR.NO
			WHILE VALVE IS CLOSED			
	05-16-85	1-FCV-30-19	BOTH LIMIT LIGHTS ARE ON WHEN VALVE IS IN CLOSED POSITION	LIMIT SWITCH BRACKET WAS BENT AWAY FROM VALVE		A531009
	05-16-85	1-FCV-62-73	CLOSED LIMIT SWITCH LIGHT NOT OPERATING PROPERLY	LIMIT SWITCH OUT OF ADJUSTMENT	ADJUSTED LIMIT SWITCH AND LIGHT OPERATED PROPERLY	A528877
	05-17-85	0-XFD-313-911	REPLACED FIRE DAMPER FUSIBLE LINKS	CAUSE UNKNOWN	REPLACED FUSIBLE LINKS	A526417
	05-17-85	0-XFD-313-905	RESET ELECTRICALLY OPERATED FIRE DAMPER	BAD FUSIBLE LINKS	REPLACED FUSIBLE LINKS	A526410
	05-17-85	0-XFD-313-904	RESET ELECTRICALLY OPERATED FIRE DAMPER	BAD FUSIBLE LINKS	REPLACED FUSIBLE LINKS	A526411
-21-	05-17-85	0-XFD-313-910	REPLACE FIRE DAMPER FUSIBLE LINKAGE ASSEMBLY	and the second second	REPLACED FIRE DAMPER FUSIBLE LINKS	A526418
	05-18-85	1-FCV-30-10	LIGHT INDICATING SWITCH IS BAD	CAUSE UNKNOWN	REPLACED INDICATING LAMP HOUSING	A527556
	05-23-85	1-LT-63-119	CLEAN BLOCKAGE IN SENSE LINE TAP BETWEEN THE TANK TAP AND 1ST TEE	CAUSE UNKNOWN	INSTALLED TEMPERATURE HEAT TRACE AND INSULATED PIPE. OPENED HIGH SIDE VALVE AND VERIFIED H20 BLEEDING INSTEAD OF NITROGEN	A042628

36 records listed.

INSTRUMENT MAINTENANCE

Unit 1

Performed calibration (SI-196) of the UHI level switches. LS-87-21 and -23 were found out of tech spec tolerance. PRO 1-85-150 was written.

Steam flow transmitter FT-1-28A (Barton model 764) was replaced because of repeated instrument drift.

During shutdown for the ice outage both pressurizer spray valves were found inoperable. The positioner on PCV-68-340B had a broken drive arm and hub pin and the I/P was defective on PCV-68-340D. Both spray paths were repaired and calibrated.

Air was found in the sealed sense lines on both trains of RVLIS. Both trains were evacuated and refilled and pressure tested to ensure proper fill. Also the RVLIS transmitters were recalibrated.

Two auxiliary building isolations occurred due to spiking on radiation monitors. PROs 1-85-175 and -177 were initiated. Both appear to have been caused by spurious signals.

Workplan 11123 removed the pressure indicating controllers associated with auxiliary feedwater discharge pressure on both motor driven aux feedwater pumps. The loops were modified for pressure indication only since the pressure control valves were replaced with cavitating venture.

Workplan 11488 installed new controllers with proportional control only on the mini flow recirculation line on the main feedwater pumps. The controller removed had proportional control with reset.

Workplan 11463 recalibrated the pressure switches on the motor driven and turbine driven auxiliary pump suction pressure which controls condensate to ERCW switchover. It also set the time delay relays associated with the flow control valves.

Workplan 11494 implemented the installation of automatic control on the main feedwater bypass control valves for steam generator level control during low power operation. The controllers have been installed on a TACF. The TACF will be cleared during the next refueling outage when new controllers are installed under ECN 6193.

All of the steam generator auxiliary feedwater level indicating controllers were removed, bench aligned, calibrated, reinstalled, and the controll valves stroked to ensure operability when the unit returned to service.

INSTRUMENT MAINTENANCE (Cont.)

During the solder modification performed on Barton lot 2 transmitters, transmitter LT-3-51 would not calibrate within acceptable limits. A new transmitter was acquired, installed, calibrated and the loop declared operable.

During the ice outage the P-250 was removed from service for maintenance and diagnostic testing. All circuit cards and connectors were cleaned and memory capacity was upgraded.

UNIT 2

Performed calibration (SI-196.2) of the UHI level switches. One switch LS-87-22 was found out of tech spec tolerance. PRO-85-066 was written.

During startup control rod N11 fell in after being stepped out about twenty five steps. The unit was cooled down to mode 5 and the problem was isolated to burnt pins in the connector. Thermal lock-up occurred since the rod was on bottom during cooldown and the rod could not be moved until we heated up to about 300°F and vibrated the rod with the lift coil. Prior to startup a rod drop test was performed on N11 to verify that it was within tech spec limits.

Train B containment hydrogen analyzer was found to have erratic oscillations during calibration using calibrated gas samples. The oscillations were minimized by purging the long sample lines with an increased flow rate of sample gas. Apparently pockets of different percent hydrogen sample gas were being trapped in the sample line and contaminating the sample we were using for calibration.

MOUNTING BOLTS

		I MO	KUTEN	1194	INTENHALE	COMP	SUPPRAKT 06-05-85 PAGE 1	
	MR.COM	P U	FLINC	SYS	ADDRESS.	DATE	DESCRIPTION	CORRECTIVE ACTION
	A11445	6 1	LS	087	24	05/20/85	1-LS-087-24 REPLACE SW INTERNAL SCREW ON MICROSW IS STRIPPED	BAD SW. REPLACE SW
	A12154	9 1	ZS	001	22	05/13/85	1-ZS-001-22, THIS MSIV GREEN LIGHT INDICATION DOES NOT INDICATE WHEN PARTIAL STROKING THE VLV.	EAST VALVE RM
	A12298	3 :	LI	977	1	05/06/85	2-LT-077-1, *I* CHECK CALIB, OF XMITTER	XMTR OUT OF CAL. RECAL XMTR
	A23346	3 7	FT	003	147	05/15/85	2-FT-003-147, VERIFY CALIB OF 2 FT 3 147 UP TO THE TEST POINT IN BACKUP CONTROL RM	NONE. VERFIED CAL ON XMTR
	A23346	5 3	FT	003	170	05/16/85	2-FT-003-170, VERIFY CALIB OF 2 FT 3 170 UP TO THE TEST POINT IN BACKUP CONTROL RM	NONE. VERIFIED CAL AND TOUCHED UP CAL
	A23346	6	? FT	003	155	05/16/85	2-FT-003-155, VERIFY CALIB OF 2 FT 3 155 UP TO THE TEST POINT IN BACKUP CONTROL RM	HONE. RECAL XMTR
	A23346	7 7	FT	003	163	05/15/85	2-FT-003-163, VERIFY CALIB OF 2 FT 3 163 UP TO THE TEST POINT IN BACKUP CONTROL RM	NOME. RECALIB XMTR
	A23750	3 2	2 CON	363	1V7968A	05/10/85	2-CON-363-1V7968A, ON 2-FCV-1-29 - U2 WEST VLV RM-REPAIR THE FLEX PORTION OF THE BOX.SEE 45N826-21 DET E21	WEST VALVE RM.
	A28178					05/03/85	1-LT-068-320, CHECK CALIB, OF 1-LT-68-32	XMTR WAS FOUND SLIGHTLY OUT OF CALIB ON THE HIGH SIDE. RECALIB THE XMTR
2	A29177	8	PT	068	322	05/03/85	1-PT-068-322. HARD WIRE PIN CONNECTOR FOR BARTON LOT 2 XMITTER TO MAINTAIN QUALIFICATIONS PER SMI 1 317 23	
	A29178	4	LT	068	335		1-LT-068-335, HARD WIRE PIN CONNECTOR FOR BARTON LOT 2 XMITTER TO MAINTAIN QUALIFICATIONS PER SMI 1 317 23	
	A29179	7	U	063	179	05/08/85	1-LT-063-179, HARD WIRE PIN CONNECTOR FOR BARTON LOT 2 XMITER TO MAINTAIN QUALIFICATIONS PER SMI 1 317 23	NONE. HARDWIRED PIN CONNECTOR
	A29488	2	LS	087	24	05/13/85	1-LS-087-24, TUBING CONNECTIONS LEAKING	BAD FITTING CONNECTIONS. REPLACED TUBING AND CONNECTIONS
	A29675	0	I TI	068	316	05/06/85		RTD LEAD SHORTED TO CASE. REPAIRED TO CLEAR SHORTED LEAD
	A29848	6	LT	003	51	05/30/85	1-LT-003-51, REPLACE LVL TRANSMITTER	NONE. REPLACED XMTR FOUND LOW IN CAL DURING SMI 1 317 23
	A29848	7	5 FCA	003	171	05/09/85	2-LCV-003-171, TORGUE RELAY ASSEMBLY SCREWS AND MOUNTING BOLTS	NONE. VERIFIED TORQUE (RETORQUED) VERIFIED CAL AND STROKE AFTER TORQUEING
	A29848						MOUNTING BOLTS	NONE. RETURQUED SCREWS TO PROPER SETTING-VERIFIED CAL AND OPERATION
	A29848						2-LCV-003-156, TORQUE RELAY ASSEMBLY SCREWS AND MOUNTING BOLTS	NONE. RETURBUED SCREWS AND VERIFIED CAL AND PROPER OPERATION
	A29849	0	2 LCV	003	148	05/09/85	2-LCV-003-148, TORGUE RELAY ASSEMBLY SCREWS AND	NOME. VERIFIED RELAY SCREWS PROPERLY

TORQUED AND VERIFIED STROKE

1		INST	RUMENT	MA.	INTENANCE	MONTHLY S	LIMMARY 06-05-85 PAGE 2
	MR.COM	Рυ	FUNC	SYS	ADDRESS.	DATE	DESCRIPTION CORRECTIVE ACTION
	A29849	1 1	LCV	003	172	05/13/85	1-LCV-003-172, TORQUE RELAY ASSEMBLY SCREWS AND NONE, TORQUED SCREWS AND VERIFIED PROPER RELAY MOUNTING BOLTS. UPDATE EQUIS OPERATION
	A29849	2 1	LCV	003	173	05/16/85	1-LCV-003-173-, TORQUE RELAY ASSEMBLY SCREWS AND FOUND A BAD CAPACITOR. REPLACED THE BAD CAPACITOR
	A29849	3 1	LEV	003	174	05/14/85	1-LCV-003-174-, TORQUE RELAY ASSEMBLY SCREWS AND NONE. RETORQUED SCREWS VERIFIED CAL AND RELAY MOUNTING BOLTS. UPDATE EQUIS STROKE
	A29849	4 1	LCV	003	175	05/13/85	1-LCV-003-175-, TORQUE RELAY ASSEMBLY SCREWS AND NONE. RETORQUED VERIFIED PROPER RELAY MOUNTING BOLTS UPDATE EQUIS OPERATION
	A29925	ŏ 1	PM	880	340H	05/03/85	1-PM-068-340H, REPAIR/REPLACE I/P MODULE BLOCKAGE IN I/P REPLACED I/P AND CAL ALSO RESTROKE VLV
	A29926	0 1	LT	063	179	05/07/85	1-LT-063-179, TOP OFF SENSE LINE FILL AND CALIB FILLED LEG LOW. REFILLED XMTR LINE AND CALIB
	A29926	1 1		968	RVLIS	05/14/85	1-068-RVLIS, TROUBLE SHOOT SYSTEM APPEARS TO BE FILL LOW AND ZERO OFFSET, REFILLED AND A PROBLEM WITH UPPER PLENUM XMTRS ADJUSTED CAL
	A29926	2 1	LT	068	369		1-LT-068-369. *I* DRN AND PURGE RVLIS HEAD SENSE NONE, DRAINED AND PURGED SENSE LINES BECAUSE OF THE UNIT BEING DOWN FOR MAINT RETURNED TO SERVICE PER SI 484
	A29926	3 1	LT	068	36272	05/20/85	1-LT-068-36272, XIX DRN AND PURGE RVLIS UPPER NONE. DRN AND PURGE SENSE LINE TO PUT PLENUM SENSE LINE TO PUT INTO DEPRATION
	A30077	9 1	PT	068	322	05/23/85	1-PT-068-322 PRESS TEST SENSE LINE FOR 1 PT 68 NONE, SMALL LEAK, FOUND 2 SMALL LEAKS 322 REPAIRED AND RETURNED TO SERVICE
	A30117 A30171						2-XA-055-1A, CHECK ANNUNICATOR CARD PROBLEM WITH POWER AMP DRAWER #2 1-LS-087-22, REPLACE LVL SW WITH A NEW ONE W/SAME SW CONSISTANTLY OUT OF TOLERANCE. MODEL AND RANGE REPLACED SW
	A30172	4 1	PCV	001	23	05/21/85	1-PCV-001-23, REPLACE REGULATOR REGULATOR IS REGULATOR LEAKING AIR. REPLACED REGULATOR STROKED VLV BY OPERATIONS FROM THE CONTROL RM TO VERIFY PROPER OPERATION RETURNED TO SERVICE
	A30189	0 1	FCV	003	35A	05/16/85	1-FCV-003-35A, STROKE VLV. PERFORM FUNCTIONAL OUT OF CALIB AND ADJUSTMENT BECAUSE OF REPLACEMENT OF VLV STEM PACKING. RECALIB SET LEFT OFF PRESS REPLACED PILOT VLV ASSEMBLY STROKED VLV TO VERIFY SMOOTH OPERATION PER IMI 134 INSTRUCTIONS
	A30189	2 1	FCV	003	90A	05/17/85	1-FCV-003-90A, STROKE VLV. PERFORM FUNCTIONAL ON VLV AND MODIFIER OUT OF CAL. RECAL MODIFIER AND RESTROKE VLV
	A51845	4 1	PCV	068	3408	05/03/85	1-PCU-068-3408, SPRAY VLV IS STUCK OPEN; INVESTIGATE AND REPAIR VLV OVERTRAVEL BROKE FEEDBACK ARM. RECAL I/P AND REPLACED FEEDBACK ARM AND CAM HUB
	A52643	6 . 2	LEV	003	171	05/24/85	2-LCV-003-171.*I* LVL CONTROL VLV OPENS IN AUTO AIR LEAK. TIGHTENED FITTINGS TO CORRECT AND MANUAL WHEN IT HAS NO LVL DEMAND SIGNAL PROBLEM

-25-

1)	NST	RUMENT	MA)	INTENANCE		SUMMARY 06-05-85 PAGE 3	
MR.COMP	U	FUNC	SYS	ADDRESS.	DATE	DESCRIPTION	CORRECTIVE ACTION
A526458	2	LIC	003	175	05/21/85	2-LIC-003-175, LVL IND CONTROLLER IS NOT SHOWING ACTUAL LVL	LVL INDICATOR STUCK. FREED UP INDICATOR AND VERIFIED READING WITH LIKE INDICATOR
A527513	1	FI	890	290		1-FI-068-29D FLOW IND IS SHOWING 45% FLOW WHEN ACTUAL FLOW IS ZERO	
A527548	10.3	LI	003	107	05/09/85	1-LI-003-107, LVL B INDICATING GREATER THAN 100% WHILE ACTUAL LVL IS 30%	PNL ISOL VLVS FOUND CLOSED AND THE LOSS OF BACKFILL. BACKFILLED TRANSMITTER-VLVED IN AND RETURNED TO SERVICE
A529041	2	FT	030	242	05/02/85	2-FT-030-242, FLOW DOES NOT APPEAR TO BE INDICATING CORRECT	AIR IN SEMSE LINES SQUARE, ROOT CONVERTER OUT OF CAL HI XMTR OUT OF CAL LD, BLED SEMSE LINES RECAL XMTR AND SQ RT CONVERTER
A528316	2		099	TRB	05/03/85	2-099-TRB,RT B WILL NOT CLOSE DUE TO HI FLUX TRIP ON 1 OF 2 IR EXCORE DETECTOS	The state of the s
A528609	2	PIC	001	13		2-PIC-001-13,*I* CHECK OUT CONTROLS ON VLV. VLV WILL NOT STROKE	201 (201 (201 (201 (201 (201 (201 (201 (
A528865	. 2		085	NII	05/13/85	2085-N11, SHUTDOWN BANK D ROD N11 WILL NOT WITHDRAW	BURNED PENS IN CONNECTOR, REPLACED PENS IN CONNECTOR AND VERIFIED PROPER OPERATION
A528885	1 1	Ц	077	410	05/29/85	1-LI-077-410, ×I× RECALIB LVL INDICATION RUN CALIB LOOP	LOOSE SCREWS ON TERMINAL BOARD. TIGHTENED SCREWS ON TERM BOARD AND VERIFIED PROPER OPERATION
A530380	. 2	TI :	968	316	05/29/85	2-TI-068-316, CHECK CALIBRATION ON INSTRUMENT. INSTRUMENT IS READING LOW WHICH IS CAUSING ANN.	NONE. VERIFIED CAL ON LOOP COMPONENTS AS
A531002	1	TR	068	1P001	05/28/85	1-TR-068-19001, RED PEN FOR LOOP 1 RCS HOT LEG TEMP IS READING APPROX 25 DEG HIGHER THAN OTHER	RECORDER OUT OF CAL. RECAL RECORDER

05/23/85 1-LI-003-107, XIX DEVIATING FROM THE OTHER TWO BY AIR IN XMTR SENSE LINES. BACKFILLED

05/30/85 1-FI-068-480, FLOW INDICATOR FI-68-48D IS LO SIDE VLV CLOSED. BLED DOWN XMTR

SENSE LINES

3 LOOPS HOT LEGS INDICATORS

DRIFTING BETWEEN 40% AND 60% FOW WITH #3 RCP OFF

) 5%

A531054 1 LI 003 107

A531096 1 FI 068 48D

	D	NST	RUMENT	nal	INTENANCE		SUMMARY 06-05-85 PAGE 4	
	MR.COMP	U	FUNC	SYS	ADDRESS.	DATE	DESCRIPTION	CORRECTIVE ACTION
	A535239	- 2	FEV	003	84		2-FCV-003-84, XIN VLV WILL NOT STROKE FROM MCR	
	A541565	1	LI	003	94	05/09/85	WHEN USE THE CONTROLLER ON 2 M 3 1-LI-003-94, LVL IND IS INDICATING 28% WHILE ACTUAL LVL IS APPROX 13%	AIR IN XMTR SENSE LINE. BACKFILLED SENSE
	A541574	1 2	LIC	003	148	05/11/85	2-LIC-003-148, WHEN INDICATED LVL SHOWS >46% LVL AND FCV 3 148 IN AUTO-FI 3 147 SHOWS >0R=100 GPM $$	CONTROLLER RESET ACTION OUT OF
	A541576		LIC.	003	171		2-LIC-003-171, WHEN INDICATED LVL SHOWS >46% AND FCV 3 171 IN AUTO AND CLOSED FI 3 170 SHOWS APPROX 150 GPM FLOW, WHEN PLACED IN MAN & MAN	
	A541955	1	LT	003	175		APPEARS TO BE INCORRECT	POSITIVE LEAD BROKEN OFF ON XMTR. REPLACED POSITIVE AND NEGATIVE LEADS
	A541956	1	LT	003	171		1-LT-003-171,0.0825 VOLTS WERE OBSERVED ACROSS	FROM TERMINAL BLOCK TO XMTR NONE, NONE/CHECKED CIRCUIT NO PROBLEM FOUND
2.0	A545805	1	PCV	001	12	05/21/85		AIR REGULATOR BLOWING AIR. REPLACED REGULATOR STROKED VLV BY OPERATIONS FROM CONTROL RM TO VERIFY PROPER OPERATION RETURNED TO SERVICE
	A545811		PCV	001	31			BAD GASKET AND AIR REGULATOR. REPLACED GASKET AND AIR REGULATOR AND VERIFIED VLV STROKE
	A545814		PCV	001	6	05/21/85	1-PCV-001-6, POSITIONER LEAKING AIR BADLY AND AIR REGULATOR FOR POSITIONER AND VLV IS LEAKING	BAD GASKET AND AIR REGULATOR, REPLACED
	A548768		LT	063	178		1-LT-063-178, HARD WIRE PIN CONNECTOR FOR BARTON LOT 2 XMITTER TO MAINTAIN QUALIFICATIONS PER SMI 1 317 23	NONE. HARDWIRED PIN CONNECTOR REPLACED
	A548807		2 PCV	068	3400		2-PCV-068-3400, XIX CHECK STROKE OF VLV ENSURE ROLLER CANNOT EXCEED END OF CAM	
	A564828	3	I LM	003	148A	05/01/85	1-LM-003-148A, INSPECT I/P INTERNAL MECHANISM AND	THE RESERVE OF THE PROPERTY OF THE PARTY OF

COIL ASSEMBLY FOR ALIGNMENT AND TIGHTNESS

COIL ASSEBBLY FOR ALIGNMENT AND TIGHTNESS

COIL ASSEMBLY FOR ALIGNMENT AND TIGHTNESS

RELAY MOUNTING BOLTS. UPDATE EQUIS

05/01/85 1-LM-003-156A, INSPECT I/P INTERNAL MECHANISM AND NONE. TIGHTENED SCREWS FOR QUALIFIED

05/13/85 1-LCV-003-148-, TORQUE RELAY ASSEMBLY SCREWS AND NONE, REPLACED RELAY WITH HI TEMP UNIT

05/01/85 0-LM-003-164A, INSPECT I/P INTERNAL MECHANISM AND NONE. CHECKING FOR PROPER TORQUE.

A564829 1 LM 003 156A

A564830 0 LM 003 164A

A564840 1 LCV 003 148

TIGHTNESS FOR QUAL LIFE

TIGHTENED RELAY MOUNTING SCREWS TO

TORQUED AND VERFIED PROPER OPERATION

LIFE

PROPER TORQUE

	INSTRUMENT		MAINTENANCE		MONTHLY	SUMMARY 06-05-85		PAGE	5		
MR.	COMP	U	FUNC	SYS	ADDRESS.	DATE	DESCRIPTION	*******			CORRECTIVE ACTION
A56	4841	1	LCV	003	156	05/15/85	1-LCV-003-156, REPLA POSITIONER	ACE RELAY S	SUB ASSEMBLY	Y ON ULU	NONE. REPLACED RELAY TO SATISIFY 1E REGMTS
A56	4842	1	LCV	003	164	05/13/85	1-LCV-003-164, REPLA POSITIONER WITH HIG STORES				NONE. REPLACED WITH HI TEMP MODEL TORQUE AND VERIFY CAL
A56	4843	1	LCV	003	171	05/13/85	1-LCV-003-171, REPLA POSITIONER	ACE RELAY S	SUB ASSEMBLY	Y ON VLV	NONE. REPLACED RELAY WITH HI TEMP MODEL AND TORQUED AND VERIFIED OPERATION
A56	4844	1	LCV	003	172	05/02/85	1-LCV-003-172, REPLA POSITIONER	NCE RELAY S	SUB ASSEMBLY	Y ON VLV	NONE. REPLACED RELAY WITH HI TEMP UNIT
A56	4845	. 1	LEV	003	173	05/02/85	1-LCV-003-173, REPLA POSITIONER	ACE RELAY S	SUB ASSEMBLY	Y ON VLV	NONE. RELAY SUB ASSEMBLY CHANGED TO HIGH TEMP UNIT. CHANGED TO HI TEMP UNIT. NONE. REPLACED RELAY WITH HI TEMP UNIT.
A56	4853	2	LEV	003	172	05/08/85	2-LCV-003-172, TORGE MOUNTING BOLTS	JE RELAY AS	SEMBLY SCRE	EWS AND	NONE. TORQUED SCREWS AND VERIFIED PROPER OPERATION
A56	4854	2	LCV	003	173	05/08/85	2-LCV-003-173, TOROL MOUNTING BOLTS	JE RELAY AS	SEMBLY SCRE	EWS AND	NONE. TORQUED SCREWS AND VERIFIED PROPER OPERATION
A56	4855	2	LEV	003	174	05/08/85	2-LCV-003-174-, TORK MOUNTING BOLTS	BUE RELAY A	ASSEMBLY SCI	REWS AND	NONE. TORQUED SCREWS AND VERIFIED PROPER OPERATION
A56	4856	2	LEV	003	175	05/08/85	2-LCV-003-175, TORGE MOUNTING BOLTS	JE RELAY AS	SEMBLY SCRE	EWS AND	NONE. TORQUED SCREWS TO PROPER TIGHTNESS AND VERIFIED PROPER OPERATION
A56	4857	1	LEV	062	118	05/28/85	1-LCV-062-118, VLV (AUTO	DOES NOT CO	ONTROL PROPE	ERLY IN P	VLV DOESN'T CONTROL PROPERLY IN P-AUTO.NONE. TOOK VOLTAGE READING ALL
											OK PER ENGR AND RETURNED TO SERVICE

73 records listed.

Mechanical Maintenance Section

May 1985

Unit 0

- 1) Performed the monthly diesel general inspection.
- 2) Replace the mechanical seals on the 1B-B, 2A-A, and 2B-B boric acid transfer pumps per workplan 11528.
- Installed new bolting on the "A" boric acid filter per TACF.
- 4) Replaced the filter and unplugged the inlet line on the "A" boric acid filter.
- 5) Plugged 3 tubes in the "B" auxiliary boiler.

Unit 1

- 1) Inspected the #2 and #3 seals on reactor coolant pump #4.
- 2) Completed servicing the ice condenser.
- 3) Replaced the plug seat and gasket in 1-LCV-3-156.
- 4) Lapped the seats in 1-LCV-3-164.
- 5) Changed out the rotor on the 1-A low pressure turbine.
- 6) Changed out the "A" reactor coolant drain tank pump.
- 7) Replaced solenoid valve on 1-LCV-62-118 and 1-FCV-62-74.
- 8) Completed sweeping and venting the steam generators.
- 9) Plugged 3 tubes in the "A" spent fuel pit heat exchanger.
- 10) 1-VLV-67-561A was leaking through and badly galled so it was replaced per work plan 11597.
- 11) Repaired the galled stem and backseat on 1-FCV-5-42.
- 12) Changes some extraction steam line at 1-FCV-5-97 from carbon steel to stainless steel due to erosion problems.
- 13) Repaired the stator bars in the main generator to eliminate the hydrogen leak.

Mechanical Maintenance

May, 1985

(Continued)

Unit 2

- 1) Replaced the inboard bearing in the 2-A condenser vacuum pump.
- Repaired the crack on the pipe at the floor mounted spring cam by 2-LCV-6-106 A&B.
- 3) Replaced the broken stems on 2-FCV-3-33 and 2-FCV-3-100.
- 4) Welded on a scab plate at 2-LCV-6-106B.
- 5) Changed the bearings in the buss duct cooling fan.
- 6) Replaced the air side seal oil pump.
- 7) Welded a brace and realigned the 2B stator cooling water pump.
- 8) Replaced the regulator diaphram on the 2-PCV-1-12.
- 9) Repaired a Furmanite job on 2-FCV-1-291.

SUMMARY OF WORK COMPLETED

MODIFICATIONS

MAY 1985

NUREG 0588

ECN 5746 - Component Cooling System Pump Motor Replacement

The final pump motor was changed out this period.

ECN 5824 - MOV Operator Replacement

All twelve remaining operators have been replaced.

ECN 5883 - Pressure Switch/Flow Switch Replacement

Final tie-in for the flow switch replacement is in progress.

ECN 5970 - MOV Operator Replacement

One remaining operator will be replaced when the unit is off residual heat removal.

ECN 5971 - MOV Operator Replacement

All three operators have been replaced

ECN 6207 - Conax Connectors

The final connector was installed during this timeframe.

ECN 6278 - ABGTS and EGTS Heater Controller Replacement

The moisture sensor was jumpered out this month. Two rubber hoses will be replaced the first week in June.

ECN 6398 - Limit Switch Replacement

All ten limit switches were replaced by relays in this period

ECN 6404 - Replace Limit Switches

Three limit switches were replaced in System 63 by this ECN.

ECN 6408 - Replace Gears in MOV Operator

Two operators were qualified by replacing the motors and gears.

ECN 6409 - Replace Limit Switches

Two limit switches were replaced in System 65 by this ECN.

All identified mechanical activities have been completed. Inservice leak checks will be performed on unit 1 during startup.

Appendix R

ECN 5484 - Emergency Lights

Work is in progress to install new lights.

ECN 6209 - Wrap Fire Protection Blanket Around Conduit

Field work is complete.

ECN 6235 - Reroute Various Cables

The eight workplans are in the approval cycle. Two additional workplans are being written.

ECN 6305 - Elevation 714 Fire Barrier

Workplan has been prepared and placed in the approval cycle.

ECN 6319 - Fire Protection Piping

A large amount of material was delivered to the warehouse and is being received. Prefabrication work on hangers began at the end of the month. Outages for piping reroutes, etc., are planned for early June. Three more workplans were placed in the approval cycle.

ECN 6316 - Seal for Penetrations

Sealing work is complete.

Other Items

DCR 1739 - Install VAACS Computer

System is functional.

DCR L2108 - Flammable Liquid Storage Building

This project is complete, and the building is in use.

ECN 5009 - ERCW Piping Changeout from Carbon Steel to Stainless Steel

The prefabrication of piping assemblies is complete on one train and continues on the second train for the auxiliary feedwater/boric acid area coolers. Installation is planned for mid-June. Installation work continued in several areas.

ECN 5024 - Install Steam Generator Lay-Up Water System

Unit 1 insulation is complete.

ECNs 5111 and 5503 - Office and Power Stores Facility

Punchlist items for elevation 694 and the exterior are near completion. Elevation 694 is scheduled to be transferred to NUC PR on June 10.

Other Items (Cent.)

ECN 5119 - Install Radiation Monitor Cables in Conduit

Final tie-ins are underway.

ECN 5194 - Iodine Monitor

We are waiting for the Power Block reconfiguration to tie in the two doors into the security system.

ECN 5200 - Postaccident Sampling Facility

Rework of postmodification test deficiencies is in hold for the final design of one level loop.

ECN 5202 - Fifth Diesel Generator

Permanent power tie-in will be made in June. This project is complete with the exception of pouring two sections of missile protection over discharge piping and a splash pad for the overflow structure. Backfilling will be done after the completion of these two items.

ECN 5237 - Laundry Facility

Remaining work is on hold until the last wall is built.

ECN 5373 - Condensate Demineralizer Air Compressor

The vendor representative for startup testing is scheduled to be here June 3. This project is complete with the exception of the installation of the motor coupling and minor repairs.

ECN 5599 - Fifth Vital Battery

This project is complete with the exception of the remaining Appendix R work, which is continuing. The protective coating will also be applied.

ECNs 5609 and 5610 - Makeup Water Treatment Plant

This project is almost complete. Pre-op tests are essentially complete. The interface work is near completion as well. Most of the materials for remaining punchlist items have arrived. The septic tank and demineralizer modification work are being held up by OE.

ECN 5613 - Installation of Emergency Lights

Work has commenced to install emergency lights in the service building.

ECN 5645 - Steam Generator Blowdown

The sample line from the new system was rerouted through the existing heat exchanger to protect the radiation monitor.

Other Items (Cont.)

ECN 5657 - Installation of Isolation Valves, Moisture Separator Reheaters

Approximately 150 valves were installed on unit 1 during the ice outage, which completes unit 1 work. Repair of insulation remains incomplete.

ECN 5664 - Replace Relays in Wells Fargo Alarms

Remaining work has been restarted

ECN 5795 - Field Services Building

Fire detection system work is in hold for materials.

ECN 5841 - Hot Machine Shop

All work is complete with the exception the hallway on elevation 706. This delay is due to Power Stores and will be completed when they move to their new facility. We are still awaiting drawings on the monorail to be added to the decontamination room.

ECN 5878 - CDWE, Modification to Moisture Separator

Preoutage work continued on this project.

ECNs 5932 and 5935 - Power Block Modifications

This project is approximately 85 percent complete. Some cable pulling and equipment setting have been accomplished. We are still awaiting NRC approval of the security plan so the remaining work can be completed.

ECN 5938 - Feedwater Heater Replacement

Structural modifications for installation of monorails continued. The monorail electric hoist system was received. Two heaters have been hydrostatically tested. Reworking of several pipe supports continues.

ECN 5990 - CDWE, Install Divert Valve

Preoutage work continued on this project.

ECN 6057 - Cable Tray Covers

Approximately 240 out of 290 cable tray covers have been remanufactured or replaced.

ECN 6182 - Cooling Tower Repairs

Repair of ice damage that occurred in January 1985 will be done under a new ECN.

Other Items (Continued)

ECN 6202 - Component Cooling System Surge Tank Instrumentation

This work is scheduled to resume in early June.

ECN 6204 - Electrical Penetration Overcurrent Protection

Fuse replacement and fuse block installation are complete. We are waiting for a Technical Specification change to place the circuits in operation.

ECNs 6238 and 6356 - ERCW Pumping Station Piping Erosion

All piping has been replaced. Insulation reinstallation remains incomplete.

ECN 6342 - Health Physics Calibration Facility

This project is near completion. The installation of two doors, air conditioning lines, and a control box remains.

ECN 6351 - CDWE, Demineralized Water Supply

Workplan preparation is just starting.

ECN 6362 - CDWE, Install Sample Connections

Workplan preparation is just starting.

Training Buildings

The two structures have been constructed, and roofing work is being done at the present time.

Miscellaneous Yard Work

Work remaining includes the removal of overhead power lines and buildings, paving, and landscaping. This work is in progress.

Completed Modifications - Information Obtained from Printout

As we stated last month, in the future we plan to summarize the information obtained from the attached printout. We did not receive this printout until June 6; therefore, because of time constraints, we were unable to do this.

Sequoyah Nuclear Plant
P. O. Box 2000
Soddy-Daisy, Tennessee 37379

June 14, 1985

Nuclear Regulatory Commission Office of Management Information and Program Control Washington, DC 20555

Gentlemen:

SEQUOYAH NUCLEAR PLANT - MONTHLY OPERATING REPORT - MAY 1985

Enclosed is the May 1985 Monthly Operating Report to the NRC for Sequoyah Nuclear Plant.

Very truly yours,

TENNESSEE VALLEY AUTHORITY

O.R. Wallan

P. R. Wallace Plant Manager

Enclosure

cc (Enclosure):

Director, Region II Nuclear Regulatory Commission Office of Inspection and Enforcement 101 Marietta Street Suite 3100 Atlanta, GA 30323 (1 copy)

Director, Office of Inspection and Enforcement Nuclear Regulatory Commission Washington, DC 20555 (10 copies)

Mr. A. Rubio, Director Electric Power Research Institute P. O. Box 10412 Palo Alto, CA 94304 (1 copy)

Mr. R. C. Goodspeed MNC 461 Westinghouse Electric Corporation P. O. Box 355 Pittsburgh, PA 15230 (1 copy)

Mr. K. M. Jenison, NRC Inspector Sequoyah Nuclear Plant Director, Office of Management Information and Program Control Nuclear Regulatory Commission Washington, DC 20555 (2 copies)

INPO Records Center Suite 1500 1100 Circle 75 Parkway Atlanta, GA 30339 (1 copy)

1983-TVA 50TH ANNIVERSARY

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