

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2	PAGE (3) 1 OF 0 2
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TITLE (4)
ESF Actuation - Control Room Ventilation Isolation

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)		
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES		DOCKET NUMBER(S)
0 7	0 2	8 5	8 5	0 4	0	0 8	0 1	8 5			0 5 0 0 0

THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)

OPERATING MODE (9) 1	<input type="checkbox"/> 20.402(b)	<input checked="" type="checkbox"/> 20.406(c)	<input type="checkbox"/> 50.73(a)(2)(iv)	<input type="checkbox"/> 73.71(b)
	<input type="checkbox"/> 20.406(a)(1)(i)	<input type="checkbox"/> 50.36(c)(1)	<input type="checkbox"/> 50.73(a)(2)(v)	<input type="checkbox"/> 73.71(c)
	<input type="checkbox"/> 20.406(a)(1)(ii)	<input type="checkbox"/> 50.36(c)(2)	<input type="checkbox"/> 50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	<input type="checkbox"/> 20.406(a)(1)(iii)	<input type="checkbox"/> 50.73(a)(2)(i)	<input type="checkbox"/> 50.73(a)(2)(viii)(A)	
	<input type="checkbox"/> 20.406(a)(1)(iv)	<input type="checkbox"/> 50.73(a)(2)(ii)	<input type="checkbox"/> 50.73(a)(2)(viii)(B)	
	<input type="checkbox"/> 20.406(a)(1)(v)	<input type="checkbox"/> 50.73(a)(2)(iii)	<input type="checkbox"/> 50.73(a)(2)(ix)	

LICENSEE CONTACT FOR THIS LER (12)

NAME Merlin G. Williams - Superintendent of Regulatory, Quality and Administrative Services	TELEPHONE NUMBER	
	AREA CODE 3 1 1 6	3 1 6 4 - 1 8 1 3 1 1

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS
B	IIL	MIOIN	G 0 1 6 3	N					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On seven different occasions between July 2 and July 12, 1985, an Engineered Safety Features actuation was initiated by a Control Room intake radiation monitor spurious alarm causing a Control Room Ventilation Isolation. In each instance, all required Engineered Safety Features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to two of these events, and in Mode 1, Power Operation, with Reactor power levels ranging between 36 percent and 50 percent prior to the five other events. Each incident occurred with the Reactor Coolant System at normal operating pressure and temperature.

On each occasion, no radiation above normal background was present, as determined by a redundant radiation monitor, and at no time was the public health or safety threatened.

The cause of the radiation monitor spurious alarms has been attributed to a mismatch between the software and hardware in the microprocessing unit for the radiation monitor. A modification to the microprocessing unit has been installed to resolve the mismatch problem.

Previous actuations due to spurious alarms from this monitor were discussed in Licensee Event Reports (LER) 85-013-00 and 85-037-00. One additional actuation due to a spurious alarm occurred on July 12, 1985, and will be reported in a separate LER.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Wolf Creek Generating Station	DOCKET NUMBER (2) 0 5 0 0 0 4 8 2	LER NUMBER (6)			PAGE (3)	
		YEAR	SEQUENTIAL NUMBER	REVISION NUMBER		
		8 5	- 0 4 0	- 0 0	0 2	OF

TEXT (If more space is required, use additional NRC Form 366A's) (17)

On seven different occasions between July 2 and July 12, 1985, an Engineered Safety Features actuation was initiated by a spurious electronic "spike" in Control Room intake radiation monitor [IL-MON] GK-RE-04. The incidents occurred at 2002 CDT on July 2, 1506 CDT on July 5, 0550 CDT on July 7, 0215 CDT and 2205 CDT on July 8, 0932 CDT on July 9, and 0736 CDT on July 12. Each of these spikes resulted in a Control Room Ventilation Isolation Signal (CRVIS). In each instance, all required Engineered Safety Features equipment responded properly.

The plant was in Mode 3, Hot Standby, prior to the events on July 2 and July 12, 1985, and in Mode 1, Power Operation, with the Reactor power levels ranging between 36 percent and 50 percent, prior to the five events occurring between July 5 and July 9, 1985. Each event occurred with the Reactor Coolant System [AB] at normal operating pressure and temperature.

On each occasion, no radiation above normal background was present, as determined by redundant radiation monitor GK-RE-05, and the actuated systems were restored to a normal configuration per plant operating procedures.

Subsequent investigation of each event identified a mismatch between the software and hardware in the RM-80 microprocessing unit for the radiation monitor as the probable cause of the spurious alarms. Prior CRVIS actuations attributed to this mismatch were identified in Licensee Event Reports (LER) 85-013-00 and 85-037-00. One additional CRVIS due to the same cause occurred on July 12, 1985, and will be reported in a separate LER.

The radiation monitor was supplied by General Atomic Co. and is a Particulate, Iodine and Gas Monitor (Assembly 0356-1601). Several of these radiation monitors are installed in the plant and are now being modified in accordance with directions provided by General Atomic to resolve the software/hardware mismatch problems. This modification involves replacement of several integrated circuit chips and some software reprogramming in each radiation monitor. Modification of GK-RE-04 was completed on 7/13/85 and no further events due to the mismatch problem have occurred.

No damage to plant equipment occurred as a result of these events and at no time did conditions develop which could have threatened the public health or safety.



KANSAS GAS AND ELECTRIC COMPANY

GLENN L. KOESTER
VICE PRESIDENT - NUCLEAR

August 1, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Mr. R.P. Denise, Director
Wolf Creek Task Force
U.S. Nuclear Regulatory Commission
Region IV
611 Ryan Plaza Drive, Suite 1000
Arlington, Texas 76011

KMLNRC 85-188
Re: Docket No. STN 50-482
Subj: Licensee Event Report 85-040-00

Dear Gentlemen:

The enclosed Licensee Event Report is submitted pursuant to 10 CFR 50.73(a) (2) (iv) concerning an Engineered Safety Feature actuation.

If you have any questions concerning this matter, please contact me or Mr. Otto Maynard of my staff.

Yours very truly,

Glenn L. Koester
Vice President - Nuclear

GLK:dab

Enclosure

xc: PO'Connor (2), w/a
JCummins, w/a

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11