

Mr. H. B. Barron
Site Vice President
McGuire Nuclear Station
Duke Power Company
12700 Hagers Ferry Road
Huntersville, NC 28078

January 9, 1997

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING MCGUIRE NUCLEAR STATION, UNIT 2 FIRST 10-YEAR INTERVAL INSERVICE INSPECTION PROGRAM PLAN ASSOCIATED REQUEST FOR RELIEF NO. 96-003 (TAC NO. M95773)

Dear Mr. Barron:

The Nuclear Regulatory Commission (NRC) staff, with assistance from its contractor, Idaho National Engineering Laboratory (INEL), is reviewing and evaluating the first 10-year interval inservice inspection program plan associated Request for Relief No. 96-003 from the ASME Boiler and Pressure Vessel Code, Section XI, requirements for McGuire Nuclear Station, Unit 2. Additional information (see enclosure) is required from Duke Power Company in order for the staff to complete its review.

The staff would appreciate that a response be provided within 60 days to meet its inservice inspection program plan review schedule. In addition, to expedite the review process, please send a copy of your response to NRC's contractor, INEL, at the following address:

Michael T. Anderson
INEL Research Center
2151 North Boulevard
P.O. Box 1625
Idaho Falls, Idaho 83415-2209

If you have any questions, please contact me at (301) 415-1484.

Sincerely,
Original signed by:

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-370

Enclosure: As stated

cc w/encl: See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20555-0001

January 9, 1997

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Site Vice President
McGuire Nuclear Station
Duke Power Company
12700 Hagers Ferry Road
Huntersville, NC 28078

SUBJECT: REQUEST FOR ADDITIONAL INFORMATION REGARDING MCGUIRE NUCLEAR
STATION, UNIT 2 FIRST 10-YEAR INTERVAL INSERVICE INSPECTION PROGRAM
PLAN ASSOCIATED REQUEST FOR RELIEF NO. 96-003
(TAC NO. M95773)

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Idaho Falls, Idaho 83415-2209

If you have any questions, please contact me at (301) 415-1484.

Sincerely,

A handwritten signature in cursive script that reads "Victor Nerses".

Victor Nerses, Senior Project Manager
Project Directorate II-2
Division of Reactor Projects - I/II
Office of Nuclear Reactor Regulation

Docket No. 50-370

Enclosure: As stated

cc w/encl: See next page

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Duke Power Company

McGuire Nuclear Station

cc:

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Request for Additional Information
First 10-Year Interval Inservice Inspection (ISI)
Request for Relief 96-003
Duke Power Company
McGuire Nuclear Station, Unit 2
Docket No. 50-370

1. Scope/Status of Review

Throughout the service life of a water-cooled nuclear power facility, 10 CFR 50.55a(g)(4) requires that components (including supports) that are classified as American Society of Mechanical Engineers (ASME) Boiler and Pressure Vessel Code Class 1, Class 2, and Class 3 meet the requirements, except design and access provisions and preservice examination requirements, set forth in the ASME Code Section XI, *Rules for Inservice Inspection of Nuclear Power Plant Components*, to the extent practical within the limitations of design, geometry, and materials of construction of the components. This section of the regulations also requires that inservice examinations of components and system pressure tests conducted during a 120-month inspection interval comply with the requirements in the latest edition and addenda of the Code incorporated by reference in 10 CFR 50.55a(b) on the date 12 months prior to the start of the 120-month interval, subject to the limitations and modifications listed therein. The components (including supports) may meet requirements set forth in subsequent editions and addenda of the Code that are incorporated by reference in 10 CFR 50.55a(b) subject to the limitations and modifications listed therein and subject to Nuclear Regulatory Commission (NRC) approval.

The staff has reviewed the available information provided by the licensee in the June 6, 1996, submittal. The applicable edition of the Code for the McGuire Nuclear Station, Unit 2, first 10-year ISI interval is the 1980 Edition, through Winter 1980 Addenda of ASME Section XI.

2. Additional Information Required

Based on the above review, the staff has concluded that the following information and/or clarification is required to complete the review of Request for Relief 96-003.

- A. The Code of Federal Regulations (CFR) provides that a licensee may propose an alternative to CFR or Code requirements in accordance with 10 CFR 50.55a(a)(3)(i) or 10 CFR 50.55a(a)(3)(ii). Under 10 CFR 50.55a(a)(3)(i), the proposed alternative must be shown to provide an acceptable level of quality and safety, i.e., essentially be equivalent to the original requirement in terms of quality and safety. Under 10 CFR 50.55a(a)(3)(ii), the licensee must show that compliance with the original requirement results in a hardship or unusual difficulty without a compensating increase in the level of quality and safety. Examples of hardship and/or unusual difficulty include, but are not limited to, excessive radiation exposure, disassembly of components solely to provide access for examination, and development of sophisticated tooling that would result in only minimal increases in examination coverage.

Enclosure

A licensee may also submit a request for relief from ASME requirements. In accordance with 10 CFR 50.55a(g)(5)(iii), if a licensee determines that conformance with certain Code requirements is impractical for its facility, the licensee shall notify the Commission and submit, as specified in §50.4, information to support that determination. When a licensee determines that an inservice inspection requirement is impractical, e.g., component design or configuration limits the examination and the system would have to be redesigned or a component replaced to enable inspection, the licensee should cite 10 CFR 50.55a(g)(5)(iii). The NRC may, giving due consideration to the burden placed on the licensee, impose an alternative examination requirement.

Request for Relief 96-003 was submitted in accordance with 10 CFR 50.55a(a)(3)(ii) for welds that were examined to the "extent practical" due to component configuration. However, there is no discussion describing how compliance would result in a hardship or unusual difficulty. Review this request and provide the appropriate reference to the Code of Federal Regulations along with the documentation necessary to support the regulatory basis, as discussed above.

- B. In a letter dated February 14, 1995, the licensee stated that the augmented reactor pressure vessel (RPV) exams were performed in the last outage of the first interval and that for welds not meeting the >90% requirement an alternative would be submitted in accordance with 10 CFR 50.55a(g)(6)(ii)(A)(5). Furthermore, the licensee confirmed in a telecon that the relief requested in the June 6, 1996, submittal is intended to satisfy the ASME Section XI requirements for the first 10-year interval. However, relief from the Section XI requirements cannot be granted until the augmented requirement is satisfied. Provide the status of the augmented RPV examination with references to any applicable correspondence.

The schedule for timely completion of this review requires that the licensee provide, by the requested date, the above requested information and/or clarifications regarding the McGuire Nuclear Station, Unit 2, first 10-year ISI Request for Relief 96-003.