AVERAGE DAILY UNIT POWER LEVEL

Docket No. 50-272
Unit Name Salem # 1
Date July 10, 1985
Telephone 609-935-6000
Extension 4455

Completed by J. P. Ronafalvy

Month June 1985

Day	Average Daily Power Level (MWe-NET)	Day	Average Daily Power Level (MWe-NET)
1	1091	17	1066
2	1080	18	1092
3	1088	19	1086
4	1096	20	1092
5	1094	21	1091
6	1081	22	1079
7	1085	23	1087
8	1079	24	1093
9	1075	25	1089
10	1090	26	1087
11	1080	27	1089
12	1094	28	1077
13	1101	29	1071
14	1095	30	1070
15	1097	31	
16	1103		

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OPERATING DATA REPORT

Docket No. 50-272
Date July 10, 1985
Telephone 935-6000
Extension 4455

Completed by J. P. Ronafalvy

Operating Status

1.	Unit Name	Salem No. 1	Notes
2.	Reporting Period	June 1985	
3.	Licensed Thermal Power (MWt)	3338	
4.	Nameplate Rating (Gross MWe)	1170	
5.	Design Electrical Rating (Net MWe)	1090	
6.	Maximum Dependable Capacity (Gross MWe)	1124	
7.	Maximum Dependable Capacity (Net MWe)	1079	

8. If Changes Occur in Capacity Ratings (items 3 through 7) since Last Report, Give Reason N/A

9. Power Level to Which Restricted, if any (Net MWe) N/A

10. Reasons for Restrictions, if any N/A

		This Month	Year to Date	Cumulative
11.	Hours in Reporting Period	720	4343	70152
12.	No. of Hrs. Reactor was Critical	720	4325.6	40149.1
13.	Reactor Reserve Shutdown Hrs.	0	0	3088.4
14.	Hours Generator On-Line	720	4322.7	38481.3
15.	Unit Reserve Shutdown Hours	0	0	0
16.	Gross Thermal Energy Generated (MWH)	2405532	14366391	117134388
17.	Gross Elec. Energy Generated (MWH)	814080	4907320	38823370
18.	Net Elec. Energy Generated (MWH)	782311	4717899	36815881
19.	Unit Service Factor	100	99.5	54.9
20.	Unit Availability Factor	100	99.5	54.9
21.	Unit Capacity Factor (using MDC Net)	100.7	100.7	48.6
22.	Unit Capacity Factor (using DER Net)	99.7	99.7	48.1
23.	Unit Forced Outage Rate	0	.5	30.6

24. Shutdowns scheduled over next 6 months (type, date and duration of each)

N/A

25. If shutdown at end of Report Period, Estimated Date of Startup:
N/A

26. Units in Test Status (Prior to Commercial Operation):

	Forecast	Achieved
Initial Criticality	9/30/76	12/11/76
Initial Electricity	11/1/76	12/25/76
Commercial Operation	12/20/76	6/30/77

UNIT SHUTDOWN AND POWER REDUCTIONS
REPORT MONTH June 1985

Docket No. 50-272
Unit Name Salem No. 1
Date July 10, 1985
Telephone 609-935-6000
Extension 4455

Completed by J.P. Ronafalvy

No.	Date	Type 1	Duration Hours	Reason 2	Method of Shutting Down Reactor	License Event Report	System Code 4	Component Code 5	Cause and Corrective Action to Prevent Recurrence

* There were no outages for the month of June resulting in a load reduction of greater than 20%.

1
F: Forced
S: Scheduled

2 Reason
A-Equipment Failure-explain
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & Licensing Exam
F-Administrative
G-Operational Error-explain
H-Other-explain

3 Method
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuation of Previous Outage
5-Load Reduction
9-Other

4 Exhibit G Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

5 Exhibit 1 Salem as Source

MAJOR PLANT MODIFICATIONS
REPORT MONTH June 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: July 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1FC-0643	Annunciator System	Revise all call broadcast acknowledge answer back system to be compatible with GE/TAC cabinet.
1EC-1324	Circulating Water	Install permanent discharge water sample system for continuous sampling.
1EC-1447	Fresh Water	Connect one end of the former concrete plant 4" water supply line into the station potable water system and connect the other end at #1 Well House to lines feeding 901, 902 and other buildings.
1EC-1483	Meteorological Tower Control House	Addition of a primary and back-up transmitter, antenna and controller in the Meterological Tower Control House.
1EC-1531	Waste Gas	Install redundant waste gas oxygen analysis capability.
1EC-1731	Drains/Sump Pumps	Replace No. 13 Turbine Area Sump Pump with a submersible type slurry pump of different design and materials.
1EC-1852	Moisture Separator Reheater Steam and Drains	Modify the MSR shell side internals. Design change includes the separation, distribution and drain systems.
1EC-1892	Circulating Water	Replace circulating water pump vacuum breakers valves 11CW15, 12CW15, 13CW15, 11CW115, 12CW115, 13CW115.

*DCR NO.	PRINCIPLE SYSTEM	SUBJECT
1EC-1965	Structural-#1 North Penetration	Replace shield blocks in shielding wall 15' west of GG line.
1EC-1972	500KV Lines	Rearrangement of 500KV lines in Salem switchyard to provide relay modification necessary to reroute Salem-New Freedom line to Salem-Deans line.
1SC-1237	Generator Exciter	Modify exciter enclosure roof to provide direct access to Monitor No. 11 bearing vibration.
1SC-1317	Service Building El. 127'-Structural	Removal of 7' section of non-load bearing wall between Operations Staff Office and room previously used as Station Bunk Room. Removal and blocking up existing door to Operations Staff Office.
1SC-1428	Existing Walls	Install hangers for herculite rolls and poly bags at locations determined by the Solid Radwaste Supervisor.
1SC-1586	Switchyard Miscellaneous Power	Replace transformer that failed with new style. Original transformer had PCB's (# changeout will require installation of a temporary transformer).

MAJOR PLANT MODIFICATIONS
REPORT MONTH JUNE 1985

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: July 10, 1985
COMPLETED BY: J. Ronafalvy
TELEPHONE: 609/339-4455

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-0643	This DCR is not Safety Related and does not affect any safety related equipment. No unreviewed safety or environmental questions are involved.
1EC-1324	The installation of a concrete foundation pad and a meter shed will neither affect any previously performed safety analyses nor affect the safe operation of the plant. No unreviewed safety or environmental questions are involved.
1EC-1447	The installation of piping to bypass production well No. 1 does not affect any previously performed safety analyses nor affect the safe operation of the plant. No unreviewed safety or environmental questions are involved.
1EC-1483	No safety function or safety margin will be affected by this DCR. No unreviewed safety or environmental questions are involved.
1EC-1531	No function of any system or component required to safely shutdown the Unit is affected. Also, the ability to keep the Unit in a shutdown condition is not affected. No unreviewed safety or environmental questions are involved.
1EC-1731	This DCR replaces the #13 Turbine Generator Area Sump Pump from a 5 horse power pump to a 10 horse power pump. New cable will be routed for the change. The associated motor control center is revised to accept the new motor. The system function will not change. No unreveiwed safety or environmental questions are involved.
1EC-1852	This DCR provides for the modification of the internals of the MSR's to improve their performance and reliability. No unreviewed safety or environmental questions are involved.
1EC-1892	The replacement of the vacuum breaker valves (CW115S) does not affect any previously performed safety analyses nor does it create any new hazards. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1EC-1965	This DCR involves minor modifications for repair of the masonry block wall radiation shield in the North Penetration Area at elevation 100' in line with the Containment Personnel Lock. This wall was notched to allow passage of a replacement coil for one of the Containment Fan Coil Units. The design modification was analyzed by the Civil Design Div. to insure that the wall as revised will maintain integrity in the event of a seismic occurrence. No unreviewed safety or environmental questions are involved.
1EC-1972	This DCR replaces 500KV relaying. No safety related components are affected. The same number of offsite power sources as are currently present will be available after the change. Seismic considerations for the new relay racks and the new relays are considered acceptable since the mounting configurations, rack structural members, and all mounting hardware are identical to the equipment which is being removed and has been evaluated. No work is being done which affects interfaces, either directly or through isolation devices, with any safety related function or component. Therefore, no failure mode exists which could affect the safe shutdown of the Reactor Plant. The Tech. Specs. are not affected by this DCR. Sections 8.1 and 8.2 of the FSAR are affected by this DCR. The FSAR currently states that two lines from Salem switchyard go to the New Freedom switchyard. The sections will be changed to indicate that one line runs between Salem and New Freedom switchyards and one line runs between Salem and Deans switchyard. No unreviewed safety or environmental questions are involved.
1SC-1237	This modification will not alter any plant discharge or process and will not affect the existing plant impact. This change has no effect on any safety related system. No unreviewed safety or environmental questions are involved.
1SC-1317	The work to this DCR will all be done in the Service Building (a non-safety related structure). No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

*DCR NO.	SAFETY EVALUATION 10 CFR 50.59
1SC-1428	Work to this DCR does not impact the safe shutdown or isolation of the reactor, nor is the margin of safety decreased. No unreviewed safety or environmental questions are involved.
1SC-1586	This DCR replaces a miscellaneous power transformer in the 500KV switchyard. This is a non-vital application and no safety related systems are affected. There is no change in the design basis for the FSAR or the Tech. Specs. Testing will be used to confirm that the transformer as non-PCB. No unreviewed safety or environmental questions are involved.

*DCR - Design Change Request

PSE&G SALEM GENERATING STATION
SAFETY RELATED WORK ORDER LOG

SALEM UNIT 1

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8506260205	SIC	1	IC D/G	
			FAILURE DESCRIPTION:	WHEN INCREASING LOAD, ACTUAL LOAD SPIKES DOWNWARD. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION:	TROUBLE SHOT IC D/G, FOUND E.G.A. GOV. CONT. BOX WAS NOT OPER. PROP. REPLACED BOX. SURVEILLANCE TEST - SAT.
8506200458	SMD	1	1CV75	
			FAILURE DESCRIPTION:	1CV75 STARTING TO LEAK AGAIN. PLEASE RE-INJECT WITH FURMANITE.
			CORRECTIVE ACTION:	FURMANITE MADE TEMP. REPAIRS BY INJECTING FURMANITE COMPOUND FURM. JOB N/L 9830
8506200032	SMD	1	PZR HEATER DEION	
			FAILURE DESCRIPTION:	THE DEION IS TRIPPED. PLEASE DETERMINE THE CAUSE FOR THE TRIPPED CONDITION AND CORRECT.
			CORRECTIVE ACTION:	REPLACED 1EP12X BREAKER WITH A NEW BREAKER
8506200041	SMD	1	PZR HEATER DEION	
			FAILURE DESCRIPTION:	THE DEION IS TRIPPED. PLEASE DETERMINE THE REASON FOR THE TRIPPED CONDITION AND CORRECT.
			CORRECTIVE ACTION:	REPLACED 1EP7X BREAKER WITH A NEW BREAKER

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8506181178				
	SMD	1	1EP PZR. HTR. BUS	
			FAILURE DESCRIPTION:	1EP10X, 9X, 4X, AND 13X DEIONS KEEP TRIPPING. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION:	REPLACED THE ABOVE MENTIONED BREAKERS WITH NEW BREAKERS
8506171814				
	SIC	1	STM FLOW/FEED FLOW REC	
			FAILURE DESCRIPTION:	THERE IS APPROX. A 10% DISCREPANCY BETWEEN THE FEED FLOW AND STEAM FLOW ON THE RECORDER. THE BEZEL METERS ARE APPROX. THE SAME. PLEASE CORRECT.
			CORRECTIVE ACTION:	CHECK CAL OF STEAM FLOW AND FEED FLOW AS PER ICD CARD. REPLACED SERVO-AMP V155H6 WITH V14DH6.
8506140056				
	SMD	1	12 FHB EXH FAN	
			FAILURE DESCRIPTION:	THE FAN TRIPPED OUT. LOCALLY THE FAN IS SAT. PLEASE CHECK THE BREAKER.
			CORRECTIVE ACTION:	REPLACED WORN AND BROKEN FAN BELTS ON MOTOR
8506131456				
	SMD	1	12 SPENT FUEL PUMP	
			FAILURE DESCRIPTION:	REPAIR LEAK ON DISCHARGE FLANGE OF 12 SPENT FUEL COOLING PUMP
			CORRECTIVE ACTION:	REPLACED FLEX GASKET IN DISCHARGE FLANGE WITH GRAPHIOL GASKET

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8506131464				
	SMD	1	11 SPENT FUEL PUMP	
			FAILURE DESCRIPTION:	REPAIR LEAK ON THE DISCHARGE FLANGE OF 11 SPENT FUEL COOLING PUMP.
			CORRECTIVE ACTION:	REPLACED FLEX GASKET WITH GRAFOIL GASKET IN DISCHARGE FLANGE
8506131472				
	SMD	1	SPENT FUEL COOLING	
			FAILURE DESCRIPTION:	REPAIR 2 FLANGE LEAKS IN FRONT OF SPENT FUEL HEAT EXCHANGER IN THE VICINITY OF 1SF900 & 1SJ901 VALVES
			CORRECTIVE ACTION:	REPAIRED LEAK IN 2 FLANGES BY REPLACING FLEX GASKETS WITH GRAFOIL GASKET
8506071178				
	SOD	1	COMPONENT COOLING	
			FAILURE DESCRIPTION:	REVISE THE COMPONENT COOLING FLOW RATE TO THE RHR HEAT EXCHANGERS DURING COLD LEG RECIRCULATION FROM 2000 GPM TO 4000 GPM PER HEAT EXCHANGER.
			CORRECTIVE ACTION:	OTSC MADE TO II 7.3.2 TABLE II 7.3.2 TABLE AND III TO REFLECT 4000 GPM SETPOINT CHANGE. OTSC MADE TO SP(O)4.0.5.P-CC 11,12,13 STEP 5.7 & 5.8.1 TO VERIFY CC15 FLOW CONTROL VALVE IS SET FOR 4000 GPM

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION
8506080037	SIC	1	RMS CH 1 R41B
			FAILURE DESCRIPTION: THE CHANNEL IS FAILING INTERMITTENTLY. PLEASE INVESTIGATE.
			CORRECTIVE ACTION: CLEARED RAMS AND RESEATED MODULE AT LOCAL MONITOR. VERIFIED SETPOINTS AND REPLACED 1PR1 METER FACE GLASS.
8506060010	SMD	1	B/U PZR HTRS 27,28,55
			FAILURE DESCRIPTION: THIS BREAKER PERIODICALLY TRIPS. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION: TESTED NEW BREAKER AND INSTALLED IN 1EP 5X POS.
8506060028	SMD	1	B/U PZR HTRS 12,13,&14
			FAILURE DESCRIPTION: THIS BREAKER TRIPS PERIODICALLY. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION: TESTED NEW BREAKER AND INSTALLED 1EP 1X POS.
8505110366	SIC	1	1R41B PLANT VENT IODINE
			FAILURE DESCRIPTION: 1R41B, PLANT VENT IODINE RADIATION MONITOR IS CONSTANTLY FAILING. THE MONITOR IS READING E11, AND THE METER IS SPIKING. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION: REMOVED LOCAL SCALER S/N 168 AND CLEANED J1 AND J2 PINS WITH D FLUX CLEANING SOLUENT. REPLACED R2 FUSE XMIT LOOP 1 IN REMOTE 862C1 CONTROLLER. REPLACED REMOTE POWER SUPPLY S/N 459 WITH S/N 474. MODULES REINSTALLED.

WO NO	DEPT	UNIT	EQUIPMENT IDENTIFICATION	
8505141318				
	SMD	1	12SW23, S.W. NUC. HDR CROSSTIE	
			FAILURE DESCRIPTION:	12SW23, THE NUCLEAR HEADER CROSS-TIE VALVE WILL NOT CLOSE ALL THE WAY. VALVE STROKE STOPS APPROX 2" FROM LIMIT. THE BREAKER DOES NOT TRIP OPEN HOWEVER. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION:	RUST ON FACE OF CONTACTS OF TORQUE SWITCH. REMOVED RUST AND STROKED VALVE SAT.
8504161226				
	SMD	1	13 CHARGING PUMP	
			FAILURE DESCRIPTION:	THERE IS INDICATION OF PISTON SEAL LEAKAGE. PLEASE INVESTIGATE AND REPAIR.
			CORRECTIVE ACTION:	REPACKED 5 CYLINDERS, REPLACED 3 PLUNGERS, REPLACED 5 STUFFING BOXES.
0099164230				
	SIC	1	1C DIESEL GEN	
			FAILURE DESCRIPTION:	PLEASE REPLACE THIS RELAY DUE TO INOPERABILITY OF OLD RELAY.
			CORRECTIVE ACTION:	FOUND INPUT SHORTENED ON 2 STG. INSTALLED SPARE. CHECKED CALIBRATION.

SALEM GENERATING STATION
MONTHLY OPERATING SUMMARY - UNIT NO. 1
JUNE 1985

SALEM NO. 1

The Unit operated at full power for the entire period.

REFUELING INFORMATION

COMPLETED BY: J. Ronafalvy

DOCKET NO.: 50-272
UNIT NAME: Salem 1
DATE: July 10, 1985
TELEPHONE: 609/935-6000
EXTENSION: 4455

Month June 1985

1. Refueling information has changed from last month:
   YES _____ NO X

2. Scheduled date for next refueling: February 22, 1986

3. Scheduled date for restart following refueling: May 4,1986

4. A) Will Technical Specification changes or other license amendments be required?
   YES _____ NO _____
   NOT DETERMINED TO DATE _____

   B) Has the reload fuel design been reviewed by the Station Operating Review Committee?
   YES _____ NO X
   If no, when is it scheduled? January 1986

5. Scheduled date(s) for submitting proposed licensing action:
   January 1986 if required

6. Important licensing considerations associated with refueling:
   NONE

7. Number of Fuel Assemblies:
   A) Incore 193
   B) In Spent Fuel Storage 296

8. Present licensed spent fuel storage capacity: 1170

   Future spent fuel storage capacity: 1170

9. Date of last refueling that can be discharged to spent fuel pool assuming the present licensed capacity: September 2001

8-1-7.R4

PSE&G

Public Service Electric and Gas Company P.O. Box E Hancocks Bridge, New Jersey 08038

Salem Generating Station

July 10, 1985

Director, Office of Inspection and Enforcement
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Dear Sir:

MONTHLY OPERATING REPORT
SALEM NO. 1
DOCKET NO. 50-272

In Compliance with Section 6.9, Reporting Requirements for the Salem Technical Specifications, 10 copies of the following monthly operating reports for the month of June 1985 are being sent to you.

Average Daily Unit Power Level
Operating Data Report
Unit Shutdowns and Power Reductions
Major Plant Modification
Safety Related Work Orders
Operating Summary
Refueling Information

Sincerely yours,

J. M. Zupko, Jr.
General Manager - Salem Operations

JR:sbh
cc: Dr. Thomas E. Murley
Regional Administrator USNRC
Region I
631 Park Avenue
King of Prussia, PA 19406

Director, Office of Management
Information and Program Control
U.S. Nuclear Regulatory Commission
Washington, DC 20555

Enclosures
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