LASALLE NUCLEAR POWER STATION

UNIT 1

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-373

LICENSE NO. NPF-11

IE24
1/1

I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Comapny and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architect-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edision Company.

Unit one was issued operating license number NPF-11 on April 17, 1982. Initial criticality was achieved on June 21, 1982, and commercial power operation was commenced on January 1, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

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II. MONTHLY REPORT FOR UNIT ONE

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT ONE

July 1-26

July 1, 0001 Hours	Reactor power at 6.3%.
July 1, 0230 Hours	Generator Synchronized to Grid
July 1, 0700 Hours	Reactor Power at 34%.
July 2, 1500 Hours	Reactor Power at 66%.
July 3, 2300 Hours	Reactor Power at 86%.
July 6, 0830 Hours	Suppression pool spray inoperable, commence 7 day timeclock.
July 11, 2300 Hours	Reactor Power at 47%.
July 12, 0615 Hours	Reactor manually scrammed. The reactor was critical for 270 hours and 15 minutes.

JULY 27-31

July 27, 1930 Hours	Reactor Critical
July 28, 1215 Hours	Generator Synchronized to Grid.
July 28, 2300 Hours	Reactor Power at 45%.
July 29, 0700 Hours	Reactor Power at 72%.
July 31, 2300 Hours	Reactor Power at 96%. The reactor was critical for 100 hours and 30 minutes. Totaling 370 hours and 45 minutes for the month of July.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specification.

   There were no amendments to the facility license or Technical Specifications during this reporting period.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval during this reporting period.

3. Tests and Experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during this reporting period.

4. Corrective maintenance of safety related equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit One during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, Cause of Malfunction, Results and Effects on Safe Operation, and Corrective Action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L35149	RHR Shutdown Cooling Discharge Valve, 1E12-F053A.	Eroded seat and disc.	Vave leaked in excess of allowable limits.	Lapped seat and ground disc.
L49899	Hathaway Sequence-of-Events recorder.	Faulty series switch 1E21-N008A.	MSIV 1/2 Isolation on erroneous signal.	Jumpered switch out.
L49994	Outboard MSIV "A".	Limit switch in improper position.	Dropped out RPS K3B relay and would not reset.	Corrected Limit Switch Position.
L50046	1A RHR Heat Exchanger.	Fouled tubes.	Could not obtain desired service water flow.	Cleaned tubes.
L50066	Accident Monitoring Wide Range Level Recorder.	Recorder Out-of-Calibration	Incorrect indication.	Recalibrated.
L50100	RHR Suppression Pool Spray Valve, 1E12-F027B.	Faulty torque switch.	Tripped Thermal Overloads while closing.	Cleaned and adjusted torque switch.
L50121	1A Drywell Pneumatic Compressor.	Bent tubing allowed control air to leak.	Compressor would not load.	Installed new tubing.
L50122	Division I Post-LOCA Oxygen Monitor.	Faulty reagent flow regulator.	Indicated low.	Replaced reagent flow regulator.
L50130	RHR Suppression Pool Spray Valve, 1E12-F027B.	Worn seat and disc.	Excessive leakage through valve B RHR could not be maintained full in standby.	Lapped Valve seat and disc.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L50134	HCU Accumulator for CRD 02-31.	Stem bent on instrument block stop valve.	Leaked nitrogen; potential to cause failure to scram this rod if combined with other events.	Replaced Valve.
L50196	Ammonia Detector.	Optics wire broken during surveillance.	Inoperable detector.	Repaired wire.
L50251	Control Room Ventilation Air Condition-Compressor "B".	Blown Oil seal	Degraded ventilation from "B" Control Room HVAC.	Replaced seal.
L50416	1B Diesel Generator	Loose bolts on turbocharger	Potential to cause degraded Diesel performance	Torqued bolts.
L50460	1B Diesel Generator K9 Relay.	Bent Plunger Striker on Switchgear 143 cubicle 001.	Prevented proper over-current trip function of ACB 1432.	Installed new plunger striker.
L50481	Standby Liquid Control Pump 1A Discharge Relief Valve.	Would not open at desired pressure.	Potential to cause failure of Standby Liquid Control Piping.	Changed relief setting.
L50525	HCU Accumulator for CRD 34-51.	III Valve leaked Nitrogen.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50643	HCU for CRD 30-35, and HCU for CRD 46-07.	Instrument Block Valves leaked nitrogen to atmosphere.	Potential to cause failure to scram these rods if combined with other events.	Replaced valves.
L40644	HCU for CRD 54-39.	Instrument block valve leaked nitrogen to atmosphere.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L41909	Reactor Recirculation Discharge Valve, 2B33-FO67A.	Degraded Packing	Packing leak.	Repacked Valve.
L50069	1A RHR Service Water Strainer.	Assembled incorrectly following maintenance.	Strainer leaked considerably; potentially degraded RHR "A" performance.	Reassembled strainer.
L50513	1A RHR Heat Exchanger	Tubes fouled.	Could not obtain required Service Water flow.	Cleaned tubes.
L50336	Safety Relief Valve C.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
LS0337	Safety Relief Valve D.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50338	Safety Relief Valve E.	Broken set screw on nozzle ring.	No significant effect.	Replaced with imrpoved set screw.
L50339	Safety Relief Valve F.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50342	Safety Relief Valve J.	Broken set screw on nozzle ring.	No significant effect.	Replaced with imrpoved set screw.
L50344	Safety Relief Valve L.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50345	Safety Relief Valve M.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L50347	Safety Relief Valve P.	Broken set screw on nozzle ring.	No significant effect.	Replaced with imrpoved set screw.
L50348	Safety Relief Valve R.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.
L50350	Safety Relief Valve U.	Broken set screw on nozzle ring.	No significant effect.	Replaced with improved set screw.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit One, logged during the reporting period, July 1 through July 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-048-00	6-14-85	"A" RHR WS PRM INOP
85-049-00	6-25-85	Chlorine Detector Trip
85-050-00	6-26-85	"A" VC/VE Ammonia/Chlorine Detector Alarms
85-051-00	6-27-85	Spurious Chlorine Detector Trip.
85-052-00	6-29-85	Manual Reactor Scram
85-053-00	7-17-85	RHR Shutdown Cooling Suction High Flow Isolation Switches Installed Backwards.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report
2. Average Daily Unit Power Level
3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-373
UNIT LaSalle One
DATE August 10, 1985
COMPLETED BY Richard J. Rohrer
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: JULY, 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY):N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	370.8	3879.5	10161
6.	REACTOR RESERVE SHUTDOWN HOURS	373.3	476.2	1642
7.	HOURS GENERATOR ON LINE	351.5	3726.0	9783
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	847778	10290841	27114130
10.	GROSS ELEC. ENERGY GENERATED (MWH)	266164	3375933	8846576
11.	NET ELEC. ENERGY GENERATED (MWH)	249325	3244270	8439332
12.	REACTOR SERVICE FACTOR	49.8%	75.9%	73.1%
13.	REACTOR AVAILABILITY FACTOR	100%	85.2%	84.9%
14.	UNIT SERVICE FACTOR	47.2%	72.9%	70.4%
15.	UNIT AVAILABILITY FACTOR	47.2%	72.9%	70.4%
16.	UNIT CAPACITY FACTOR (USING MDC)	32.3%	61.3%	58.6%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	31.1%	58.9%	56.3%
18.	UNIT FORCED OUTAGE RATE	52.6	23.6%	19.9%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH)
Unit one is scheduled for a refueling, maintenance, modification, and surveillance outage beginning September 3, 1985 and lasting 26 weeks.
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP: NA

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-373
UNIT: LASALLE ONE
DATE: August 10, 1985
COMPLETED BY: Richard J. Rohrer
TELEPHONE: (815) 357-6761

MONTH: JULY, 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	170	17.	-18
2.	439	18.	-17
3	814	19.	-17
4.	749	20.	-18
5.	884	21.	-18
6.	871	22.	-21
7.	785	23.	-17
8.	878	24.	-15
9.	877	25.	-16
10.	879	26.	-16
11.	821	27.	-16
12.	51	28.	90
13.	-18	29.	695
14.	-17	30.	724
15.	-17	31.	923
16.	-19		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-373
UNIT NAME LaSalle One
DATE JULY 10, 1985
COMPLETED BY Richard J. Rohrer
TELEPHONE (815)357-6761

REPORT MONTH JULY 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
14	850629	F	2.5	A	4	Continuation of outage from previous month.
15	850712	F	390.0	A	2	Unit shutdown due to inoperable suppression pool spray valve 1E12-F027B.

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E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief valve operations for Unit One.

DATE	VALVES ACTUATED	NO & TYPE ACTUATION	PLANT CONDITION	DESCRIPTION OF EVENT

There were no Safety Relief Valves Operated for Unit One during this reporting period.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-552-85	1C RHR Pump	Lubrication
1-554-85	1A RHR Heat Exchanger	Clean Tubes
1-556-85	1E12-F027B	Repair Valve
1-557-85	1E12-F027B	Repair Valve
1-558-85	1E12-F027B	Adjust Limit Switches
1-565-85	1E12-F063A	Keep RHR pressurized
1-568-85	1B RHR Service Water Pump	Repair Pump
1-572-85	1B D/G	Lubrication
1-573-85	Shutdown Coolirg Suction Header	PreventInitiation with vents open.
1-574-85	1E12-F027B	Maintain Primary Containment.
1-576-85	1B D/G	Calibration
1-577-85	1E12-F027B	Maintain Primary Containment
1-578-85	1E12-F027B	Remove Actuator
1-579-85	1E12-F027B	Remove Actuator
1-585-85	1E12-F053A	Repair Valve
1-586-85	1E12-F053A	Repair Actuator
1-587-85	1E12-F004A	Prevent Operation
1-588-85	1E12-F027B	Repair Valve.
1-594-85	1B RHR Pump	Oil Sample
1-609-85	1E12-F004A	Repair Torque Switch

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
1-611-85	1E12-F023	Prevent operation
1-626-85	1E12-N012AA	Repipe Instrument
1-630-85	1E12-F004A	Repair Limitorque
1-636-85	1A RHR Heat Exchanger	Clean Tubes
1-638-85	1E12-D300A	Inspect and Clean
1-642-85	1E12-F023	Verify wiring on valve
1-652-85	1E12-F008	Perform LIS-NB-311
1-653-85	1E12-D300A	Repair Leaks
1-655-85	RHR Shutdown Cooling Valves	Vent path for LST-85-45
1-658-85	1E12-F336A	Replace Retainer Ring

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation Manual during this reporting period.

4. Radioactive Waste Treatment Systems.

There were no significant changes to the radioactive waste treatment system during this reporting period.

LASALLE NUCLEAR POWER STATION

UNIT 2

MONTHLY PERFORMANCE REPORT

JULY 1985

COMMONWEALTH EDISON COMPANY

NRC DOCKET NO. 050-374

LICENSE NO. NPF-18

DOCUMENT ID 0036r/0005r

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I. INTRODUCTION

The LaSalle County Nuclear Power Station is a two-unit facility owned by Commonwealth Edison Company and located near Marseilles, Illinois. Each unit is a Boiling Water Reactor with a designed net electrical output of 1078 Megawatts. Waste heat is rejected to a man-made cooling pond using the Illinois River for make-up and blowdown. The architecht-engineer was Sargent and Lundy, and the primary construction contractor was Commonwealth Edison Company.

Unit two was issued operating license number NPF-18 on December 16, 1983. Initial criticality was achieved on March 10, 1984, and commercial power operation was commenced on June 19, 1984.

This report was compiled by Richard J. Rohrer, telephone number (815)357-6761 extension 575.

II. MONTHLY REPORT FOR UNIT TWO

A. SUMMARY OF OPERATING EXPERIENCE FOR UNIT TWO

JULY 1-19

July 1, 0001 Hours	Reactor subcritical. Unit two still in scheduled outage.

JULY 20-31

July 20, 2125 Hours	Reactor Critical
July 22, 0530 Hours	Generator Synchronized to Grid
July 22, 0630 Hours	Main Turbine Trip Due to High Level in MSR Drain Tank.
July 22, 0800 Hours	Generator Synchronized to grid.
July 23, 0300 Hours	Removed Main Turbine From Grid for RCIC Surveillance
July 23, 0420 Hours	Generator Synchronized to Grid
July 24, 0700 Hours	Reactor Power at 34%
July 25, 1500 Hours	Reactor Power at 64%
July 26, 1500 Hours	Reactor Power at 82%.
July 27, 0700 Hours	Reactor Power reduced to 67% for Rod Shuffle
July 27, 1500 Hours	Reactor Power at 86%.
July 31, 0700 Hours	Reactor Power at 96%.
July 31, 1500 Hours	Reactor Power at 78%.
July 31, 2300 Hours	Reactor Power Reduced to 23%. Bringing Unit Down to Investigate High Drywell Temperatures. The Reactor was Critical for 266 Hours and 35 Minutes During the Month of July.

B. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS AND SAFETY RELATED MAINTENANCE.

1. Amendments to facility license or Technical Specifications.

   Ther were no Amendments to the facility License or Technical Specifications for this reporting Month.

2. Facility or procedure changes requiring NRC approval.

   There were no facility or procedure changes requiring NRC approval during the reporting period.

3. Tests and experiments requiring NRC approval.

   There were no tests or experiments requiring NRC approval during the reporting period.

4. Corrective Maintenance of Safety Related Equipment.

   The following table (Table 1) presents a summary of safety-related maintenance completed on Unit Two during the reporting period. The headings indicated in this summary include: Work Request number, Component Name, cause of malfunction, results and effects on safe operation, and corrective action.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L43822	HCU for CRD 02-39	Scram pilot valve had an air leak.	Potential to cause half-scram for this rod.	Replaced O-ring seals in valve.
L42825	HCU for CRD 02-39	Scram pilot valve had an air leak.	Potential to cause half-scram for this rod.	Replaced O-ring seals in vlave.
L46594	HCU for CRD 10-31.	Scram pilot valve had an air leak.	Potential to cause half-scram for this rod.	Rebuilt valve.
L46655	RCIC Outboard Steam Isolation Valve.	Degraded valve packing.	Significant steam leak through packing.	Repacked valve.
L47320	Outboard Feedwater Check Valve "B".	Actuating Cylinder leaked air.	Degraded Valve Operation.	Rebuilt actuating cylinder.
L47356	Outboard Feedwater Check Valve "A".	Both actuating cylinders leaked air.	Degraded Valve Operation.	Rebuilt Actuating cylinders
L47518	Drywell Pneumatic Dryer Purge Valve.	Worn Valve seat.	Leakage in excess of desired amount.	Lapped seat.
L47529	Floor Drain Inboard Isolation Valve.	Crud obstructing valve motion.	Valve leaked in excess of allowable amount.	Cleaned valve.
L47591	Safety Relief Valve "E".	Damaged Valve Seat and disc.	Valve leaked steam by.	Replaced valve.
L47592	Safety Relief Valve "R".	Damaged Valve seat and disc.	Valve leaked steam by.	Replaced valve.
L47631	RCIC Turbine Exhaust Isolation valve, 2E51-F068.	Limit Switches out of Adjustment.	Valve would not fully close exept manually.	Repositioned limit switches.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L47692	HCU for CRD 18-07.	Scram, pilot valve had a severe air leak.	Potential to cause a half-scram of this control rod.	Rebuilt valve.
L47763	HCU for CRD 06-27.	Scram pilot valve leaked air.	Potential to cause a half-scram of this control rod.	Rebuilt valve.
L48300	Hydrogen Recombiner exhaust upstream isolation valve, 2HG006A.	Worn valve disc.	Failed local leak rate test.	Replaced Valve disc.
L48693	Reactor Pressure interlock switch, 2B21-N039L.	No instrument rack stop valve.	Could not isolate instrument.	Installed new stop valve.
L49227	Hydrogen Recombiner exhaust downstream isolation valve, 2HG005A.	Worn Valve disc. and seat.	Valve failed local leak rate test.	Replaced disc and lapped seat.
L49768	RHR Shutdown Cooling High Flow isolation switches, 2E12-N012AA and AB.	Switches were piped backwards.	Switches were inoperable.	Piping corrected.
L49788	HCU for CRD 06-35.	Leaking drain valve on accumulator.	Potential to cause failure to scram this rod if combined with other events.	Rebuilt Valve.
L49843	Feedwater Check Valve 2B21-F032A.	Incorrect solenoids installed.	Air leaked on actuator; Possibly possibly degraded valve operation.	Installed correct solenoids.

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TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49856	Scram Reset Switch.	Switch stuck in reset position.	Groups 1 and 4 would automaticaly reset if scram signal cleared.	Installed new switch operator.
L49858	Safety Relief Valve A.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49860	Safety Relief Valve C.	Broken Set Screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49861	Safety Relief Valve D.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49863	Safety Relief Valve F.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49864	Safety Relief Valve G.	Broken set screw in nozzle ring.	No significant effect. set screw.	Replaced with improved
L49865	Safety Relief Valve H.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49868	Safety Relief Valve L.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49869	Safety Relief Valve M.	Broken set screw in nozzle ring.	No significant effect.	Replaced with inproved set screw.
L49871	Safety Relief Valve P.	Broken set screw in nozzle ring.	No significant effect.	Replaced with improved set screw.
L49875	Safety Relief Valve V.	Broken set screw in nozzle ring.	No significant effect.	Replaced with inproved set screw.

TABLE 1

CORRECTIVE MAINTENANCE OF
SAFETY RELATED EQUIPMENT

WORK REQUEST	COMPONENT	CAUSE OF MALFUNCTION	RESULTS AND EFFECTS ON SAFE PLANT OPERATION	CORRECTIVE ACTION
L49900	Diesel Generator Cooling Water Pump B.	Bent Actuating Arm on auxiliary contacts.	Cooling Water Pump did not start when HPCS Pump started.	Adjusted Actuating arm.
L49983	Various HCU's for Control Rod Drives.	Instrument block stop valves leaked at steam.	Potential to cause failure to scram the affected rods if combined with other events.	Replaced leaking valves.
L49988	HCU for CRD 26-03.	Instrument block stop valve leaked by stem.	Potential to cause failure to scram this rod if combined with other events.	Replace Valve.
LS0366	Division III Battery Charger.	High Voltage shutdown board set incorrectly.	Charger output would not reach desired value.	Reset high voltage shutdown board.
L50421	2A Diesel Generator.	Loose bolts on turbocharger.	Potential for degraded diesel operation.	Torqued bolts.
L50479	HCU for CRD 18-39.	Instrument block stop valve had a severe stem leak.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50514	HCU for CRD 50-31.	Instrument block stop valve had a stem leak.	Potential to cause failure to scram this rod if combined with other events.	Replaced valve.
L50602	RCIC Turbine.	Governor out of adjustment.	Could not control turbine speed or pump output.	Adjusted governor.
L50636	HCU for CRD 58-31.	Instrument Block stop valve leaked at packing.	Potential to cuase failure to scram this rod if combined with other events.	Replaced valve.

C. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for LaSalle Nuclear Power Station, Unit Two, logged during the reporting period, July 1 through July 31, 1985. This information is provided pursuant to the reportable occurrence reporting requirements as set forth in 10CFR 50.73.

Licensee Event Report Number	Date	Title of Occurrence
85-029-00	6-10-85	Pressure Switch 2B21-N037AA and 2B21-N037AB piped Backwards.
85-030-00	6-26-85	Group II Isoltion
85-031-00	6-22-85	RHR Shutdown Cooling High Suction Flow Isolation
85-032-00	7-1-85	Leak Detection Div. I & II RHR ΔT
85-033-00	7-1-85	RHR Shutdown Cooling Isolation.
85-034-00	6-25-85	Temporary Voltage Degradation During 237 Transformer Failure.

D. DATA TABULATIONS

The following data tabulations are presented in this report:

1. Operating Data Report

2. Average Daily Unit Power Level

3. Unit Shutdowns and Power Reductions

1. OPERATING DATA REPORT

DOCKET NO. 050-374
UNIT LaSalle Two
DATE August 10, 1985
COMPLETED BY Richard J. Rohrer
TELEPHONE (815)357-6761

OPERATING STATUS

1. REPORTING PERIOD: July, 1985 GROSS HOURS IN REPORTING PERIOD: 744
2. CURRENTLY AUTHORIZED POWER LEVEL (MWt):3323 MAX DEPEND CAPACITY (MWe-Net): 1036 DESIGN ELECTRICAL RATING (MWe-Net):1078
3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A
4. REASONS FOR RESTRICTION (IF ANY): N/A

		THIS MONTH	YR TO DATE	CUMULATIVE
5	NUMBER OF HOURS REACTOR WAS CRITICAL	266.6	1666.4	3278.2
6.	REACTOR RESERVE SHUTDOWN HOURS	0.0	0.0	125.3
7.	HOURS GENERATOR ON LINE	231.7	1629.0	3166.4
8.	UNIT RESERVE SHUTDOWN HOURS	0.0	0.0	0.0
9.	GROSS THERMAL ENERGY GENERATED (MWH)	493368	4880753	9388345
10.	GROSS ELEC. ENERGY GENERATED (MWH)	156348	1616735	3101721
11.	NET ELEC. ENERGY GENERATED (MWH)	141584	1515220	2907537
12.	REACTOR SERVICE FACTOR	35.8%	32.6%	47.6%
13.	REACTOR AVAILABILITY FACTOR	35.8%	32.6%	49.4%
14.	UNIT SERVICE FACTOR	31.3%	31.9%	46.0%
15.	UNIT AVAILABILITY FACTOR	31.1%	31.9%	46.0%
16.	UNIT CAPACITY FACTOR (USING MDC)	18.4%	28.6%	40.8%
17.	UNIT CAPACITY FACTOR(USING DESIGN MWe)	17.7%	27.5%	39.2%
18.	UNIT FORCED OUTAGE RATE	0.0%	0.0%	7.0%

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A
20. IF SHUT DOWN AT END OF REPORT PERIOD, ESTIMATED DATE OF STARTUP N/A

2. AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO: 050-374
UNIT: LASALLE TWO
DATE: August 10, 1985
COMPLETED BY: Richard J. Rohrer
TELEPHONE: (815) 357-6761
MONTH: July 1985

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	-13	17.	-17
2.	-13	18.	-17
3	-15	19.	-16
4.	-18	20.	-17
5.	-14	21.	-18
6.	-15	22.	80
7.	-15	23.	165
8.	-15	24.	300
9.	-14	25.	563
10.	-15	26.	807
11.	-18	27.	668
12.	-16	28.	869
13.	-17	29.	1003
14.	-17	30.	1016
15.	-17	31.	765
16.	-18		

ATTACHMENT E

3. UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 050-374
UNIT NAME LaSalle Two
DATE August 10,1985
COMPLETED BY Richard J. Rohrer
TELEPHONE (815)357-6761

REPORT MONTH JUNE 1985

NO.	DATE	TYPE F: FORCED S: SCHEDULED	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN THE REACTOR OR REDUCING POWER	CORRECTIVE ACTIONS/COMMENTS
3	850228	S	509.5	B	4	Maintenance and Surveillance outage continued from February.
4	850722	F	0.0	A	5	Turbine trip due to high level in MSR Drain Tank.
5	850723	S	0.0	B	5	Took Turbine off for RCIC Surveillance.

DOCUMENT 0044r/0005r

E. UNIQUE REPORTING REQUIREMENTS

1. Safety/Relief Valve Operations for Unit Two.

DATE	VALVES ACTUATED	NO & TYPE ACTUATIONS	PLANT CONDITION	DESCRIPTION OF EVENT
7-23-85	2B21-F013E	2 Manual	960 PSIG	Inadvertantly Opened during Set Pressure Verification Test.
7-23-85	2B21-F013N	1 Manual	960 PSIG	Inadvertantly Opened during Set Pressure Verification Test.

2. ECCS Systems Outages

The following outages were taken on ECCS Systems during the reporting period.

OUTAGE NO.	EQUIPMENT	PURPOSE OF OUTAGE
2-1013-85	LPCS Water Leg Pump	Lubrication
2-1019-85	2E12-F024A	Repair Limitorque
2-1023-85	2E12-F024A	Disconect Motor
2-1030-85	2A D/G	Lubrication
2-1059-85	A/B RHR Service Water Pumps	Polarization Test
2-1062-85	A/B RHR Shutdown Cooling	LIS-NB-211
2-1063-85	C/D RHR Service Water Pump	Surveillance

3. Off-Site Dose Calculation Manual

There were no changes to the off-site dose calculation manual during this reporting period.

4. Radioactive Waste Treatment Systems.
There were no changes to the radioactive waste treatment system during this reporting period.

Commonwealth Edison
LaSalle County Nuclear Station
Rural Route #1, Box 220
Marseilles, Illinois 61341
Telephone 815/357-6761

August 10, 1985

Director, Office of Management Information
and Program Control
United States Nuclear Regulatory Commission
Washington, D.C. 20555

ATTN: Document Control Desk

Gentlemen:

Enclosed for your information is the monthly performance report covering LaSalle County Nuclear Power Station for the period July 1 through July 31, 1985.

Very truly yours,

G. J. Diederich
Station Manager
LaSalle County Station

GJD/RJR/crh

Enclosure

xc: J. G. Keppler, NRC, Region III
NRC Resident Inspector LaSalle
Gary Wright, Ill. Dept. of Nuclear Safety
D. P. Galle, CECo
D. L. Farrar, CECo
INPO Records Center
Ron A. Johnson, PIP Coordinator SNED
J. E. Ellis, GE Resident
J. M. Nowicki, Asst. Comptroller
H. E. Bliss, Nuclear Fuel Services Manager
C. F. Dillon, Senior Financial Coordinator, LaSalle

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Document 0043r/0005r