

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
SAN ONOFRE NUCLEAR GENERATING STATION, UNIT 2DOCKET NUMBER (2)
0 5 0 0 0 3 6 1 1 OF 0 1

TITLE (4)

CPIS TRAIN 'A' ACTUATION DUE TO 2RT-7856 FAILURE

EVENT DATE (5)
MONTH DAY YEAR
0 6 1 0 8 5
LER NUMBER (6)
YEAR SEQ. NUMBER REV. NUMBER
8 5 0 3 6 0 0
REPORT DATE (7)
MONTH DAY YEAR
0 7 0 1 8 5
OTHER FACILITIES INVOLVED (8)
FACILITY NAMES
DOCKET NUMBER(S)
0 5 0 0 0 0 0 0OPERATING MODE (9)
1
POWER LEVEL (10)
1 0 0
THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §: (Check one or more of the following) (11)
20.402(b) X 50.73(a)(2)(iv) 73.71(b)
20.405(a)(1)(i) 50.36(c)(1) 50.73(a)(2)(v) 73.71(c)
20.405(a)(1)(ii) 50.36(c)(2) 50.73(a)(2)(vii) OTHER (Specify in Abstract below and in Text, NRC Form 366A)
20.405(a)(1)(iii) 50.73(a)(2)(i) 50.73(a)(2)(viii)(A)
20.405(a)(1)(iv) 50.73(a)(2)(ii) 50.73(a)(2)(viii)(B)
20.405(a)(1)(v) 50.73(a)(2)(iii) 50.73(a)(2)(x)

LICENSEE CONTACT FOR THIS LER (12)

NAME
H. E. MORGAN, STATION MANAGER
TELEPHONE NUMBER
AREA CODE
7 1 4 4 9 2 - 7 7 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NRC
B	VA	RIT	N305	Y					

SUPPLEMENTAL REPORT EXPECTED (14)

YES (If yes, complete EXPECTED SUBMISSION DATE) X NO
EXPECTED SUBMISSION DATE (15)
MONTH DAY YEAR

Abstract (Limit to 3400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 9, 1985, at 0825 with Unit 2 at 100% power and no containment purge in progress, in 'A' of the Containment Purge Isolation System (CPIS) (EIIS System Code VA) was actuated by an instrument failure signal from Containment Area Radiation Monitor 2RT-7856 (EIIS Component Code RIT). Since no containment purge was in progress, no CPIS components actuated.

The instrument failure was caused by loss of AC power to 2RT-7856. The loss of power occurred during an ongoing investigation for electrical noise (see LER 2-85-027) using current probes that clamp around the wires being tested. A wire from the vital bus (EIIS Component Code BU) power supply came loose when it was being tested. Inspection of the wire indicated that it had not been properly wrapped around the terminal lug prior to soldering and was connected by a small cold solder joint. The wire was re-soldered and power to the monitor was restored at 0945 of June 10, 1985. This was an isolated occurrence and no further action is planned at this time.

Since no containment purge was in progress and the system actuates on loss-of-power, there was no safety significance to the event. There are no reasonable or credible circumstances which could have increased the severity of this event.

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Southern California Edison Company

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SAN ONOFRE NUCLEAR GENERATING STATION

P.O. BOX 128

SAN CLEMENTE, CALIFORNIA 92672

July 1, 1985

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Subject: Docket No. 50-361
30-Day Report
Licensee Event Report No. 85-036
San Onofre Nuclear Generating Station, Unit 2

Pursuant to 10 CFR 50.73(a)(2)(iv), this submittal provides the required 30-day written Licensee Event Report (LER) for an occurrence involving an actuation of the Containment Purge Isolation System (CPIS). Neither the health and safety of plant personnel nor the health and safety of the public was affected by this event.

If you require additional information, please so advise.

Sincerely,


H. E. MORGAN
Station Manager

Enclosure: LER No. 85-036

cc: F. R. Huey (USNRC Senior Resident Inspector, Units 1, 2 and 3)
J. P. Stewart (USNRC Resident Inspector, Units 2 and 3)

J. B. Martin (Regional Administrator, USNRC Region V)

Institute of Nuclear Power Operations (INPO)

J. E. P.
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