Commonwealth Edison Company
Quad Cities Generating Station
22710 206th Avenue North
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ComEd

LWP-96-089

December 10, 1996

U.S. Nuclear Regulatory Commission
ATTN: Document Control Desk
Washington, D.C. 20555

SUBJECT: Quad Cities Nuclear Station Units 1 and 2
Monthly Performance Report
NRC Docket Nos. 50-254 and 50-265

Enclosed for your information is the Monthly Performance Report covering the operation of Quad-Cities Nuclear Power Station, Units One and Two, during the month of November 1996.

Respectfully,

ComEd
Quad-Cities Nuclear Power Station

L. W. Pearce
Station Manager

LWP/dak

Enclosure

cc: A. Beach, Regional Administrator
C. Miller, Senior Resident Inspector

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QUAD-CITIES NUCLEAR POWER STATION

UNITS 1 AND 2

MONTHLY PERFORMANCE REPORT

NOVEMBER 1996

COMMONWEALTH EDISON COMPANY

AND

MID-AMERICAN ENERGY COMPANY

NRC DOCKET NOS. 50-254 AND 50-265

LICENSE NOS. DPR-29 AND DPR-30

TABLE OF CONTENTS

I. INTRODUCTION

Quad-Cities Nuclear Power Station is composed of two Boiling Water Reactors and Steam Turbine/Generators, each with a Maximum Dependable Capacity of 769 MWe Net, located in Cordova, Illinois. The Station is jointly owned by Commonwealth Edison Company and Mid-American Energy Company. The Nuclear Steam Supply Systems are General Electric Company Boiling Water Reactors. The Architect/Engineer was Sargent & Lundy, Incorporated, and the primary construction contractor was United Engineers & Constructors. The Mississippi River is the condenser cooling water source. The plant is subject to license numbers DPR-29 and DPR-30, issued October 1, 1971, and March 21, 1972, respectively; pursuant to Docket Numbers 50-254 and 50-265. The date of initial Reactor criticalities for Units One and Two, respectively were October 18, 1971, and April 26, 1972. Commercial generation of power began on February 18, 1973 for Unit One and March 10, 1973 for unit Two.

This report was compiled by Kristal Moore and Debra Kelley, telephone number 309-654-2241, extensions 3070 and 2240, respectively.

II. SUMMARY OF OPERATING EXPERIENCE

A. Unit One

Quad Cities Unit One started the month of November 1996 reducing power to troubleshoot and repair the 1A Gland Seal Condenser Level Control Valve. Testing was completed and the unit ascended to full power on November 7, 1996.

B. Unit Two

Quad Cities Unit Two was on-line the entire month of November 1996. A few load drops were performed for Weekly Turbine Testing, however the average daily power level remained at 80% or greater.

III. PLANT OR PROCEDURE CHANGES, TESTS, EXPERIMENTS, AND SAFETY RELATED MAINTENANCE

A. Amendments to Facility License or Technical Specifications

There were no Amendments to the Facility License or Technical Specifications for the reporting period.

B. Facility or Procedure Changes Requiring NRC Approval

There were no Facility or Procedure changes requiring NRC approval for the reporting period.

C. Tests and Experiments Requiring NRC Approval

There were no Tests or Experiments requiring NRC approval for the reporting period.

IV. LICENSEE EVENT REPORTS

The following is a tabular summary of all licensee event reports for Quad-Cities Units One and Two occurring during the reporting period, pursuant to the reportable occurrence reporting requirements of 10CFR50.73.

UNIT 1

Licensee Event Report Number	Date	Title of occurrence
96-023	11/24/96	"B" Control Room HVAC Inoperable due to Make Up Flow Inaccuracy.

UNIT 2

Licensee Event Report Number	Date	Title of occurrence
96-003	11/12/96	U2 LPCI Inoperable due to 2D RHR Pump Discharge Check Valve Failure.

V. DATA TABULATIONS

The following data tabulations are presented in this report:

A. Operating Data Report

B. Average Daily Unit Power Level

C. Unit Shutdowns and Power Reductions

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-254
UNIT One
DATE December 10, 1996
COMPLETED BY Kristal Moore
TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 110196 2400 113096 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	3301.70	163765.20
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	3421.90
7. HOURS GENERATOR ON LINE	720.00	3025.70	158784.90
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	909.20
9. GROSS THERMAL ENERGY GENERATED (MWH)	1624770.00	6935335.90	345341133.50
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	528362.00	2211268.00	111825349.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	506112.00	2109582.00	105583958.00
12. REACTOR SERVICE FACTOR	100.00	41.07	75.83
13. REACTOR AVAILABILITY FACTOR	100.00	41.07	77.41
14. UNIT SERVICE FACTOR	100.00	37.63	73.52
15. UNIT AVAILABILITY FACTOR	100.00	37.63	73.94
16. UNIT CAPACITY FACTOR (Using MDC)	91.41	34.12	63.57
17. UNIT CAPACITY FACTOR (Using Design MWe)	89.09	33.26	61.96
18. UNIT FORCED OUTAGE RATE	0.00	3.72	7.52

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): N/A

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX C

OPERATING DATA REPORT

DOCKET NO. 50-265

UNIT Two

DATE December 10, 1996

COMPLETED BY Kristal Moore

TELEPHONE (309) 654-2241

OPERATING STATUS

1. REPORTING PERIOD: 0000 110196 2400 113096 GROSS HOURS IN REPORTING PERIOD: 720

2. CURRENTLY AUTHORIZED POWER LEVEL (MWt): 2511 MAX > DEPEND > CAPACITY: 769
DESIGN ELECTRICAL RATING (MWe-NET): 789

3. POWER LEVEL TO WHICH RESTRICTED (IF ANY) (MWe-Net): N/A

4. REASONS FOR RESTRICTION (IF ANY):

	THIS MONTH	YR TO DATE	CUMULATIVE
5. NUMBER OF HOURS REACTOR WAS CRITICAL	720.00	5772.10	159842.55
6. REACTOR RESERVE SHUTDOWN HOURS	0.00	0.00	2985.80
7. HOURS GENERATOR ON LINE	720.00	5605.80	155505.35
8. UNIT RESERVE SHUTDOWN HOURS	0.00	0.00	702.90
9. GROSS THERMAL ENERGY GENERATED (MWH)	1773379.00	13577339.90	337918560.82
10. GROSS ELECTRICAL ENERGY GENERATED (MWH)	571483.00	4310784.00	108456319.00
11. NET ELECTRICAL ENERGY GENERATED (MWH)	548225.00	4128036.00	102780922.00
12. REACTOR SERVICE FACTOR	100.00	71.79	74.55
13. REACTOR AVAILABILITY FACTOR	100.00	71.79	75.94
14. UNIT SERVICE FACTOR	100.00	69.72	72.53
15. UNIT AVAILABILITY FACTOR	100.00	69.72	72.85
16. UNIT CAPACITY FACTOR (Using MDC)	99.01	66.77	62.34
17. UNIT CAPACITY FACTOR (Using Design MWe)	96.50	65.07	60.76
18. UNIT FORCED OUTAGE RATE	0.00	30.28	11.22

19. SHUTDOWNS SCHEDULED OVER NEXT 6 MONTHS (TYPE, DATE, AND DURATION OF EACH): 3/1/97

20. IF SHUTDOWN AT END OF REPORT PERIOD < ESTIMATED DATE OF STARTUP:

21. UNITS IN TEST STATUS (PRIOR TO COMMERCIAL OPERATION): N/A

	FORECAST	ACHIEVED	
INITIAL CRITICALITY			
INITIAL ELECTRICITY			
COMMERCIAL OPERATION			

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-254
UNIT One
DATE December 10, 1996
COMPLETED BY Kristal Moore
TELEPHONE (309) 654-2241

MONTH November 1996

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	549	17.	784
2.	387	18.	784
3.	377	19.	784
4.	379	20.	784
5.	379	21.	785
6.	378	22.	785
7.	643	23.	779
8.	780	24.	785
9.	765	25.	785
10.	784	26.	785
11.	784	27.	783
12.	783	28.	785
13.	783	29.	785
14.	782	30.	779
15.	783	31.	
16.	767		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX B
AVERAGE DAILY UNIT POWER LEVEL

DOCKET NO 50-265
UNIT Two
DATE December 10, 1996
COMPLETED BY Kristal Moore
TELEPHONE (309) 654-2241

MONTH November 1996

DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)	DAY	AVERAGE DAILY POWER LEVEL (MWe-Net)
1.	502	17.	763
2.	772	18.	774
3.	702	19.	775
4.	774	20.	774
5.	776	21.	774
6.	777	22.	766
7.	776	23.	757
8.	776	24.	760
9.	774	25.	773
10.	773	26.	772
11.	774	27.	771
12.	775	28.	772
13.	774	29.	772
14.	775	30.	772
15.	775	31.	
16.	776		

INSTRUCTIONS
On this form, list the average daily unit power level in MWe-Net for each day in the reporting month. Compute to the nearest whole megawatt. These figures will be used to plot a graph for each reporting month. Note that when maximum dependable capacity is used for the net electrical rating of the unit, there may be occasions when the daily average power level exceeds the 100% line (or the restricted power level line). In such cases, the average daily unit power output sheet should be footnoted to explain the apparent anomaly.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-254

UNIT NAME One COMPLETED BY Kristal Moore

DATE December 10, 1996 REPORT MONTH November 1996 TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
96-05	11/1/96	F	0	A	9	---------	---	----	Repairs to the 1A Gland Seal Condenser Level Control Valve.

APPENDIX D
UNIT SHUTDOWNS AND POWER REDUCTIONS

DOCKET NO. 50-265

UNIT NAME Two COMPLETED BY Kristal Moore

DATE December 10, 1996 REPORT MONTH November 1996 TELEPHONE 309-654-2241

NO.	DATE	TYPE F OR S	DURATION (HOURS)	REASON	METHOD OF SHUTTING DOWN REACTOR	LICENSEE EVENT REPORT	SYSTEM CODE	COMPONENT CODE	CORRECTIVE ACTIONS/COMMENTS
									None for the month of November.

VI. UNIQUE REPORTING REQUIREMENTS

The following items are included in this report based on prior commitments to the Commission:

A. Main Steam Relief Valve Operations

There were no Main Steam Relief Valve Operations for the reporting period.

B. Control Rod Drive Scram Timing Data for Units One and Two

The basis for reporting this data to the Nuclear Regulatory Commission are specified in the surveillance requirements of Technical Specifications 4.3.C.1 and 4.3.C.2.

The following table is a complete summary of Units One and Two Control Rod Drive Scram timing for the reporting period. All scram timing as performed with reactor pressure greater than 800 PSIG.

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 01/01/96 TO 11/30/96

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		Technical Specification 4.3.D, 4.3.E & 4.3.F (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
1/13/96	29	0.339	0.708	1.459	2.538	2.91 (K10)	Scram Time Test for Viton Issue U-2
2/09/96	30	0.325	0.691	1.436	2.495	2.71 (M-8)	Scram Time Test for Viton Issue at EOC U1
2/25/96	18	0.349	0.722	1.496	2.577	3.07 (M-8)	Scram Time Test for Viton Issue U-2
4/22/96	1	0.32	0.69	1.46	2.54	2.54 (K-5)	PMTV for Scram Valve Leak on U-2
5/05/96	177	0.308	0.713	1.543	2.700	3.03 (N-7)	Beginning of Cycle for U-1
8/16/96	177	0.330	0.707	1.473	2.569	2.85 (F-6)	PMTV for SSPV Maintenace U-2
9/06/96	12	0.319	0.703	1.478	2.573	2.79 (F-6)	PMTV for various Maintenance U-1
9/26/96	10	0.358	0.734	1.497	2.602	2.78 (H-12)	STT for Viton Issue U-2
11/04/96	1	0.31	0.69	1.46	2.54	2.54 (E-6)	PMTV for Scram Valve U-1
11/08/96	10	0.36	0.73	1.50	2.57	2.68 (G-15)	STT for Viton Issue U-2

RESULTS OF SCRAM TIMING MEASUREMENTS
PERFORMED ON UNIT 1 & 2 CONTROL
ROD DRIVES, FROM 01/01/96 TO 11/30/96

DATE	NUMBER OF RODS	AVERAGE TIME IN SECONDS AT % INSERTED FROM FULLY WITHDRAWN				MAX. TIME FOR 90% INSERTION	DESCRIPTION
		5	20	50	90		Technical Specification 3.3.C.1 & 3.3.C.2 (Average Scram Insertion Time)
		0.375	0.900	2.00	3.5	7 sec.	
11/11/96	1	0.31	0.67	1.40	2.42	2.42 (F-15)	PMTV for F-15 on U-2

VII. REFUELING INFORMATION

The following information about future reloads at Quad-Cities Station was requested in a January 26, 1978, licensing memorandum (78-24) from D. E. O'Brien to C. Reed, et al., titled "Dresden, Quad-Cities and Zion Station--NRC Request for Refueling Information", dated January 18, 1978.

QTP 300-S32
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q1 Reload: 14 Cycle: 15

2. Scheduled date for next refueling shutdown: 2/10/98

3. Scheduled date for restart following refueling: 4/1/98

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

   No

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   Approx. 216 SPC 9X9IX Fuel Bundles Q1R15 will be loaded.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 1933

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3657

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

APPROVED
OCT 30 1989
Q.C.O.S.R.

QTP 300-532
Revision 2
October 1989

QUAD CITIES REFUELING INFORMATION REQUEST

1. Unit: Q2 Reload: 13 Cycle: 14

2. Scheduled date for next refueling shutdown: 3/1/97

3. Scheduled date for restart following refueling: 5/23/97

4. Will refueling or resumption of operation thereafter require a Technical Specification change or other license amendment:

5. Scheduled date(s) for submitting proposed licensing action and supporting information:

   August 1996

6. Important licensing considerations associated with refueling, e.g., new or different fuel design or supplier, unreviewed design or performance analysis methods, significant changes in fuel design, new operating procedures:

   216 Siemens 9X9IX Power Corporation Fuel Bundles will be loaded during Q2R14.

7. The number of fuel assemblies.

   a. Number of assemblies in core: 724

   b. Number of assemblies in spent fuel pool: 2727

8. The present licensed spent fuel pool storage capacity and the size of any increase in licensed storage capacity that has been requested or is planned in number of fuel assemblies:

   a. Licensed storage capacity for spent fuel: 3897

   b. Planned increase in licensed storage: 0

9. The projected date of the last refueling that can be discharged to the spent fuel pool assuming the present licensed capacity:

APPROVED
OCT 30 1989
Q.C.O.S.R.

VIII. GLOSSARY

The following abbreviations which may have been used in the Monthly Report, are defined below:

ACAD/CAM - Atmospheric Containment Atmospheric Dilution/Containment Atmospheric Monitoring
ANSI - American National Standards Institute
APRM - Average Power Range Monitor
ATWS - Anticipated Transient Without Scram
BWR - Boiling Water Reactor
CRD - Control Rod Drive
EHC - Electro-Hydraulic Control System
EOF - Emergency Operations Facility
GSEP - Generating Stations Emergency Plan
HEPA - High-Efficiency Particulate Filter
HPCI - High Pressure Coolant Injection System
HRSS - High Radiation Sampling System
IPCLRT - Integrated Primary Containment Leak Rate Test
IRM - Intermediate Range Monitor
ISI - Inservice Inspection
LER - Licensee Event Report
LLRT - Local Leak Rate Test
LPCI - Low Pressure Coolant Injection Mode of RHRs
LPRM - Local Power Range Monitor
MAPLHGR - Maximum Average Planar Linear Heat Generation Rate
MCPR - Minimum Critical Power Ratio
MFLCPR - Maximum Fraction Limiting Critical Power Ratio
MPC - Maximum Permissible Concentration
MSIV - Main Steam Isolation Valve
NIOSH - National Institute for Occupational Safety and Health
PCI - Primary Containment Isolation
PCIOMR - Preconditioning Interim Operating Management Recommendations
RBCCW - Reactor Building Closed Cooling Water System
RBM - Rod Block Monitor
RCIC - Reactor Core Isolation Cooling System
RHRS - Residual Heat Removal System
RPS - Reactor Protection System
RWM - Rod Worth Minimizer
SBGTS - Standby Gas Treatment System
SBLC - Standby Liquid Control
SDC - Shutdown Cooling Mode of RHRS
SDV - Scram Discharge Volume
SRM - Source Range Monitor
TBCCW - Turbine Building Closed Cooling Water System
TIP - Traversing Incore Probe
TSC - Technical Support Center