



UNITED STATES
NUCLEAR REGULATORY COMMISSION
WASHINGTON, D.C. 20555-0001

RHODE ISLAND ATOMIC ENERGY COMMISSION

DOCKET NO. 50-193

AMENDMENT TO FACILITY LICENSE

Amendment No. 22
License No. R-95

1. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by the Rhode Island Atomic Energy Commission, (the licensee), dated December 5, 1996, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the Commission's rules and regulations as set forth in 10 CFR Chapter I;
 - B. The facility will operate in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the Commission's regulations;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public;
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the Commission's regulations and all applicable requirements have been satisfied; and
 - F. Prior notice of this amendment was not required by 10 CFR 2.105 and publication of notice for this amendment is not required by 10 CFR 2.106.

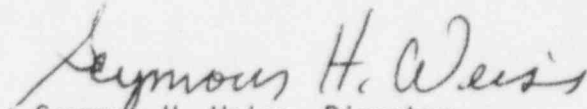
2. Accordingly, the license is amended by change of the Technical Specifications as indicated in the enclosure to this license amendment, and paragraph 3.b of Facility License No. R-95 is hereby amended to read as follows:

3.b Technical Specifications

The Technical Specifications contained in Appendix A, as revised through Amendment No. 22 are hereby incorporated in this license. The licensee shall operate the facility in accordance with the Technical Specifications.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Reactor Program Management
Office of Nuclear Reactor Regulation

Enclosure:
Appendix A Technical
Specifications Change

Date of Issuance: December 18, 1996

ENCLOSURE TO LICENSE AMENDMENT NO. 22

FACILITY LICENSE NO. R-95

DOCKET NO. 50-193

Replace the following pages of the Appendix A Technical Specifications with the enclosed pages. The revised pages are identified by amendment number and contain vertical lines indicating the areas of change.

Remove Page

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Insert Page

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4.0 SURVEILLANCE REQUIREMENTS

Surveillance tests for Reactivity Limits (4.1), Reactor Safety System (4.2), Surveillance of Experiments (4.8) and Reactor Components (4.9) may be deferred for periods of reactor shutdown providing they are performed prior to restart (ANS 15.1, 4.1). Surveillance tests for the following will be performed as stated in the appropriate sections:

Water Coolant System (4.3)

Confinement and Emergency Exhaust System (4.4, 4.5, 4.6)

Radiation Monitoring System and Effluents (4.7)

4.1 Reactivity Limits

Applicability:

This specification applies to the surveillance requirements for reactivity limits.

Objective:

To assure that the reactivity limits of Specification 3.1 are not exceeded.

Specification:

1. Shim safety blade reactivity worths and insertion rates shall be measured:

a. annually;*

b. whenever the core is changed from the startup core to the three other cores as analyzed and specified in the SAR (Part A, Section V).

2. Shim safety blades shall be visually inspected and checked for swelling at least annually.

*TO BE CONDUCTED AT 16 MONTH INTERVAL IN JANUARY 1997

- a. Prior to each reactor startup following a period when the reactor was secured;
 - b. After a channel has been repaired or deenergized.
2. A channel calibration of the safety channels listed in Table 3.1, which can be calibrated, shall be performed annually.
 3. The radiation monitoring system required in Table 3.2 shall be operable prior to every reactor startup for which safety system channel tests are required as in 4.2.1. If the system has been repaired, the system shall be operable prior to use.
 4. Shim safety blade release-drop time shall be measured annually.
 5. Shim safety rod release-drop time shall be measured whenever the shim safety rod's core location is changed or whenever maintenance is performed which could effect the rod's drop time. (Specification 3.2.3)
 6. Shutdown Margin (Specification 3.1.1)

The shutdown margin shall be determined annually. * It shall be determined when a new core is configured as described in the SAR (Part A, Section V). The determination will be made in accordance with operating procedures.

7. Excess Reactivity (Specification 3.1.2)

The excess reactivity shall be determined annually. * It shall be determined when a new core is configured as described in the SAR (Part A, Section V). The determination will be made in accordance with operating procedures.

*TO BE CONDUCTED AT 16 MONTH INTERVAL IN JANUARY 1997

8. Reactivity Insertion Rate (Specification 3.2.4)

The reactivity insertion rate shall be measured annually. * It shall be determined when a new core is configured as described in the SAR (Part A, Section V). The determination will be made in accordance with written procedures.

*TO BE CONDUCTED AT 16 MONTH INTERVAL IN JANUARY 1997

Bases:

Prestartup tests of the safety system channels assure their operability. Annual calibration detects any long term drift that is not detected by normal intercomparison of channels. The channel operability check of the neutron flux level channels assures that the detectors are properly adjusted to accurately monitor the parameter they are measuring.

Radiation monitors are checked for proper operation in Specification 4.2.3. Calibration and setpoint verification involve use of a calibration source and significant personnel radiation exposure. It is determined that annual calibration of radiation monitors is adequate since they displayed excellent stability over many years of operation.

The measured release-drop times of the shim safety blades have been consistent over many years. Annual check of these parameters is considered adequate to detect any deterioration which could change the release-drop time. Binding or rubbing caused by rod misalignment, could result from maintenance; therefore, release drop times will be checked after such maintenance.

4.3 Water Coolant System

a. Primary Coolant System

Applicability:

This specification applies to the surveillance of the primary coolant system.