NRC Form 366
(9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)	DOCKET NUMBER (2)	PAGE (3)
Shoreham Nuclear Power Station Unit #1	0 5 0 0 0 3 2 2	1 OF 0 3

TITLE (4)
RPS Actuation due to High RPV Pressure

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)	
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES	DOCKET NUMBER(S)
0 6	0 6	8 5	8 5	0 2 2	0 0	0 7	0 3	8 5		0 5 0 0 0
										0 5 0 0 0

OPERATING MODE (9)	THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR § (Check one or more of the following) (11)			
4	20.402(b)	20.405(c)	X 50.73(a)(2)(iv)	73.71(b)
POWER LEVEL (10)	20.405(a)(1)(i)	50.36(c)(1)	50.73(a)(2)(v)	73.71(c)
0 0 0	20.405(a)(1)(ii)	50.36(c)(2)	50.73(a)(2)(vii)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)
	20.405(a)(1)(iii)	50.73(a)(2)(i)	50.73(a)(2)(viii)(A)	
	20.405(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)	
	20.405(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(x)	

LICENSEE CONTACT FOR THIS LER (12)

NAME	TELEPHONE NUMBER
Gary G. Rhoads, Operational Compliance Engineer	AREA CODE 5 1 6 9 2 9 - 8 3 0 0

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS		CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPRDS	

SUPPLEMENTAL REPORT EXPECTED (14)

		EXPECTED SUBMISSION DATE (15)	MONTH	DAY	YEAR
YES (If yes, complete EXPECTED SUBMISSION DATE)	X NO				

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

On June 6, 1985 at 1513 a RPS actuation occurred. The plant was in Operational Condition 4 with the mode switch in refuel and all rods inserted in the core. Operators were performing an inservice reactor pressure boundary leak test utilizing both the CRD (Control Rod Drive) system and the RWCU (Reactor Water Cleanup) blow down valve (G33-HCV-004) to regulate and maintain the RPV pressure at 1005 psig. I&C technicians were inspecting the blow down valve for leakage per an approved Maintenance Work Request (MWR). When the technician touched the feedback arm, (which relays the actual valve position to the controller) due to the amount of play in the arm, the linkage was allowed to move up, telling the controller that the valve was not closed. The valve then closed enough to cause an increase in RPV pressure above the high pressure scram trip set point (1043 psig) resulting in a high pressure scram. Control Room Operators followed the Emergency Shutdown Procedure. To prevent reccurence, the valve positioner was adjusted and the incident was discussed with the entire Operations and I&C departments with a report generated, covering the incident in detail, and placed on their required reading list.

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

On June 6, 1985 at 1513 a RPS actuation occurred. The plant was in Operational Condition 4 with the mode switch in refuel and all rod inserted in the core. Operators were performing an inservice reactor pressure boundary leak test (SP 22.009.01) utilizing both the CRD system and the RWCU blow down valve to regulate and maintain the RPV pressure at 1005 psig.

The initiating event occurred due to exceeding the RPV high pressure trip set point resulting in a high pressure scram. There was no safety significance to this event. All plant systems functioned as designed and no ECCS systems were challenged or required. The operators carried out all required actions. The maximum RPV pressure was 1047 psig, which was well below the lowest SRV (Safety Relief Valve) setpoint.

Reactor Pressure was being controlled at 1005 psig in accordance with SP 22.009.01 (inservice reactor pressure boundary leak test). The blow down valve was throttled to a flow rate of 80 gpm with the controller set at 0 (valve closed). Pressure control was therefore being achieved by varying the CRD cooling flow. Per an approved MWR, two I&C technicians were dispatched to inspect the blow down valve for leakage but were told not to reposition the valve. The technician, upon looking at the valve, touched the linkage of the feedback arm (which relays the actual valve position to the controller) and due to the amount of free play in the linkage, the arm moved up telling the controller that the valve was not closed. The controller immediately reacted by bleeding air from the control loop (the valve operates on 3 to 15 psig control air signal with 3 psig for the closed position) and the valve started to close. The valve was closed long enough to cause pressure to increase to the high pressure scram trip point (1043 psig) resulting in a high pressure scram. The linkage arm then returned to the original position causing the controller to admit more air to the control loop and opened the valve to its original equilibrium point. The RPV pressure decreased to its original pressure. All work on the valve was ceased and Control Room Operators followed the emergency shutdown procedure (SP 29.010.01).

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TEXT (If more space is required, use additional NRC Form 366A's) (17)

To prevent recurrence the following actions were taken;

1) The incident was discussed in detail with the technicians involved and the entire I&C section.

2) To insure its proper operation, the positioner was readjusted.

3) A report was generated, describing the incident in detail, and placed on the Operations and I&C Sections required reading list. It was discussed in the report that the technicians should not have touched the valve linkage, and should not have been called to work on a valve that was being used to control reactor pressure.

LONG ISLAND LIGHTING COMPANY

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July 3, 1985 PM 85-118

U.S. Nuclear Regulatory Commission
Document Control Desk
Washington, D.C. 20555

Dear Sir:

In accordance with 10CFR50.73, enclosed is a copy of Shoreham Nuclear Power Station Unit 1's Licensee Event Report 85-022.

Sincerely yours,

W Steiger

William E. Steiger, Jr.
Plant Manager

WES/gr

Enclosure

cc: Dr. Thomas E. Murley, Regional Administrator
John Berry, Senior Resident Inspector
Institute of Nuclear Power Operations, Records Center
American Nuclear Insurers

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